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Ralph E. Beedle Executive Vice President Nuclear Generation

May 13, 1992 IPN-92-022

U.S. Nuclear Regulatory Commission Mail Station P1-137 Washington, DC 20555

ATTN: Document Control Desk

Subject: Indian Point 3 Nuclear Power Plant Docket No. 50-286 Response to Request for Additional Information Concerning the Proposed Change to the Operating License Expiration Date (TAC No. M76970)

Dear Sir:

By letter dated April 2, 1992, the NRC staff requested additional information regarding the Authority's application to extend the expiration date of the Indian Point 3 Facility Operating License. The requested information is provided in the attachment to this letter.

If you have any questions, please contact Mr. P. Kokolakis.

Very truly yours,

Ralph E. Beedle Executive Vice President Nuclear Generation

cc: See next page

2051800

Attachment

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cc: U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406

> Resident Inspector's Office Indian Point 3 U.S. Nuclear Regulatory Commission P.O. Box 337 Buchanan, NY 10511

Mr. Nicola F. Conicella, Project Manager Project Directorate I-1 Division of Reactor Projects-I/II U.S. Nuclear Regulatory Commission Mail Stop 14B2 Washington, DC 20555





Additional Information for Extending the Indian Point 3 Operating License Expiration Date

Introduction

By letter dated June 11, 1990, as supplemented by letters dated June 18, 1991, and February 11, 1992, the Power Authority applied for an amendment to the Indian Point 3 Facility Operating License to revise the expiration date to 40 years from the date of issuance of the Operating License. By letter dated April 2, 1992, the NRC staff requested additional information regarding the Authority's application and the transportation of fuel exceeding 4% uranium-235 enrichment and/or 33,000 megawatt-days per metric ton (MWD/T) burnup. Listed below are the NRC staff's requests, and the Authority's responses.

Information Requested

1. Adopt the staff's assessment of the environmental effects of transportation, as published in the Federal Register on August 11, 1988 (53 FR 30355), with a statement that it is in fact applicable to the Indian Point Nuclear Generating Unit No. 3 or provide a plant specific statement under 10 CFR 51.52(b).

Authority Response

The Authority reviewed the staff's assessment of the environmental effects of transportation (53 FR 30355). The Indian Point 3 Technical Specifications restrict the enrichment of reload fuel to no more than 4.5 weight percent of uranium-235. Indian Point 3 has some fuel assemblies in the Spent Fuel Pool with a burnup of greater than 33,000 MWD/T (but less than 60,000 MWD/T). The NRC generic assessment (53 FR 30355) indicates that the environmental impact of extended irradiation up to 60,000 MWD/T and increased enrichment up to 5 weight percent are bounded by the impacts reported in Table S-4 of 10 CFR 51.52. This generic assessment is applicable to Indian Point 3, therefore a detailed analysis as described in 10 CFR 51.52(b) does not have to be performed.

2. Assess the impact of using ZIRLOTM clad fuel as related to 10 CFR 51.52.

Authority Response

By letter from R. E. Beedle to the NRC Document Control Desk (dated January 8, 1992), the Power Authority proposed changing the Indian Point 3 Technical Specifications to address the use of ZIRLOTM, as well as Zircaloy-4, fuel rod cladding. On pages two and three of the Safety Evaluation accompanying the January 8, 1992 letter, the Authority provided an analysis of how the use of ZIRLOTM cladding affects the conditions listed in Table S-4 of 10 CFR 51.52. Please refer to that Safety Evaluation for the assessment of ZIRLOTM cladding related to 10 CFR 51.52.