

John C. Brons Senior Vice President Nuclear Generation

July 9, 1985 IPN-85-36

Director of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D.C. 20555

ATTENTION: Mr. Steven A. Varga, Chief

Operating Reactors Branch No. 1

Division of Licensing

SUBJECT: Indian Point 3 Nuclear Power Plant

Docket No. 50-286

NUREG-0737, Item II.K.3.31, "Plant - Specific

Calculation to Show Compliance with 10 CFR Part 50.46"

Dear Sir:

Your letter dated May 24, 1985 served to notify the Authority that the Westinghouse small break LOCA model, NOTRUMP has been approved for use in satisfying NUREG-0737, Item II.K.3.30, "Revised Small - Break Loss-of-Coolant-Accident Methods to Show Compliance with 10 CFR Part 50, Appendix K". As a member of the Westinghouse Owners Group (WOG), the Authority has referenced NOTRUMP as the revised small break LOCA model to satisfy the requirements of NUREG-0737, Item II.K.3.30. Pursuant to the subject NUREG-0737 item, the WOG will be performing generic studies to demonstrate that the current small break LOCA analyses utilizing the previously approved WFLASH evaluation model are more limiting than analyses utilizing the NOTRUMP model. Documentation of the comparison of NOTRUMP and WFLASH results is scheduled to be submitted by the WOG in May, 1986.

Should you or your staff have any questions regarding this matter, please contact Mr. P. Kokolakis of my staff.

Very truly yours,

John C. Brons

Senior Vice President Nuclear Generation

cc: Resident Inspector's Office
Indian Point Unit 3

U.S. Nuclear Regulatory Commission

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