



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 14, 2009

Vice President, Operations
Entergy Operations, Inc.
River Bend Station
5485 U.S. Highway 61N
St. Francisville, LA 70775

SUBJECT: RIVER BEND STATION, UNIT 1 - CORRECTION OF ERRORS IN
AMENDMENT NOS. 165 AND 166 (TAC NOS. ME0406 AND ME0157)

Dear Sir or Madam:

By letters dated August 11 and September 29, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML092010370 and ML092430145, respectively), the Nuclear Regulatory Commission (NRC) staff issued Amendments Nos. 165 and 166, respectively, to Renewed Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. By letter dated November 30, 2009 (ADAMS Accession No. ML093380291), Entergy Operations, Inc. (Entergy, the licensee), noted errors in Technical Specification (TS) pages issued by Amendment Nos. 165 and 166. Specifically, for Amendment No. 165, TS page 3.8-10 Surveillance Requirement (SR) 3.8.1.12.c was issued with the 3740 value missing the denomination (V – volts). In Amendment No. 166, TS page 5.0-19, TS 5.6.5 was issued with the incorrect title for the reference 25 for the Core Operating Limits Report. The errors were inadvertently introduced when the NRC staff was processing Amendment Nos. 165 and 166. Thus, according to the NRC's policy established by SECY-96-238 (ADAMS Accession No. 9611250030), these errors can be corrected by a letter. Enclosed please find the replacement pages 3.8-10 for Amendment No. 165 and 5.0-19 for Amendment No. 166.

This correction does not change any of the conclusions in the safety evaluations associated with the amendments. We apologize for any inconvenience caused by this error.

Sincerely,

A handwritten signature in cursive script that reads "Alan Wang".

Alan B. Wang, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-458

Enclosures:

1. Amendment No. 165, TS page 3.8-10
2. Amendment No. 166, TS page 5.0-19

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ENCLOSURE 1

REPLACEMENT TECHNICAL SPECIFICATION PAGE 3.8-10

FOR AMENDMENT NO. 165

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.12</p> <p>-----NOTES-----</p> <ol style="list-style-type: none"> 1. All DG starts may be preceded by an engine prelube period. 2. This Surveillance shall not be performed in MODE 1 or 2. (Not applicable to DG 1C) However, credit may be taken for unplanned events that satisfy this SR. <p>-----</p> <p>Verify on an actual or simulated Emergency Core Cooling System (ECCS) initiation signal each DG auto-starts from standby condition and:</p> <ol style="list-style-type: none"> a. For DG 1C during the auto-start maintains voltage ≤ 5400 V and frequency ≤ 66.75 Hz; b. In ≤ 10 seconds for DG 1A and DG 1B and ≤ 13 seconds for DG 1C after auto-start and during tests, achieves voltage ≥ 3740 V and frequency ≥ 58.8 Hz. c. Achieves steady state voltage ≥ 3740 V and ≤ 4580 V and frequency ≥ 58.8 Hz and ≤ 61.2 Hz; and d. Operates for ≥ 5 minutes. 	<p>18 months</p>

(continued)

ENCLOSURE 2

REPLACEMENT TECHNICAL SPECIFICATION PAGE 5.0-19

FOR AMENDMENT NO. 166

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

- 24) NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel (GESTAR-II)".
 - 25) NEDC-33383P, "GEXL97 Correlation Applicable to ATRIUM-10 Fuel," Global Nuclear Fuel.
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.
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/RA/

Alan B. Wang, Project Manager
Plant Licensing Branch IV
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ADAMS Accession No. ML093360581

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DATE	12/09/09	12/7/09	12/14/09	12/14/09

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