

ATTACHMENT B

PROPOSED  
TECHNICAL SPECIFICATION  
CHANGES

POWER AUTHORITY OF THE STATE OF NEW YORK  
INDIAN POINT 3 NUCLEAR POWER PLANT  
DOCKET NO. 50-286  
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the expiration of one-third of the inspection interval (with credit for no more than 33-1/3 percent if additional examinations are completed) and at least 50 percent shall have been completed by the expiration of two-thirds of the inspection interval (with credit for no more than 66-2/3 percent). The remaining required examinations shall be completed by the end of the inspection interval. Successive inspections shall meet the requirements of Paragraph IS-243 of the ASME Rules for In-Service Inspection of Nuclear Reactor Coolant Systems.

#### 4.2.8 BASES

The inspection program, where practical, is in compliance with Section XI of the ASME Code for In-Service Inspection of Nuclear Reactor Coolant Systems dated January 1970. Though examinations in certain areas are desirable, it should be recognized that equipment and techniques to perform the inspection are still in development. In all areas scheduled for volumetric examination, a detailed pre-service mapping will be conducted using techniques expected to be used for post-operation examinations. The areas indicated for inspection represent those of representative stress levels, and therefore will serve to indicate potential problems before significant flaws develop there or at other areas. As more experience is gained in operation of pressurized-water reactors, the time schedule and location of inspection may be altered or, should new techniques be developed, consideration may be given to incorporate these new techniques into this inspection program.

The techniques for inspection include visual inspections, ultrasonic, radiographic, magnetic particle and liquid penetrant testing of selected parts during refueling periods or other appropriate plant outages.

The augmented inspection of steam generator weld number 6 required by table 4.2.-1., Item No. 3.8 (NOTE), may be deleted with specific approval of the NRC if experience over an interval of approximately three refueling outages or changes of plant components indicate that this augmented inspection is no longer necessary. For this augmented inspection the 45° shear wave method was chosen based on the review of the original ultrasonic data. This search was the most sensitive of the three used (0°, 45° and 60°). It has also been determined that it will be adequate to perform the inspection by UT in the vertical plane only. This method of search will detect cracks parallel to the weld which were typical of those originally found. There were a limited number of cracks reported on the original ultrasonic inspection as transverse, however, in reviewing subsequent radiographs, magnetic particle and liquid penetrant examination results, it is evident that these cracks emanated from defects parallel to the weld.

The inspection requirements of this section shall apply to all pressure-containing components that are part of the system boundary defined herein. Due to the design of Indian Point Unit #3, there may be areas where weld access is impossible due to high radiation and/or physical access problems. Exception is taken to performing inspections in these areas.

TABLE 4.2-1 (Sheet 8 of 12)

<u>Item No.</u>	<u>Examination Category</u>	<u>Components and Parts to be Examined</u>	<u>Method</u>	<u>Extent of Examination (Percent in 10 Year Interval)</u>	<u>Remarks</u>
3.8	-	Secondary side shell welds	UT	See Remarks	The total examination completed over the service life time will be equivalent of having performed 100% of the required examination.

3.8 (NOTE): **AUGMENTED STEAM GENERATOR GIRTH WELD INSPECTION**

To provide surveillance of the steam generator welds number 6, after the repairs made during the 1982/1983 outage, the Authority will perform ultrasonic inspection using the 45° shear wave method of one hundred and seventy five (175) linear inches of weld. Thirty five (35) inches will be examined on Steam Generators 31, 32 and 33. Seventy (70) inches will be examined on Steam Generator 34.

The following areas have been selected for this augmented examination:

<u>Steam Generator</u>	<u>Location on Circumference</u>	<u>Segment Location</u>
31	204" clockwise to 239" from 0 Reference	17-20
32	316" clockwise to 334" from 0 Reference	26-28
	348" clockwise to 365" from 0 Reference	29-31
33	360" clockwise to 395" from 0 Reference	30-33
34	0 Reference clockwise to 18"	0-2
	505" clockwise to 522" from 0 Reference	42-0
	168" clockwise to 203" from 0 Reference	14-17

TABLE 4.2-1 (Sheet 8a of 12)

<u>Item No.</u>	<u>Examination Category</u>	<u>Components and Parts to be Examined</u>	<u>Method</u>	<u>Extent of Examination (Percent in 10 Year Interval)</u>	<u>Remarks</u>
		PIPING PRESSURE BOUNDARY			
4.1	F	Vessel, pump and valve safe-ends to primary pipe welds and safe-ends in branch piping welds.	UT, PT & V	100%	This examination covers only the pressurizer safe-ends.
4.2	J-1	Circumferential and longitudinal pipe welds and branch pipe connections welds larger than 4 inches in diameter	V & UT	25%	Exception is taken to inaccessible welds and welds where examination techniques limit inspections.
4.3	G-1	Pressure-retaining bolting		Not applicable	
4.4	G-2	Pressure-retaining bolting	V	100%	

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records of any drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components designed for a limited number of transient cycles.
- g. Records of training and qualifications for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the PORC and the SRC.
- l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.13.
- m. Records of secondary water sampling and water quality.

#### 6.11 RADIATION AND RESPIRATORY PROTECTION PROGRAM

6.11.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure as to maintain exposures as far below the limits specified in 10 CFR Part 20 as reasonable achievable. Pursuant to 10 CFR 20.103 allowance shall be made for the use of respiratory protective equipment in conjunction with activities authorized by the operating license for this plant in determining whether individuals in restricted areas are exposed to concentrations in excess of the limits specified in Appendix B, Table I, Column 1 of 10 CFR 20.