

BEFORE THE UNITED STATES
NUCLEAR REGULATORY COMMISSION

In the Matter of)
POWER AUTHORITY OF THE STATE OF NEW YORK) Docket No. 50-286
Indian Point 3 Nuclear Power Plant)

APPLICATION FOR AMENDMENT TO
OPERATING LICENSE

Pursuant to Section 50.90 of the regulations of the Nuclear Regulatory Commission (NRC), the Power Authority of the State of New York, as holder of Facility Operating License No. DPR-64, hereby applies for an Amendment to the Technical Specifications contained in Appendix A of this license.

The proposed changes to the Indian Point 3 Technical Specifications serve to amend certain Sections of Appendix A to the Operating License in accordance with the requirements of Generic Letter No. 83-37.

The proposed changes to the Technical Specifications are presented in Attachment No. 1 to this Application. The Safety Evaluation is included in Attachment No. 2.

POWER AUTHORITY OF THE STATE
OF NEW YORK

BY C. M. Wilverding
for J. P. Bayne
Executive Vice President
Nuclear Generation

Subscribed and Sworn to before
me this 17 day of Feb. 1984

Jeanne La Luna
Notary Public

JEANNE LA LUNA
NOTARY PUBLIC, STATE OF NEW YORK
NO. 60-4614305
QUALIFIED IN WESTCHESTER COUNTY
TERM EXPIRES MARCH 30th 1985

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Attachment A

Response to Generic Letter No. 83-37
NUREG-0737 Technical Specifications
IPN-84-06

Power Authority of the State of New York
Indian Point 3 Nuclear Power Plant
Docket No. 50-286

The information below is provided in response to Generic Letter No. 83-37 regarding implementation of NUREG-0737 Technical Specifications. Each of the Items contained in the Generic Letter is addressed below. The revised Technical Specification pages are contained in Attachment No. 1 of this submittal

(1) Reactor Coolant System Vents (II.B.1)

As stated in the Authority's letter dated December 7, 1983, IPN-83-98, the reactor vessel vent system will be made operational prior to startup from the next (cycle 4/5) refueling outage. The Authority will prepare and submit proposed Technical Specifications for the vent system no later than two months prior to start of the cycle 4/5 outage.

(2) Post-Accident Sampling (II.B.3)

Technical Specification Sections 6.4 (Training) and 6.8 (Procedures) have been modified to reflect the administrative controls specified in the Generic Letter.

(3) Long-Term Auxiliary Feedwater System Evaluation (II.E.1.1)

As stated in the Authority's letter dated June 1, 1983, IPN-83-49, the Limiting Conditions for Operation (LCO) and the Surveillance Requirements for the Auxiliary Feedwater (AFWS) System are defined in Technical Specification Sections 4.3.1 and 4.8, respectively. Both the LCO and the Surveillance Requirements for the AFWS System are similar to those for other safety-related systems and the requirements of the Generic Letter are thereby fulfilled. NRC acceptance of the current Indian Point 3 AFWS Technical Specifications has been previously documented in Mr. S. A. Varga's letter dated August 10, 1982 and no additional Technical Specification changes are required.

(4) Noble Gas Effluent Monitors (II.F.1.1)

Technical Specification Table 3.5-4, Table 4.1-1 and Section 6.9.2 (Special Reports) have been modified to reflect the recommendations of the Generic Letter for the post-accident effluent monitor on the unit vent. The main steam safety valves/atmospheric dump valves monitors' will be installed during the cycle 4/5 outage. The Authority will prepare and submit proposed Technical Specifications for the main steam effluent monitors no later than two months prior to start of the cycle 4/5 outage.

(5) Sampling and Analysis of Plant Effluents (II.F.1.2)

Technical Specification Sections 6.4 (Training) and 6.8 (Procedures) have been modified to reflect the Administrative Controls specified in the Generic Letter. Operability requirements for this item were proposed in the Authority's letter dated February 1, 1982, IPN-82-10, and were subsequently incorporated in the Indian Point 3 Technical Specifications via Amendment No. 44 to the Facility Operating License.

(6) Containment High-Range Radiation Monitor (II.F.1.3)

Technical Specification Table 3.5-5, Table 4.1-1 and Section 6.9.2 have been modified to reflect recommendations of the Generic Letter for the High-Range Containment Radiation Monitors.

(7) Containment Pressure Monitors (II.F.1.4)

Technical Specification Section 3.5 and Table 3.5-5 have been modified to require that at least two containment pressure channels be operable when Tavg is greater than 350°F.

(8) Containment Water Level Monitor (II.F.1.5)

The containment water level monitoring system will be installed during the cycle 4/5 refueling outage. The Authority will prepare and submit proposed Technical Specifications for the containment water level monitoring system no later than two months prior to start of the cycle 4/5 outage.

(9) Containment Hydrogen Monitor (II.F.1.6)

The containment hydrogen monitoring system will be installed during the cycle 4/5 refueling outage. The Authority will prepare and submit proposed Technical Specifications for the containment hydrogen monitoring system no later than two months prior to start of the cycle 4/5 outage.

(10) Instrumentation for Detection of Inadequate Core Cooling (II.F.2)

The Authority has previously committed to installation of a Westinghouse designed differential pressure, reactor vessel level indication system (RVLIS) during the cycle 5/6 refueling outage. The Authority will prepare and submit proposed Technical Specifications for the RVLIS no later than two months prior to the start of the cycle 5/6 outage.

Technical Specification Tables 3.5-5 and 4.1-1 have been modified to reflect the recommendations of the Generic Letter for the core exit thermocouples.

(11) Control Room Habitability Requirements (III.D.3.4)

The Authority has previously committed to installation of toxic gas monitors during the cycle 4/5 outage. The Authority will prepare and submit proposed Technical Specifications for the toxic gas monitors no later than two months prior to the start of the cycle 4/5 outage.