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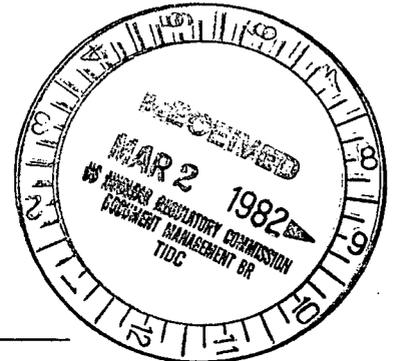
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February 26, 1982
IPN-82-22

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Mr. Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
Steam Generator Inspection Plan for
the Cycle 3/4 Refueling Outage



Dear Sir:

This letter provides the subject inspection plan in accordance with your November 20, 1980 letter.

Facility Operating License DPR-64, condition 2.J requires, in part, that at the end of Cycle 3 operations: "An inspection of all four steam generators shall be performed..." This inspection is intended to monitor the status of denting in the steam generators. During the cycle 3/4 refueling outage, the Authority proposes to perform gauge inspections of approximately 750 tubes on the hot leg sides of each steam generator. The length of each tube inspected will extend from the tube inlet through the sixth tube support plate on the hot leg side. Attachment I contains a map of each steam generator (hot leg side) which indicates the tubes which will be inspected.

IP-3 Technical Specification (TS) § 4.9.A.2.c. requires that: "At the third inservice inspection subsequent to the pre-service inspection, twelve percent of the tubes in the steam generator not inspected during the first two inservice inspections shall be inspected as a minimum." This TS applies to the cycle 3/4 refueling outage inspection and to steam generator no. 31. The Authority will inspect, in accordance with TS § 4.9 requirements, those tubes indicated in Attachment I (refer to Steam Generator no. 31 map notes).

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In addition, the Authority proposes to inspect 100% of the tubes on the cold leg side of steam generator no. 31 in order to determine the status of the pitting phenomenon discovered during the September 1981 steam generator inspection. Eddy current

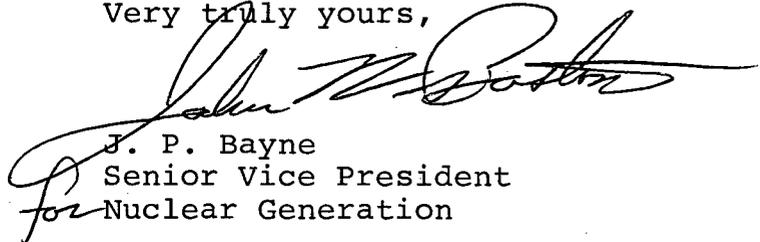
testing (ECT) techniques will be utilized and alternate probe designs are currently being evaluated to ensure the most accurate results possible. Emphasis will be placed on minimizing the effects of copper deposits on the tube surfaces. The length of the tubes inspected will extend from the outlet of each tube up to the second tube support plate on the cold leg side.

Following the above ECT inspections of steam generator no. 31, the Authority will determine what further inspections will be conducted in the remaining steam generators and will expeditiously contact the Commission regarding that determination.

Finally, the Authority will perform a prototypical sleeving of 10 tubes which will be selected based on the results of steam generator no. 31 pitting phenomenon inspection. Westinghouse Corporation will perform this sleeving utilizing Inconel 600 sleeves and rolled joints.

This matter was discussed with Mr. E. Murphy of the NRC staff on February 22, 1982. If you have any questions, please contact Mr. J. Lamberski of my staff.

Very truly yours,



J. P. Bayne
Senior Vice President
for Nuclear Generation

JPB/BB:mp

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