

ATTACHMENT I TO IPN-83-78
PROPOSED
TECHNICAL SPECIFICATIONS
CHANGES RELATED TO
INSERVICE INSPECTION

POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT 3 NUCLEAR POWER PLANT
DOCKET NO. 50-286

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4.2 INSERVICE INSPECTION

4.2.1 WELD AND SUPPORT PROGRAM

4.2.1.1 APPLICABILITY

Applies to in-service surveillance of ASME Code Class 1, 2, and 3 systems' welds and supports.

4.2.1.2 OBJECTIVE

To assure the continued integrity and functionability of welds and supports in ASME Code Class 1, 2, and 3 systems.

4.2.1.3 SPECIFICATION

- a. Inservice inspection of ASME Code Class 1, 2, and 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55 (g) (6) (i).
- b. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.
- c. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.
- d. Detailed records of each inspection shall be maintained to allow comparison and evaluation of future inspections.

The surveillance specified in Table 4.2-1 shall be performed on the steam generator girth welds number 6 and such surveillance shall be in addition to that required by the ASME Code, Section XI.

4.2.1.5

BASES

This specification ensures that inservice inspection of ASME Code Class 1, 2 and 3 components will be performed in accordance with a periodically updated version of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda as required by 10 CFR 50.55a. Relief from any of the above requirements has been provided in writing by the Commission and is not a part of these Technical Specifications.

Under the terms of this Specification, the more restrictive requirements of the Technical Specifications take precedence over the ASME Boiler and Pressure Vessel Code and applicable Addenda.

The augmented inspection of steam generator girth welds number 6 required by table 4.2.-1 may be deleted with specific approval of the NRC if experience over an interval of approximately three refueling outages or changes of plant components indicate that this augmented inspection is no longer necessary. For this augmented inspection the 45° shear wave method was chosen based on the review of the original ultrasonic data. This search was the most sensitive of the three used (0°, 45°, and 60°). It has also been determined that it will be adequate to perform the inspection UT in the vertical plane only. This method of search will detect cracks parallel to the weld which were typical of those originally found. There were a limited number of cracks

reported on the original ultrasonic inspection as transverse, however, in reviewing subsequent radiographs, magnetic particle and liquid penetrant examination results, it is evident that these cracks emanated from defects parallel to the weld.

TABLE 4.2-1

AUGMENTED STEAM GENERATOR GIRTH WELD INSPECTION

To provide surveillance of the steam generator girth welds number 6, after the repairs made during the 1982/1983 outage, the Authority will perform ultrasonic inspection using the 45° shear wave method of one hundred and seventy five (175) linear inches of weld. Thirty five (35) inches will be examined on Steam Generators 31, 32 and 33. Seventy (70) inches will be examined on Steam Generator 34.

The following areas have been selected for this augmented examination:

Steam Generator	Location on Circumference	Segment Location
31	204" clockwise to 239" from 0 Reference	17-20
32	316" clockwise to 334" from 0 Reference	26-28
	348" clockwise to 365" from 0 Reference	29-31
33	360" clockwise to 395" from 0 Reference	30-33
34	0 Reference clockwise to 18" 505" clockwise to 522" from 0 Reference	0-2 42-0
	168" clockwise to 203" from 0 Reference	14-17

ATTACHMENT II TO IPN-83-78
SAFETY EVALUATION OF
PROPOSED TECHNICAL SPECIFICATIONS
CHANGES RELATED TO
INSERVICE INSPECTION

POWER AUTHORITY OF THE STATE OF NEW YORK
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Section I - Description of Changes

This application seeks to amend Section 4.2 of Appendix A to the Operating License in response to the Commission's letters dated June 13, 1983 and July 15, 1983, concerning the Inservice Inspection (ISI) program for Indian Point 3. The proposed amendment would simplify the Technical Specifications by referring to the NRC-approved ISI program instead of listing the detailed requirements of the program as is presently the case. The proposed Amendment would make the Indian Point 3 Technical Specifications related to ISI consistent with the Standard Technical Specifications for Westinghouse plants and would avoid the need to revise the Technical Specifications to reflect periodic changes in the ISI program.

Section II - Evaluation of the Change

The inservice inspection requirements for ASME Code Class 1, 2, and 3 components must be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50.55a (g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50.55 a(g)(6)(i). The current Section 4.2 of the Indian Point 3 Technical Specifications defines the ISI requirements for the plant and includes a detailed list of inspection program. In its current format, Section 4.2 has to be revised to reflect periodic changes in the ISI program.

The proposed amendment seeks to delete the details of the inspection program from Section 4.2 of the Indian Point 3 Technical Specifications and to insert a reference to 10 CFR 50.55 a(g) similar to that contained in Section 4.05 of the Standard Technical Specifications for Westinghouse Pressurized Water Reactors (NUREG-0452, Revision 3). The proposed amendment would serve to simplify the Technical Specifications and eliminate the potential for future amendment requests related to changes in the ISI program without any reduction in the normal inspection requirements specified in 10CFR50.55a(g).

Section 4.2 of the Technical Specifications currently require augmented inspections to be performed in three areas:

- (1) Weld L-1 in the pressurizer,
- (2) Steam Generator Head welded cladding surfaces- primary sides, and
- (3) Steam Generator Girth Welds number 6.

The essence of the augmented inspection for pressurizer weld L-1 was utilization of the Summer 1975 Addenda to the 1974 Edition of Section XI of the ASME Code at a time when the ISI program was based on the January, 1970 Code Edition.

The Authority has submitted, and the NRC approved, revision of the ISI program based on the 1974 ASME Section XI Code Edition with Addenda through the Summer 1975. The Safety Evaluation Report for the updated program was attached to Mr. S.A. Varga's letter of July 15, 1983 to Mr. J.P. Bayne. In light of the above, there is no need to specify pressurizer weld L-1 as an area requiring augmented inspection and the proposed amendment is so written.

Item no. 3.7 of existing Technical Specification Table 4.2-1 specifies that 100% of the steam generator head welded cladding surfaces (primary sides) be inspected by the end of the third refueling outage. These inspections have been completed and the results of the third and final inspection has been transmitted to the NRC via the report "Third In-Service Surveillance Examination of the Indian Point 3 Steam Generator Channel Head Cladding" (WCAP-10222). In light of the above, there is no need to specify the steam generator head welded surfaces (primary side) as an area requiring augmented inspection and the proposed amendment is so written.

The augmented inspection of the steam generator girth welds number 6 has been retained in revised Table 4.2-1 of the proposed Technical Specifications.

The Authority considers that the proposed changes can be classified as not likely to involve significant hazards considerations since the proposed changes constitute an administrative change to achieve consistency with the Standard Technical Specifications for Westinghouse Plants. The proposed changes do not themselves alter the inservice inspection requirements for Indian Point 3. (Example (i), Federal Register, Vol. 48, No. 67 dated April 6, 1983, page 14870).

Section III - Impact of the Change

The proposed amendment will not result in a reduction in the Inservice Inspection requirements for Indian Point 3 nor will it impact the following:

- ALARA Program
- Fire Protection Program
- Emergency Plan
- FSAR or SER Conclusions
- Overall Plant Operations

Section IV - Conclusion

The incorporation of these changes: a) will not increase the probability nor the consequences of an accident or malfunction of equipment important to safety as previously evaluated in the Safety Analysis Report; b) will not increase the possibility for an accident or malfunction of a different type than any evaluated previously in the Safety Analysis Report; c) will not reduce the margin of

safety as defined in the basis for any Technical Specification; d) do not constitute an unreviewed safety question; and e) involves no significant hazards considerations as defined in 10 CFR 50.92.

Section V - References

- (a) Indian Point 3 FSAR
- (b) Letter dated July 15, 1983, S.A. Varga to J.P. Bayne.
- (c) Letter dated June 13, 1983, S. A. Varga to J. P. Bayne.