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November 21, 1979

IPN-79-81

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Director, Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. Albert Schwencer, Chief
Operating Reactors Branch No. 1
Division of Operating Reactors

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
Containment Purge and Vent Valve Operability

Reference: Letter from Darrell G. Eisenhut (NRC), dated
September 27, 1979, to All Light Water Reactors

Dear Sir:

In response to your letter dated September 27, 1979 regarding containment purging and venting during normal operation - guidelines for valve operability, we have begun a Valve Qualification Program through Westinghouse on an expedited basis. The intent of the program is to demonstrate, through analysis, that the existing containment purge and exhaust valves' actuators will perform adequately during a design basis accident - loss of coolant accident (DBA - LOCA). The analysis will address all considerations outlined in the referenced letter.

A preliminary study based on information supplied by the valve manufacturer (closure rate and maximum ΔP related to angle of opening of valve), response time of containment pressure instrumentation (based on high containment pressure actual test data) and rate of containment pressure increase from a double ended rupture of a reactor coolant pipe (IP3 FSAR, Fig. 14.3.4-2) has indicated that the containment pressure does not exceed the maximum valve closure ΔP at any point in time from fully open to fully closed position thus demonstrating operability of the Purge Valves.

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If you have any questions regarding Authority efforts in this area please do not hesitate to contact us.

Very truly yours,



Paul J. Early
Assistant Chief Engineer-Projects