

ATTACHMENT I

PROPOSED TECHNICAL SPECIFICATION CHANGES

RELATED TO

NUREG 0737, item II.F.1.2

POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT 3 NUCLEAR POWER PLANT
DOCKET NO. 50-286
February 1, 1982

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TABLE 3.5-3 (SHEET 3 of 3)

<u>FUNCTIONAL UNIT</u>	1	2	3	4	5
3. AUXILIARY FEEDWATER					
a. Stm Gen. Water Level-Low-Low					
i. Start Motor Driven Pumps	3/stm. gen	2 in any stm gen.	2 chan. in each stm gen	1	Reduce system temperature such that $T \leq 350^{\circ}\text{F}$
ii. Start Turbine-Driven Pump	3/stm. gen	2/3 in each of two stm. gen.	2 chan. in each stm. gen.	1	$T \leq 350^{\circ}\text{F}$
b. S. I. Start Motor-Driven Pumps					
	(All safety injection initiating functions and requirements)				
c. Station Blackout Start Turbine-Driven Pump	2	1	1	0	$T \leq 350^{\circ}\text{F}$
d. Trip of Main Feedwater Pumps start Motor-Driven Pumps	2	1	1	0	Hot Shutdown
4. LOSS OF POWER					
a. 480 V Bus	2/Bus	1/bus	1/bus	0	Hot Shutdown

TABLE 3.5-4 (Sheet 1 of 2)

INSTRUMENT OPERATING CONDITIONS FOR ISOLATION FUNCTIONS

NO.	FUNCTIONAL UNIT	1	2	3	4	5
		NO. OF CHANNELS	NO. OF CHANNELS TO TRIP	MIN. OPERABLE CHANNELS	MIN. DEGREE OF REDUNDANCY	OPERATOR ACTION IF CONDITIONS IN COLUMN 3 OR 4 CANNOT BE MET
1.	CONTAINMENT ISOLATION					
	a. Automatic Safety Injection (Phase A)	See Item No. 1(b) of Table 3.5-3				Cold Shutdown (see note 1)
	b. Containment Pressure (Phase B)	See Item No. 2(b) of Table 3.5-3				Cold Shutdown (see note 1)
	c. Manual					
	Phase A	2	1	1	0	Cold Shutdown (see note 1)
	Phase B	See Item No. 2 (a) of Table 3.5-3				Cold Shutdown (see note 1)
2.	STEAM LINE ISOLATION					
	a. High Steam Flow in 2/4 Steam Lines Coincident with Low T _{avg} or Low Steam Line Pressure	See Item No. 1(e) of Table 3.5-3				Cold Shutdown and Main Steam Isolation Valves Closed (see note 1)
	b. High Containment Pressure (Hi Hi Level)	See Item No. 2(b) of Table 3.5-3				Cold Shutdown and Main Steam Isolation Valves Closed (see notes 1 and 2)
	c. Manual	1/loop	1/loop	1/loop	0	Cold Shutdown and Main Steam Isolation Valves Closed (see note 1)

TABLE 3.5-4 (Sheet 2 of 2)

	1	2	3	4	5
3. FEEDWATER LINE ISOLATION					
a. Safety Injection	See Item No. 1 of Table 3.5-3				
4. CONTAINMENT VENT AND PURGE					
a. Containment Radioactivity- High (R11 and R12 monitor)	2	1	1	0	close all containment vent and purge valves
5. PLANT EFFLUENT RADIOIODINE/ PARTICULATE SAMPLING (sample line common with monitor R13)	1	NA	1	0	(see note 3)

NOTES

1. If the conditions of Columns 3 or 4 cannot be met, the reactor shall be placed in the hot shutdown condition, utilizing normal operating procedures, within 4 hours of the occurrence. If the conditions are not met within 24 hours of the occurrence, the reactor shall be placed in the cold shutdown condition, or the alternate condition if applicable, within an additional 24 hours.
2. Main steam isolation valves may be closed in lieu of going to cold shutdown if the circuitry associated with closing the valves is the only portion inoperable.
3. If the plant vent sampling capability is determined to be inoperable when the reactor is above the cold shutdown condition, then restore the sampling capability within 72 hours or:
 - a) Initiate an alternate method of equivalent sampling capability and submit a 30 day report to NRC pursuant to Technical Specification 6.9.1.8, or if the alternate method cannot be established then,
 - b) Submit a Special Report to NRC pursuant to Technical Specification 6.9.2. within the next 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.

TABLE 4.1-1 (SHEET 4 of 4)

<u>CHANNEL DESCRIPTION</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>TEST</u>	<u>REMARKS</u>
29. Reactor Coolant System Subcooling Margin Monitor	D	R	N.A.	
30. PORV Position Indicator (Limit Switch)	N.A.	R	R	
31. PORV Position Indicator (Acoustic Monitor)	D	R	R	
32. Safety Valve Position Indicator (Acoustic Monitor)	D	R	R	
33. Auxiliary Feedwater Flow Rate	N.A.	R	N.A.	
34. Plant Effluent Radioiodine/Particulate Sampling (sample line common with monitor R 13)	N.A.	N.A.	R	

S - Each Shift
 D - Daily
 W - Weekly
 M - Monthly
 P - Prior to each startup if not done previous week
 Q - Quarterly
 R - Each Refueling Outage
 NA - Not Applicable

- d. Abnormal degradation of systems other than those specified in 6.9.1.7.c above designed to contain radioactive material resulting from the fission process. 7/

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Sealed source leakage on excess of limits (Specification 3.9)
- b. Inoperable Seismic Monitoring Instrumentation (Specification 4.10)
- c. Primary coolant activity in excess of limits (Specification 3.1.D)
- d. Seismic event analysis (Specification 4.10)
- e. Inoperable fire protection and detection equipment (Specification 3.14)
- f. The complete results of the steam generator tube inservice inspection (Specification 4.9.C)
- g. Inoperable plant vent sampling capability (Table 3.5-4 item 5)

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of changes made to Operating Procedures.
- f. Records of radioactive shipments.
- g. Records of sealed source and fission detector leak tests and results.
- h. Records of annual physical inventory of all source material of record.
- i. Records of reactor tests and experiments.

7/ Sealed sources or calibration sources are not included under this item. Leakage of packing, caskets, mechanical joints and seal welds within the limits for identified leakage set forth in technical specifications need not be reported under this item.

ATTACHMENT II

SAFETY EVALUATION

RELATED TO

NUREG 0737, item II.F.1.2

POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT 3 NUCLEAR POWER PLANT
DOCKET NO. 50-286
February 1, 1982

SECTION I - Description of Modification

The proposed changes to Appendix A (Technical Specifications) of the Indian Point 3 Operating License seeks to amend Sections 3.5,

4.1 and 6.9.2. The proposed changes add a limiting condition for operation and a surveillance requirement in accordance with your October 7, 1981 and November 18, 1981 letters. These letters requested proposed Technical Specifications for NUREG-0737 item II.F.1.2 which provides for sampling of radioiodine and particulates in the plant effluent.

SECTION II - Purpose of the Modification

The purpose of this modification is to fully implement the requirements of NUREG-0737, item II.F.1.2 in accordance with the Commission's October 7, 1981 and November 18, 1981 letter. This change will ensure the capability to assess post accident plant effluents for radioiodine and particulates.

SECTION III - Impact of the Change

This change will be consistent with the ALARA program.

This change will not impact the following:

- Fire Protection Program
- Emergency Plan
- FSAR or SER Conclusions
- Overall Plant Operations

SECTION IV - Conclusion

The incorporation of these modifications: a) will not change the probability nor the consequence of an accident or malfunction of

equipment important to safety as previously evaluated in the Safety Analysis Report; b) will not increase the possibility for an accident or malfunction of a different type than any evaluated previously in the Safety Analysis Report; c) will not reduce the margin of safety as defined in the basis for any Technical Specification; and d) does not constitute an unreviewed safety question.

SECTION V - References

(a) IP3 FSAR

(b) IP3 SER