

ATTACHMENT I TO IPN-90-049

PROPOSED TECHNICAL SPECIFICATION CHANGES
RELATED TO
THE REMOVAL OF CONTAINMENT ISOLATION VALVES UH-37 AND UH-38

NEW YORK POWER AUTHORITY
INDIAN POINT 3 NUCLEAR POWER PLANT
DOCKET NO. 50-286
DPR-64

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TABLE 3.6-1

**NON-AUTOMATIC CONTAINMENT ISOLATION VALVES
OPEN CONTINUOUSLY OR INTERMITTENTLY FOR PLANT OPERATION**

VALVE NO.	VALVE NO.	VALVE NO.
AC-MOV-744	SI-MOV-850C	SWN-51-4
AC-MOV-1870	SI-1833A	SWN-44-5
AC-MOV-743	SI-1833B	SWN-51-5
SP-990C	SI-859A	SWN-71-1
AC-732	SI-859C	SWN-71-2
SI-MOV-885A	AC-752F	SWN-71-3
SI-MOV-885B	AC-753F	SWN-71-4
SI-MOV-888A	AC-752J	SWN-71-5
SI-MOV-888B	AC-753J	SA-24-1
CH-MOV-205	SWN-41-1	SA-24-2
CH-MOV-226	SWN-43-1	PS-PCV-1111-1
CH-227	SWN-41-2	PS-PCV-1111-2
CH-MOV-250A	SWN-43-2	
CH-MOV-441	SWN-41-3	
CH-MOV-250B	SWN-43-3	SP-MOV-990A
CH-MOV-442	SWN-41-4	SP-MOV-990B
CH-MOV-250C	SWN-43-4	SI-1814A
CH-MOV-443	SWN-41-5	SI-1814B
CH-MOV-250D	SWN-43-5	SI-1814C
CH-MOV-444	SWN-44-1	HR-MOV-1882A
SI-869A	SWN-51-1	HR-MOV-1875A
SI-869B	SWN-44-2	HR-MOV-1875B
SI-878A	SWN-51-2	HR-MOV-1876A
SI-878B	SWN-44-3	HR-MOV-1876B
SI-MOV-851A	SWN-51-3	PS-7
SI-MOV-850A	SWN-44-4	PS-8
		PS-9
		PS-10

TABLE 4.4-1 (Page 5 of 7)

CONTAINMENT ISOLATION VALVES			
<u>Valve No.</u>	<u>Penetration Number</u> ⁽¹⁾	<u>Test Fluid</u> ⁽²⁾	<u>Minimum Test Pressure (PSIG)</u> ⁽⁸⁾
SWN-44-5	40	Water ⁽⁶⁾	47
SWN-51-5	40	Water ⁽⁶⁾	47
SWN-71-1	40	Water ⁽⁶⁾	47
SWN-71-2	40	Water ⁽⁶⁾	47
SWN-71-3	40	Water ⁽⁶⁾	47
SWN-71-4	40	Water ⁽⁶⁾	47
SWN-71-5	40	Water ⁽⁶⁾	47
SA-24-1	41	Water ⁽⁴⁾	47
SA-24-2	41	Water ⁽⁴⁾	47
VS-FCV-1170	48	Gas ⁽⁷⁾	43
VS-FCV-1171	48	Gas ⁽⁷⁾	43
VS-FCV-1172	49	Gas ⁽⁷⁾	43
VS-FCV-1173	49	Gas ⁽⁷⁾	43
VS-FCV-1190	50	Gas ⁽⁷⁾	43
VS-FCV-1191	50	Gas ⁽⁷⁾	43
VS-FCV-1192	50	Gas ⁽⁷⁾	43
SP-MOV-990A	51	Nitrogen ⁽⁴⁾	43
SP-MOV-990B	51	Nitrogen ⁽⁴⁾	43
SP-AOV-956A	52	Water ⁽⁴⁾	47
SP-AOV-956B	52	Water ⁽⁴⁾	47
SP-AOV-956C	53	Water ⁽⁴⁾	47
SP-AOV-956D	53	Water ⁽⁴⁾	47
SI-1814A	54	Gas	43
SI-1814B	55	Gas	43
SI-1814C	56	Gas	43
SP-SOV-506	57	Gas	43
SP-SOV-507	57	Gas	43

ATTACHMENT II TO IPN-90-049

SAFETY EVALUATION OF
PROPOSED TECHNICAL SPECIFICATION CHANGES
ASSOCIATED WITH THE REMOVAL OF
CONTAINMENT ISOLATION VALVES UH-37 AND UH-38

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INDIAN POINT 3 NUCLEAR POWER PLANT
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SAFETY EVALUATION OF PROPOSED
TECHNICAL SPECIFICATION CHANGES
ASSOCIATED WITH
THE REMOVAL OF CONTAINMENT ISOLATION VALVES UH-37 AND UH-38

Section I - Description of Changes

The proposed changes to Table 3.6-1 and Table 4.4-1 (Page 5 of 7) of the Technical Specifications reflect the removal of Containment Isolation Valves UH-37 and UH-38. Removal of these valves will eliminate the need for maintaining and testing valves UH-37 and UH-38.

Section II - Evaluation of Changes

Containment Isolation Valves UH-37 and UH-38 are on the Auxiliary Steam Supply and Condensate Return (ASC) System steam supply and condensate return lines, respectively. These lines originally were designed to supply steam to the Containment Unit Heaters. Since the temperature in containment is maintained without the use of the Unit Heaters, auxiliary steam is no longer required to be supplied to containment. Since valves UH-37 and UH-38 are containment isolation valves, they are required per the Technical Specifications to be maintained and tested.

A plant modification proposes to remove UH-37 and UH-38, and to cut and cap the adjacent piping near both sides of their respective containment penetrations. This modification would enhance the integrity of the containment penetrations by assuring that the capped piping sections through the penetrations are leak tight.

The Technical Specification Changes transmitted by this submittal reflect the changes associated with the plant modification described above. The changes will eliminate the need for maintaining and testing valves UH-37 and UH-38.

Section III - No Significant Hazards Evaluation

Consistent with the requirements of 10 CFR 50.92, the enclosed application is judged to involve no significant hazards based on the following information:

- (1) Does the proposed license amendment involve a significant increase in the probability or consequences of an accident previously evaluated?

Response:

The proposed changes reflect a plant modification that will enhance the integrity of the containment penetrations associated with the ASC steam supply and condensate return lines. The modification will cap the two lengths of piping penetrating containment. The capped piping will be leak tight,

therefore the technical specification changes associated with this modification do not involve a significant increase in the probability or consequences of an accident previously evaluated.

- (2) Does the proposed license amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

Response:

The proposed changes reflect a plant modification that will provide containment isolation at the containment penetrations associated with the ASC steam supply and condensate return lines. Since this is the same function currently provided by containment isolation valves UH-37 and UH-38, the removal of these valves and associated technical specification changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

- (3) Does the proposed amendment involve a significant reduction in a margin of safety?

Response:

The proposed amendment does not involve a significant reduction in a margin of safety since it reflects a plant modification that will enhance containment integrity by providing leak tight seals at the penetrations associated with the ASC steam supply and condensate return lines.

Section IV - Impact of Change

This change will not adversely impact the following:

- ALARA Program
- Security and Fire Protection Programs
- Emergency Plan
- FSAR or SER Conclusions
- Overall Plant Operations and the Environment

Section V - Conclusions

The incorporation of this change: a) will not increase the probability nor the consequences of an accident or malfunction of equipment important to safety as previously evaluated in the Safety Analysis Report; b) will not increase the possibility for an accident or malfunction of a different type than any evaluated previously in the Safety Analysis Report; c) will not reduce the margin of safety as defined in the bases for any Technical Specification; d) does not constitute an unreviewed safety question; and e) involves no significant hazards considerations as defined in 10 CFR 50.92.

Section VI - References

- a) IP-3 FSAR
- b) IP-3 SER