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J. Phillip Bayne Executive Vice President Nuclear Generation

June 15, 1983. IPN-83-61

Director of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Attention: Mr. Steven A. Varga, Chief Operating Reactors Branch No. 1 Division of Licensing

Subject:

Indian Point 3 Nuclear Power Plant Docket No. 50-286 Cycle 3/4 Reload Safety Evaluation

Dear Sir:

The Authority's letter of November 2, 1981 (IPN-81-87) submitted proposed changes to Sections 2.1, 2.3 and 3.10 of the Indian Point 3 Technical Specifications. Those proposed changes resulted from an Appendix K Emergency Core Cooling System (ECCS) analysis performed at the 12% equivalent steam generator tube plugging level. That analysis utilized the NRC approved February 1978 Westinghouse evaluation model. Amendment No. 40 to the Indian Point 3 Technical Specifications approved plant operation with up to 12% of the steam generator tubes plugged. In support of the cycle 3/4 refueling, Westinghouse had performed a Reload Safety Evaluation based on the NRC approved March 1978 evaluation model considering 12% steam generator tube plugging. Forty copies of this analysis, entitled "Reload Safety Evaluation Indian Point Nuclear Power Plant Unit 3, Cycle 4" are enclosed for your information. The results of this evaluation are consistent with those submitted via the Authority's letter of November 2, 1981.

The Authority has requested permission to operate at a steam generator equivalent tube plugging level of up to 24%, as necessary. Proposed Technical Specification changes reflecting a steam generator equivalent tube plugging level of 24% were submitted via the Authority's letter of May 5, 1983 (IPN-83-37). The Authority requests a timely review of this submittal so that it may be utilized, if necessary, during the current cycle.

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Very truly yours,

Y J. P. Bayne Executive Vice President

Nuclear Generation

cc: Resident Inspector's Office
Indian Point Unit 3
U.S. Nuclear Regulatory Commission
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