



John C. Brons
Senior Vice President
Nuclear Generation

August 29, 1986
IPN-86-42

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Mr. Steven A. Varga, Director
PWR Project Directorate No. 3
Division of PWR Licensing-A

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
Multi-Plant Actions (MPAs) Licensing Status
Supplemental Information

- References:
1. NRC letter (J. D. Neighbors) to NYPA (J. C. Brons) dated May 1, 1986, concerning IP-3 MPAs Listing.
 2. NYPA letter (J. C. Brons) to NRC (S. A. Varga) dated May 30, 1986, MPAs Status.

Dear Sir:

Reference 1 forwarded the NRC listing of Multi-Plant Actions (MPAs) applicable to the Indian Point 3 facility. This listing was provided with a request that the Authority review these items and provide the associated completion or projected completion dates as appropriate. Reference 2 provided the Authority's partial response in the form of a listing markup. The purpose of this letter is to provide a status for each of the remaining MPAs.

Attachment I to this letter consists of a complete listing of MPAs. The bar (|) in the notes column indicates that the status and date was added for this submittal. The dates provided are based on the latest Authority letters to the NRC, license amendment effective dates, modification packages, plant procedures, NRC Safety Evaluation Reports, various internally generated documents or the expected completion dates. In some cases the Authority and the NRC Project Manager were unable to ascertain the status of the MPA due to the lack of information concerning the nature of the MPA.

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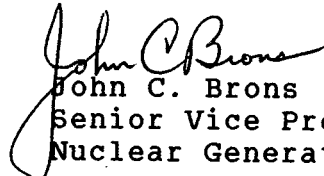
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Due to the unscheduled outage that Indian Point 3 is currently experiencing, the Cycle 5/6 refueling outage which was originally scheduled for February 1987 will be delayed. Therefore, some of the projected dates provided in Reference 2 were revised. The double bar (||) in the notes column indicates that the status and/or date has been revised.

The dates provided in Attachment I are consistent with the Authority's understanding of the specific issue and associated action.

Should you or your staff have any questions regarding this matter, please contact Mr. P. Kokolakis of my staff.

Very truly yours,


John C. Brons
Senior Vice President
Nuclear Generation

cc: Resident Inspector's Office
Indian Point Unit 3
U.S. Nuclear Regulatory Commission
Buchanan, NY 10511

Mr. J. D. Neighbors, Senior Project Manager
PWR Project Directorate No. 3
Division of PWR Licensing-A
U.S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, MD 20014

ATTACHMENT I

Multi - Plant Actions Status

NEW YORK POWER AUTHORITY
INDIAN POINT 3 NUCLEAR POWER PLANT
DOCKET NO. 50-286

REGULATORY INFORMATION TRACKING SYSTEM
FACILITY IMPLEMENTATION

INDIAN POINT 3

LOCATION: 25 MI N OF NEW YORK CITY, NY
 NUMBER: 050-00086
 REEK: UEC
 TOR: P. KUITAY

LICENSED POWER: 3025 MWT
 DESIGN POWER: 0965 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. NEIGHBORS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: L. PARRISH

MPA #	TITLE	LIC ACT COMPLETE	LICENSEE		NOTES
			STATUS	DATE	
A-01	10 CFR 50.55 A(G) - ISI	06/13/83C	C	7/25/86	
A-02	APPENDIX I - ALARA	12/07/84C	C	1/1/85	
A-05	SECURITY REVIEWS - MODIFIED AMENDMENT PLANS	02/23/79C	C	2/23/79	
A-09	PRESSURE VESSEL BELTLINE MATERIAL SURVEILLANCE	01/19/80C	C	1/19/80	
A-10	CONTINGENCY PLANNING	09/12/80C	C	10/11/80	
A-11	GUARD TRAINING PLANS	02/16/83C	C	2/85	
A-12	VITAL AREA ANALYSIS	04/13/82C	C	4/23/81	
A-14	10CFR 50.55 A(G) - INSERVICE TESTING	06/13/83C	P	10/86	
A-16	QUALIFICATIONS OF EXAMINATION, INSPECTION, TESTING	07/12/82C	C	7/12/82	
A-16	QUALIFICATIONS OF EXAMINATION, INSPECTION, TESTING	09/29/82C	NOT APPLICABLE		
A-17	INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT	05/21/86	P	1987	
A-18	TECH SPECS AFFECTED BY 50.72 AND 50.73 (GL 83-43)	07/01/85C	C	7/1785	
A-20	10 CFR 50.62 OPERATING REACTOR REVIEWS	09/30/86	P	1988	
A-21	10 CFR 50.61, PRESSURIZED THERMAL SHOCK RULE	12/31/86	C	1/23/86	
B-01	DIESEL GENERATOR LOCKOUT	12/21/78C	C	12/21/78	
B-02	FIRE PROTECTION	03/06/79C	C	2/28/80	
B-03	PWR MODERATOR DILUTION	02/21/79C	C	2/21/79	
B-04	REACTOR VESSEL OVERPRESSURE PROTECTION	06/28/84C	P	3/10/86	
B-08	PWR HPSI-LPSI FLOW RESISTANCE	02/05/79C	C	2/5/79	
B-14	SEA GUIDE TUBE WEAR	07/03/80C	C	7/3/80	

REGULATORY INFORMATION TRACKING SYSTEM
FACILITY IMPLEMENTATION

(CONTINUATION)

P.A. #	TITLE	LIC ACT COMPLETE	LICENSEE		NOTES
			STATUS	DATE	
B-44	LESSONS LEARNED IMPLEMENTATION	03/26/80C	C	2/21/80	
B-44	LESSONS LEARNED IMPLEMENTATION	03/26/80C	C	2/21/80	
B-44	LESSONS LEARNED IMPLEMENTATION	03/26/80C	C	2/21/80	
B-44	LESSONS LEARNED IMPLEMENTATION	03/26/80C	C	2/21/80	
B-44	LESSONS LEARNED IMPLEMENTATION	03/26/80C	C	2/21/80	
B-44	LESSONS LEARNED IMPLEMENTATION	06/02/80C	C	2/21/80	
B-44	LESSONS LEARNED IMPLEMENTATION	03/26/80C	C	2/21/80	
B-45	HASH 1900 EVENT V, "PRIMARY COOLANT SYSTEM PRESSUR	02/11/80C	C	2/11/80	
B-46	ANALYSIS OF TURBINE DISC CRACKS	12/02/81C	C	12/2/81	(2)
B-48	ADEQUACY OF STATION ELECTRIC DISTRIBUTION VOLTAGE	04/09/85C	C	4/9/85	
B-57	LHR CAPABILITY	04/08/85C	C	4/8/85	
B-59	MASONRY WALL DESIGN	09/22/83C	C	9/23/83	
B-60	ENVIRONMENTAL QUALIFICATION	06/05/85C	C	10/85	
B-61	LOSS OF NON CLASS IE ISC POWER	08/13/82C	C	7/30/80	
B-62	EGF RESET CONTROL DESIGN DEFICIENCY	06/03/82C	C	5/17/83	
B-63	INTERIM PROCEDURES FOR SHORT TERM BLACKOUT	02/26/81C	C	12/11/81	
B-66	NATURAL CIRCULATION COOLDOWN	06/10/83C	C	10/85	
B-69	TJB 80-4 MAINSTEAM LINE BREAK WITH CONTINUED FEEDW	06/10/83C	C	6/10/83	
B-70	FATIGUE TRANSIENT LIMIT TS	05/25/82C	NOT APPLICABLE		
B-72	NUREG 0737 TECH SPECS (GENERIC LETTER 82-16)	03/13/85C	C	3/15/85	(3)
B-76	ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION &	07/05/85C	C	12/85	
B-77	ITEM 2.1 - EQUIPMENT CLASSIFICATION & VENDOR INTER	10/01/86P	P	1/87	
B-78	ITEMS 3.1.1 & 3.1.2 -POST MAINTENANCE TEST PROCEDU	03/03/86C	P	1/87	

REGULATORY INFORMATION TRACKING SYSTEM
FACILITY IMPLEMENTATION

POINT 3

(CONTINUATION)

MPA #	TITLE	LIC ACT COMPLETE	LICENSEE		NOTES
			STATUS	DATE	
B-79	ITEM 3.1.3 - POST MAINTENANCE TESTING - CHANGES TO	11/04/85C	C	11/4/85	
B-80	ITEM 4.1 - REACTOR TRIP SYSTEM RELIABILITY - VENDO	03/03/86C	C	3/3/86	
B-81	ITEMS 4.2.1 & 4.2.2 -PREVENTATIVE MAINT PROG FOR R	12/11/85C	C	12/11/85	
B-82	ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTAC	07/10/85C	P	1987	
B-83	TECH SPECIFICATIONS COVERED BY GENERIC LETTERS 83-	05/15/86	C	4/18/86	(4)
B-85	SALEM ATWS 1.2 DATA CAPABILITY	06/10/85C	C	10/85	(5)
B-86	SALEM ATWS 2.2 S-R COMPONENTS	10/30/86	P	4/87	
B-87	SALEM ATWS 3.2.1 & 3.2.2 S-R COMPONENTS	03/03/86C	P	4/87	
B-88	SALEM ATWS 3.2.3 T.S. S-R COMPONENTS	11/04/85C	C	11/4/85	
B-89	SALEM ATWS 4.2.3 & 4.2.4 LIFE COMPONENTS	11/30/86	P	1987	
B-90	SALEM ATWS 4.3 W AND B&W T.S.	06/01/86	P	9/86	
B-92	SALEM ATWS 4.5.1 DIVERSE TRIP FEATURES	03/03/86C	C	3/3/86	
B-93	SALEM ATWS 4.5.2 & 4.5.3 TEST ALTERNATIVES	04/31/87	P	1987	
C-01	PHR SECONDARY WATER CHEMISTRY MONITORING REQUIREME	02/21/80C	C	2/21/80	
C-07	FUEL HANDLING ACCIDENT INSIDE CONTAINMENT	06/13/80C	C	6/13/80	
C-10	CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (PHASE	02/13/85C	C	2/13/85	
C-14	AUXILIARY FEEDWATER SEISMIC QUALIFICATION	04/27/83C	C	10/85	
C-15	CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA	06/28/85C	C	6/28/85	
C-16	UTILITY RESPONSES TO STAFF RECOMMENDATIONS CONCERN	12/11/85C	C	6/18/85	
D-02	ECCS ZIRC CLAD MODEL ERROR-COMPLIANCE WITH 10 CFR-	08/11/78C	C	8/11/78	
D-03	PRESSURIZER HEATUP RATE ERROR	11/06/78C	C	11/6/78	
D-10	ASYMMETRIC LOCA LOADS	03/10/86C	C	3/10/86	
D-11	FISSION GAS RELEASE	02/22/80C	C	3/10/80	

REGULATORY INFORMATION TRACKING SYSTEM
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MPA #	TITLE	LIC ACT COMPLETE	LICENSEE		NOTES
			STATUS	DATE	
			NO INFORMATION AVAILABLE		
D-15	HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAIL	02/24/81C			
D-16	REVIEW OF CORPORATE MANAGEMENT CAPABILITY	09/05/80C	C	9/5/80	
D-17	DEFINITION OF OPERABLE	09/05/80C	C	5/28/85	(6)
D-19	DIESEL GENERATOR RELIABILITY TECH SPECS (GL 84-15)	09/06/85C	C	10/1/84	
E-05	H H-1 LOOP OPERATION	09/24/80C	NOT APPLICABLE		
F-01	I.A.1.1 SHIFT TECHNICAL ADVISOR	01/19/82C	C	1/8/82	
F-02	I.A.1.3 SHIFT MANNING	06/30/83C	C	1/1/84	
F-03	I.A.2.1 UPGRADING OF RO AND SRO TRAINING	12/28/82C	C	12/28/82	
F-04	I.C.1.2/3.A EMERGENCY OPERATING PROCEDURES GENERIC	10/03/83C	C	10/85	
F-05	I.C.1.2/3.B PROCEDURES GENERATION PACKAGES (NUREG	01/31/87	C	8/29/84	
F-06	I.C.5 FEEDBACK OF OPERATING EXPERIENCE	11/05/81C	C	11/5/83	
F-07	I.C.6 CORRECT PERFORMANCE OF OPERATING ACTIVITI	11/05/81C	C	11/5/81	
F-08	I.D.1.1 DETAILED CONTROL ROOM DESIGN REVIEW PROGRA	10/24/84C	C	1/2/85	
F-09	I.D.2 SAFETY PARAMETER DISPLAY SYSTEM	08/15/86	C	10/85	
F-10	II.B.1 RCS HIGH POINT VENTS	09/19/83C	C	10/85	
F-11	II.B.2 PLANT SHIELDING	02/28/84C	C	10/85	
F-12	II.B.3 POST ACCIDENT SAMPLING	07/01/85C	C	1/30/86	
F-13	II.B.4 TRAINING FOR MITIGATING CORE DAMAGE	12/28/82C	C	12/28/82	
F-14	II.D.1 RV AND SV TRAINING	03/30/86	P	1987	
F-15	II.E.1.1 AFW SYSTEM EVALUATION	05/08/80C	C	12/30/80	
F-16	II.E.1.2.1 AFW SYSTEM INITIATION	08/13/82C	C	8/10/82	
F-17	II.E.1.2.2 AFW SYSTEM FLOW INDICATION	08/13/82C	C	8/10/82	
F-18	II.E.4.1 DEDICATED HYDROGEN PENETRATION	12/01/81C	C	12/1/81	

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MPA #	TITLE	LIC ACT COMPLETE	LICENSEE		NOTES
			STATUS	DATE	
F-19	II.E.4.2 CONTAINMENT ISOLATION DEPENDABILITY	11/09/82C	C	10/85	
F-20	II.F.1.1 NOBLE GAS MONITOR	03/17/82C	C	10/85	
F-21	II.F.1.2 IODINE/PARTICULATE SAMPLING	03/17/82C	C	3/17/82	
F-22	II.F.1.3 CONTAINMENT HIGH RANGE MONITOR	01/19/82C	C	10/85	
F-23	II.F.1.4 CONTAINMENT PRESSURE INSTRUMENT	06/14/83C	C	9/81	
F-24	II.F.1.5 CONTAINMENT WATER LEVEL INSTRUMENT	06/14/83C	C	10/85	
F-25	II.F.1.6 CONTAINMENT HYDROGEN MONITOR	06/14/83C	C	10/85	
F-26	II.F.2 INSTRUMENTS FOR DETECTION ON INADEQUATE	04/30/87	P	1987	
F-30	II.K.2.13 THERMAL-MECHANICAL REPORT	06/15/84C	C	6/15/84	
F-33	II.K.2.17 POTENTIAL FOR VOIDING IN RCS	01/18/84C	C	1/18/84	
F-36	II.K.3.1 AUTO PORV ISOLATION	12/15/83C	C	3/13/81	
F-37	II.K.3.2 REPORT ON PORV FAILURES	12/15/83C	C	3/13/81	
F-38	II.K.3.3 REPORTING SV AND RV FAILURES AND CHALLENGE	03/05/82C	C	3/4/82	
F-39	II.K.3.5 AUTO TRIP OF RCPS	02/22/83C	C	2/8/83	
F-40	II.K.3.9 PID CONTROLLER	10/29/81C	C	10/29/81	
F-41	II.K.3.10 ANTICIPATORY TRIP MODIFICATIONS	10/29/81C	C	10/29/81	
F-42	II.K.3.12 ANTICIPATORY TRIP ON TURBINE TRIP	10/29/81C	C	10/29/81	
F-47	II.K.3.17 ECCS OUTAGES	09/30/83C	C	8/15/83	
F-53	II.K.3.25 POWER ON PUMP SEALS	12/18/84C	C	12/18/84	
F-57	II.K.3.30 SB LOCA METHODS	05/24/85C	C	5/24/85	
F-58	II.K.3.31 COMPLIANCE WITH 10 CFR 50.46	01/31/87	C	8/18/86	
F-63	III.A.1.2 TECHNICAL SUPPORT CENTER	02/01/87	C	10/85	
F-64	III.A.1.2 OPERATIONAL SUPPORT CENTER	02/01/87	C	10/85	

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FA #	TITLE	LIC ACT COMPLETE	LICENSEE		NOTES	
			STATUS	DATE		
F-65	III.A.1.2 EMERGENCY OPERATIONS FACILITY	02/01/87	C	10/85	 	
F-66	III.A.1.2 NUCLEAR DATA LINK	07/30/82C	C	6/81		
F-67	III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE	10/03/83C	C	2/27/81		
F-68	III.A.2.2 METEOROLOGICAL DATA UPGRADE	01/31/87	C	8/11/80		
F-69	III.D.3.3 INPLANT RADIATION MONITORING	02/26/82C	C	2/9/82		
F-70	III.D.3.4 CONTROL ROOM HABITABILITY	01/27/82C	C	4/86		
F-71	I.D.1.2 DETAILED CONTROL ROOM REVIEW (FOLLOWUP TO	09/30/86	P	1989		
G-61	REACTOR COOLANT PUMP TRIP (D660 - II.K.3.5)	04/14/86	C	12/85		
						(7)

NOTES

- (1) NRC letter dated 2/21/80 accepted NYPA's position on the Lessons Learned items. Many of these items evolved later into NUREG-0737 action items.
- (2) Additional analysis submitted via NYPA 5/9/86 letter.
- (3) Item complete except for NRC approval of NUREG-0737, Item II.K.3.12 "Anticipatory Reactor Trip Upon Turbine Trip" Tech. Specs. submitted via NYPA's 4/30/86 letter.
- (4) Item complete except for NUREG-0737 Items II.F.2 and III.D.3.4 (RVLIS and Control Room Habitability) Tech. Specs.
- (5) Acceptance testing in progress.
- (6) NRC reopened and subsequently resolved this item by letter dated 5/28/85.
- (7) Awaiting NRC final SER.

C = Closed

P = Projected

The bar (|) in the Notes column indicates that the date associated with the item was provided by this letter.

The double bar (||) in the Notes column indicates that the date projected in NYPA's 5/30/86 letter has been revised.