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February 15, 1983
IPN-83-13

Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
Post - TMI Requirements

Dear Sir:

This letter and its Attachment serve to supplement the information contained in our letters dated April 20, 1982 (IPN-82-33), April 29, 1982 (IPN-82-39) and June 4, 1982 (IPN-82-44) regarding certain NUREG-0737 requirements. The information being provided supersedes that contained in IPN-82-33, IPN-82-39 and IPN-82-44 for the NUREG-0737 Items discussed in Attachment I to this letter. Please note that the schedules discussed herein may be revised at a later date should unforeseen circumstances arise which adversely affect them.

Should you or your staff have any questions regarding this matter, please contact Mr. P. Kokolakis of my staff.

Very truly yours,

G. U. Wilverding

for J. P. Bayne
Executive Vice President
Nuclear Generation

cc: attached

State of New York
County of New York

MITCHELL J. WEINSTEIN
Notary Public, State of New York
No. 30-4740329
Qualified in Nassau County
Commission Expires March 30, 1983

Subscribed and sworn to before me this 15th day of February 1983.

Notary Public

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PDR ADOCK 05000286
P PDR

cc: Resident Inspector's Office
Indian Point Unit 3
U. S. Nuclear Regulatory Commission
P. O. Box 66
Buchanan, New York 10511

ATTACHMENT I

SUPPLEMENTAL INFORMATION REGARDING
CERTAIN NUREG-0737 REQUIREMENTS
IPN-83-13

POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT 3 NUCLEAR POWER PLANT
DOCKET NO. 50-286

The information provided below supersedes the information contained in our responses to Generic Letter No. 82-05 (IPN-82-33) and Generic Letter No. 82-10 (IPN-82-44) for the NUREG-0737 Items discussed. The status of Items addressed in Supplement 1 to NUREG-0737 will be provided on a time frame commensurate with the requirements of Supplement 1.

1.A.1.3.1 Limit Overtime

The Indian Point Unit 3 Administrative Procedures presently contain restrictions which meet the requirements of the NRC policy statement attached to Generic Letter No. 82-12 as they apply to persons on shift coverage.

1.A.1.3.2 Minimum Shift Crew

Our letter IPN-82-44 stated that four additional reactor operators (ROs) had then recently been qualified and were awaiting their licenses, and a fifth individual was awaiting final exam processing at that time. These five individuals have since obtained their RO licenses and three of the individuals have been used to replace ROs who have been removed from shift work to be trained for Senior Reactor Operator's (SRO's) licenses. These three individuals are scheduled to take their SRO qualification examination in March 1983.

Based on the number of persons currently in the RO training program who are scheduled to take their qualification examination in November 1983, the Authority plans to provide the additional RO on each shift, contingent upon a sufficient number of persons successfully completing the November 1983 examination. In the interim, the Authority will maintain the additional RO being provided by Consolidated Edison on an as needed reciprocal basis during Emergency Plan Conditions.

II.B.2 Plant Shielding Review

The Authority's two most recent submittals regarding the subject Item, IPN-82-33 and IPN-82-39, discussed the shielding or encapsulation of specified ECCS line Nos. 60 and 56 as the intended method for assuring access to the hydrogen recombiner panel (HRP) area and for automating containment isolation valves under post-accident conditions. Further review of this matter indicates that encapsultation of line 60 is no longer required. Furthermore, the previous analysis (used as the basis for the proposed modifications discussed in the above-mentioned letters) did not include line 16 as a potential radiation source which could affect access to

the HRP area. However, the previous analysis was conservative in that it assumed that access to the HRP area was required eight hours after the accident. The Authority is continuing to re-evaluate the time frame for, and method of, providing access to the HRP area. The Authority will provide a summary of the evaluation to the NRC. In the event that the NRC staff does not concur with the AUthority's findings, the Authority will complete the modifications associated with line 56 within six months after startup from the current cycle 3/4 refueling outage . Modifications associated with line 16 would be completed prior to startup from the cycle 4/5 refueling outage, consistent with the original schedule for completion of Item II.B.2.

The Authority's IPN-82-33 submittal stated that automation of selected containment isolation valves is expected to be completed by the end of the cycle 4/5 refueling outage. Instead of automation, it has been determined that relocation of some of the existing manually operated valves will comply with the II.B.2 criteria. As a minimum, a total of fifteen valves will be automated or relocated during the current cycle 3/4 refueling outage. The remainder of the valves are scheduled for completion prior to startup from the cycle 4/5 refueling outage.

II.D.1.2 RV&SV Test Programs

The results of the generic EPRI RV&SV test programs have been transmitted directly to the NRC. Plant specific evaluation of the generic test program results have been trasmitted to the NRC by our letter dated December 15, 1982 (IPN-82-80). The plant-specific analysis of the RV & SV discharge piping is presently being performed by our contractor. The results are scheduled to be transmitted to the NRC by May 31, 1983.

II.F.1.1 Noble Gas Monitor

For reasons set forth in our letters IPN-81-97 and IPN-82-33, the Authority had scheduled the implementation of this Item for the cycle 4/5 refueling outage. Our present schedule calls for installation of the vent monitor during the current cycle 3/4 refueling outage and for installation of the main steam line monitor prior to startup from the cycle 4/5 refueling outage.

II.F.1.4 Containment Pressure Monitor

The Authority installed and placed in operation this monitor prior to January 1, 1982 in accordance with the commitment made in our IPN-80-117 letter dated December 30, 1980. The Authority has received the

Environmental Qualification Report from the manufacturer and our preliminary review of this report indicates that the transmitters should be qualified to IEEE-323-1974 for the specific environment conditions at Indian Point 3. The requirements of this NUREG-0737 Item would thus be fulfilled. In the event that our completed review of the manufacturer's report indicates otherwise, we will inform the NRC of our findings.

III.D.3.4 Control Room Habitability

The Authority has completed a review of the potential impact of toxic chemical vapors on the habitability of the control room. The results of our review, summarized in our letter IPN-81-59 dated August 10, 1981, indicated only three vapors (anhydrous ammonia, carbon dioxide, and chlorine) could reach the IP-3 control room air intake in sufficient concentration as to cause a potential hazard. The Authority plans to install standard grade chemical monitors prior to start-up from the cycle 4/5 refueling outage in order to comply with this item. The above schedule is based on the anticipated schedule for equipment delivery and availability of manpower.