

POWER AUTHORITY OF THE STATE OF NEW YORK

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June 13, 1979
IPN-79-36

Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. Albert Schwencer, Chief
Operating Reactors Branch No. 1
Division of Operating Reactors

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
Cycle III Refueling Outage Information

Dear Sir:

Enclosed please find ten (10) copies of the subject item in accordance with Branch Technical Position DOR-1, "Guidance for Reload Submittals". To date, no Technical Specification change requests will be required either for the Cycle III reload fuel or for the work to be completed during the refueling outage. If any Technical Specification changes or unreviewed safety questions are found during the Authority's review of the Westinghouse Reload Safety Evaluation Report, a formal submittal to the NRC will be made ninety (90) days prior to the startup date.

If you have any questions on this information please contact us.

Very truly yours,


Paul J. Early
Assistant Chief Engineer-Projects

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APPENDIX A

REFUELING INFORMATION REQUEST

1. Name of facility - Indian Point Unit No. 3
2. Scheduled date for next refueling shutdown - September 15, 1979
3. Scheduled date for restart following refueling - December 15, 1979
4. Indicate whether any activities, other than those associated with refueling, will take place during the refueling outage for which prior NRC approval will be sought. - None
5. Indicate whether refueling or resumption of operation thereafter will involve an unreviewed safety question or require a technical specification change. - Refueling - No.
 - a. If there are unreviewed safety questions or technical specification changes, provide a summary of these. -
 - b. If there are no unreviewed safety questions and no technical specification changes:

Provide a brief summary of the basis for this determination, and indicate if there have been any changes in the information requested in item 7 below from previous refuelings of this reactor.*

Indicate whether the reload fuel design and safety analysis for the refueled core has been reviewed by your Safety Review Committee. If no such review has taken place, provide the schedule for this review.
6. Scheduled date(s) for submitting proposed licensing action and supporting information. -
7. Identification of important considerations associated with refueling of this reactor:
 - a. Fuel Supplier - Westinghouse
 - b. Changes in fuel bundle and rod design (mechanical, thermal hydraulic, composition, enrichment, etc.) - 3.3 w/o, 15x15 LOPAR Assembly

*5. b. Fuel thermo-mechanical limits fall within envelope of previous approved submittals.

- c. Design methods and procedures that have not been reviewed and approved by the NRC - None
- d. Performance analysis methods and procedures that have not been reviewed and approved by the NRC - None
- e. Safety analysis methods and procedures that have not been reviewed and approved by the NRC - None
- f. Group who performed safety analysis (company, organization, etc.)
Westinghouse Co.