

CENTRAL FILES

POWER AUTHORITY OF THE STATE OF NEW YORK  
INDIAN POINT NO. 3 NUCLEAR POWER PLANT

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February 6, 1980  
IP-KEM-7700

58-286

Mr. Boyce H. Grier, Director  
Office of Inspection and Enforcement  
Region I  
U. S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

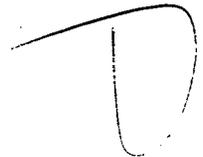
Dear Mr. Grier:

This letter addresses action on Item 2, contained in I.E. Bulletin 79-17, which we received on July 31, 1979.

During the period from September 15, 1979 to October 23, 1979, an examination of those systems or portions of systems which contain stagnant oxygenated borated water at the Indian Point 3 Nuclear Power Plant was performed by Westinghouse Nuclear Service Division. Those systems or portions thereof requiring inspection include the following:

1. The accumulator discharge piping to the class boundary.
2. High head safety injection system piping.
3. Containment spray system piping.
4. Residual heat removal system piping.
5. Safety injection pump suction piping from the residual heat removal system heat exchangers.
6. Containment recirculation spray header piping and low head safety injection piping from the discharge of the containment recirculation pumps.
7. Boron injection system excluding the boron injection tank and recirculation piping.
8. Spent Fuel Pool Cooling System.

Isometric sketches were developed for these systems or portions of systems and weld locations identified. From these identified welds, a 10 percent sample of welds in each system and pipe size were selected.



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Based on the NRC bulletin requiring that circumferential pipe welds be ultrasonically examined in accordance with Appendix III of Section XI, 2" socket welds in these systems were exempted from examination.

Ultrasonic and liquid penetrant examinations were performed on the selected welds in accordance with approved Westinghouse procedures. Surface examination was extended to include a minimum of 6T on each side of the weld where practical to include the area done by ultrasonic examination. No evidence of any cracking was found during this inspection.

During the period from November 6, 1979 to January 25, 1980, an ASME Section XI visual examination of normally accessible welds of engineered safety systems at service pressure was performed to verify system integrity. Those systems or portions thereof which were inspected included the following:

- (1) The residual heat removal system from the suction on reactor coolant hot leg loop 2 through the residual heat removal pumps and heat exchangers discharging into each of the four reactor coolant cold legs.
- (2) The containment spray system from the suction of the containment spray pumps including all discharge piping up to the containment isolation valves.
- (3) High head safety injection pump suction piping from the residual heat removal heat exchangers.
- (4) The low head safety injection system from the discharge of the safety injection accumulators to the reactor coolant cold legs.
- (5) High head safety injection system from pump discharge to containment penetrations.

Using a standard post maintenance test procedure to perform the test and inspection, no visible leakage was noted on normally accessible welds. Thus no corrective measures were required.

Very truly yours,



J. P. Bayne  
Resident Manager

KEM/jd

cc: Director, Division of Operating Reactors  
Office of Inspection & Enforcement  
Washington, D. C. 20555