



John C. Brons
Senior Vice President
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August 18, 1986
IPN-86-39

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Mr. Steven A. Varga, Director
PWR Project Directorate No. 3
Division of PWR Licensing-A

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
NUREG-0737, Item II.K.3.31, "Plant-Specific
Calculation to Show Compliance with
10 CFR 50.46"

- References:
- 1) NRC Generic Letter 83-35, "Clarification of TMI Action Plan Item II.K.3.31," dated November 2, 1983.
 - 2) Letter from John C. Brons to Steven A. Varga, dated July 9, 1985, (IPN-85-36), entitled: "NUREG-0737, Item II.K.3.31, "Plant-Specific Calculations to Show Compliance with 10 CFR 50.46."
 - 3) Letter from L. D. Butterfield (WOG) to Jim Lyons (NRC), dated June 11, 1986, (OG-190), entitled: "Westinghouse Owners Group, Transmittal of WCAP - 1145, "Westinghouse Small Break LOCA ECCS Evaluation Model Generic Study with the NOTRUMP Code."

Dear Sir:

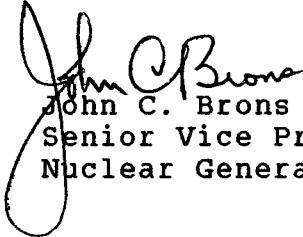
Reference 1 stated that NUREG-0737 item II.K.3.31 may be satisfied by generic analyses demonstrating that the small break LOCA results calculated by the WFLASH model are conservative compared to those results calculated by the NOTRUMP model. Reference 2 stated that as a member of the Westinghouse Owners Group (WOG), the Authority would be referencing the analyses to be performed by the WOG addressing the requirements of NUREG-0737 item II.K.3.31 on a generic basis. The results of the WOG generic small break LOCA study utilizing the NRC approved NOTRUMP model

are provided in WCAP-11145, "Westinghouse Small Break LOCA ECCS Evaluation Model Generic Study with the NOTRUMP Code (Proprietary)." This topical report was transmitted to the NRC by Reference 3.

The generic results documented in WCAP-11145 demonstrate that an Indian Point 3 specific small break LOCA reanalysis utilizing the NOTRUMP model would result in the calculation of a limiting peak clad temperature (PCT) which would be significantly lower than the 1263°F PCT currently calculated with the WFLASH model. It is, therefore, concluded that a plant specific analysis is not necessary to comply with NUREG-0737 item II.K.3.31. The Authority's reference of WCAP-11145 satisfies NUREG-0737 item II.K.3.31 on a generic basis.

Should you or your staff have any questions regarding this matter, please contact Mr. P. Kokolakis of my staff.

Very truly yours,


John C. Brons
Senior Vice President
Nuclear Generation

cc: Resident Inspector's Office
Indian Point Unit 3
U.S. Nuclear Regulatory Commission
Buchanan, NY 10511

Mr. J. D. Neighbors, Senior Project Manager
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