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**J. Phillip Bayne** Executive Vice President Nuclear Generation

July 20, 1983 IPN-83-67

Director of Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Attention: Mr. Steven A. Varga, Chief Operating Reactors Branch No. 1 Division of Licensing

Subject: Indian Point 3 Nuclear Power Plant Docket No. 50-286 Inspection Program for Welds and Pipe Supports: Request for Additional Information

Dear Sir:

Our letter of June 30, 1981 (IPN-81-47) responded to the request for additional information concerning the inservice inspection program for Indian Point 3, contained in your April 3, 1981 letter. Our June 30, 1981 letter contained a committment to provide additional information regarding integrally welded pump support materials which make UT impractical. The Attachment to this letter serves to provide this additional information.

Should you or your staff have any questions regarding this matter, please contact Mr. P. Kokolakis of my staff.

Very truly yours,

J. Bavne

Executive Vice President Nuclear Generation

cc: Resident Inspector's Office
Indian Point Unit 3
U. S. Nuclear Regulatory Commission
P. O. Box 66
Buchanan, New York 10511

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## ATTACHMENT 1

The austenitic stainless steel support feet, which were fabricated as individual castings, are integrally welded to the cast austenitic stainless steel reactor coolant pump casing making ultrasonic examination not feasible. U/T examination of the weld through the normally coarse, highly attenuative, cast structure is further complicated by the macro structure differences--dictated by cooling rates and directions--from piece to piece and, because of tapered configuration, from section to section within the individual pieces. The lack of appropriately placed and oriented reflective surfaces (in the pump support assembly) which could be used to comparatively estimate relative attenuation differences, serves only to further define attenuation/distance amplitude compensation as an uncontrolled variable.