PWR Examination Outline

Facility: SEABI	ROOK RO See	ction									Date	e of E	Exam: 10/	/1/9				
					F	ro k	/A C	ateg	ory F	Point	s				SR	O-On	ly Poin	ts
Tier	Group	К 1	К 2	К 3	К 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	ļ	2	C	3*	Total
1.	1	3	2	3				4	3			3	18					6
Emergency & Abnormal	2	1	1	1		N/A		3	2	N	A	1	9					4
Plant Evolutions	Tier Totals	4	3	4				7	5			4	27					10
	1	3	3	5	4	1	2	2	2	1	2	3	28					5
2. Plant	2	2	1	0	3	1	0	1	1	1	0	0	10					3
Systems	Tier Totals	5	4	5	7	2	2	3	3	2	2	3	38					8
3. Generic K	(nowledge and	Abili	ties		,	1		2		3	4	1	10	1	2	3	4	7
(Categories				3			2	ź	2	-	3						
Note: 1. 2. 3. 4. 5. 6. 5 7.* 7 8. 0	Ensure that at le and SRO-only ou in each K/A cate The point total fo The final point to The final RO exa Systems/evolutio at the facility sho included on the of of inappropriate Select topics from selecting a secon Absent a plant-si Use the RO and Select SRO topic The generic (G) H must be relevant On the following is for the applicable for each categor SRO-only exam, pages for RO an	ast tw utiline gory r eac tal fo util fo util fo util fo to tal fo m m with tal fo m m with tal fo m m with tal fo m m with tal fo tal fol tal fol	vo top s (i.e. shall h groor r eac ust to thin e e del e sho tatem many bic fo c price ratin Tiers n Tie s, ent nse le ne tab r it or O-on	bics fir up and b up and b h groot b h g	rom e ept fc e less and tie up ar 5 poir roup and j e ado ems s syste only t r the d 2 fr and 2 ble ev K/A and ti bove; left si ams.	every or one s that r in that tiee hts an are ic ustified ded. and e em or hose RO a om th shall volution ne po if fue de of	appli appli a cate n two ne pro- r may d the lentifi ed; op Refe evolut r evol K/As nd S he sh. l be s on or bers, int to l han colut the k	cable egory). oppose v devide SRC ed on perati r to S cions utions utions havia RO-o aded elect syste dallog umn A	K/A in Ti ed ou ate b D-only the a onally section as po ng ar nly p syste ed fro ern. F ef de ef de equi 2 for	categ er 3 o tline r y ±11 v exar sssoci v impo n D.1 sssible i impo portion erns a erns a Refer script each omen Tier :	ory a f the nust from i m mu iated ortan b of i e; sar ortan s, res nd K, ection to Se ion or i syst t is si 2, Gr	rre sa SRO matc: that s ist tot outlin t, site ES-4 mple ce rat spect /A ca 2 of ection f eacl em a ample oup 2	impled with -only outlin -only outlin -only outlin -pecified in al 25 point -specific s 01 for guid every syst -specific s 01 for guid every syst - ing (IR) of ively. tegories. the K/A C: - D.1.b of E h topic, the h topic, the h topic, the A categories 2 (Note #1	hin eache, the cified in the ta ts. or evo ystems ance r 2.5 or atalog, ES-401 e topics ry. En does r	ch tier o "Tier T h the tal ble bas plutions s/evolutions evolution higher but the for the s' impor ter the g Categor not appl	f the R otals" ble. ed on 1 that do ions th g the e on in the shall b shall b tance r group a group a y A2 on y). Us	O NRC re- not app at are r eliminati e group e select able K// ratings (and tier ' G* on se duplic	visions. ly ot on before ed. As. IRs) totals the cate

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ES-401 Emergence	;y ar	nd A	bno	PV rma	/R Ex I Plai	kamin ht Evc	ation Outline Fo Iutions - Tier 1/Group 1 (RO / SRO)	orm ES-4	401-2
E/APE # / Name / Safety Function	К 1	К 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
000007 (BW/E02&E10 CE/E02) Reactor Trip - Stabilization - Recovery / 1	x						EK1 Knowledge of the operational implications of the following concepts as they apply to the reactor trip: EK1.06 Relationship of emergency feedwater flow to S/G and decay heat removal following reactor trip. (CFR 41.8/41.10/45.3)	3.7	1
000008 Pressurizer Vapor Space Accident / 3									
000009 Small Break LOCA / 3	x						EK1 Knowledge of the following concepts as they apply to the small break LOCA: EK1.02 Use of steam tables (CFR 41.8/41.10/45.3)	3.5	1
000011 Large Break LOCA / 3			х		2000 		EK3 Knowledge of the reasons for the following responses as they apply to the Large Break LOCA EK3.12 Actions contained in EOP for emergency LOCA (large break) (CFR 41.5/41.10/45.6/45.13)	4.4	1
000015/17 RCP Malfunctions / 4					x		AA2 Ability to determine and interpret the following as they apply to the Reactor Coolant Malfunctions: AA2.01 Cause of RCP failure (CFR 43.5/45.13)	3.0	1
000022 Loss of Rx Coolant Makeup / 2						AND			
000025 Loss of RHR System / 4									
000026 Loss of Component Cooling Water / 8						x	2.4.6 Knowledge of EOP mitigation strategies. (CFR 41.10/43.5/45.13)	3.7	1
000027 Pressurizer Pressure Control System Malfunction / 3		x					AK2 Knowledge of the interrelationships beween the Pressurizer Pressure Control Malfunctions and the following: AK2.03 Controllers and positioners (CFR 41.7/45.7)	2.6	1
000029 ATWS / 1				x			EA1 Ability to operate and monitor the following as they apply to ATWS: EA1.14 Driving of control rods into the core (CFR 41.7/45.5/45.6)	4.2	1
000038 Steam Gen. Tube Rupture / 3						X	2.2.25 Knowledge of the bases in Technical Specifications for limiting conditions for operation and safety limits. (CFR 41.5/41.7/43.2)	3.2	1

ES-401 Emergence	y an	d Al	bno	PW rma	/R Ex I Plai	kamin nt Evo	ation Outline Fc Jutions - Tier 1/Group 1 (RO / SRO)	orm ES-	401-2
E/APE # / Name / Safety Function	к 1	к 2	к 3	A 1	A 2	G	K/A Topic(s)	IR	#
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4				x			AA1 Ability to operate and/or monitor the following as they apply to the Steam Line Rupture:	3.4	1
							AA1.09 Setpoints of main steam safety and PORV's.		
000054 (CE/E06) Loss of Main Feedwater / 4						X	(CFR 41.//45.5/45.6 2.4.21 Knowledge of the parameters and logic used to assess the status of safety functions, such as reactivity control, core cooling and heat removal, reactor coolant system integrity, containment conditions, radioactivity release control, etc.	4.0	1
000055 Station Blackout / 6					1.22.11 1000-000-		(CFR 41.7/43.5/45.12)		
000056 Loss of Off-site Power / 6			x				AK3 Knowledge of the reasons for the following responses as they apply to the Loss of Offsite Power:	4.4	1
							AK3.02 Actions contained in the EOP for loss of offsite power		
							(CFR 41.5/41.10/45.6/45.13)		
000057 Loss of Vital AC Inst. Bus / 6					x		AA2 Ability to determine and interpret the following as they apply to the Loss of Vital AC Instrument Bus:	3.2	1
							AA2.06 AC instrument bus alarms for the inverter and alternate power source		
							(CFR 43.5/45.13)		
000058 Loss of DC Power / 6	x						AK1 Knowledge of the operational implications of the following concepts as they apply to Loss of DC Power:	2.8	1
							AK1.01 Battery charger equipment and instrumentation		
							(CFR 41.8/41.10/45.3)		
000062 Loss of Nuclear Svc Water / 4			x				AK3 Knowledge of the reasons for the following responses as they apply to the Loss of Nuclear Service Water:	4.0	1
							AK3.03 Guidance actions contained in EOP for loss of nuclear service water		
							(CFR 41.4/41.8/45.7)		
000065 Loss of Instrument Air / 8				x			AA1 Ability to operate and/or monitor the following as they apply to the Loss of Instrument Air:	3.3	1
							AA1.05 RPS		
							(CFR 41.7/45.5/45.6)		

ES-401 Emergence	y an	d A	bno	PV rma	/R E: I Plai	camin nt Evo	ation Outline For Iutions - Tier 1/Group 1 (RO / SRO)	m ES-4	401-2
E/APE # / Name / Safety Function	к 1	к 2	к 3	A 1	A 2	G	K/A Topic(s)	IR	#
W/E04 LOCA Outside Containment / 3		x					EK2 Knowledge of the interrelations between the LOCA Outside Containment and the following: EK2.1 Components and functions of control and safety systems, including instrumentation, signals, interlocks, failure modes, and automatic and manual features (CFR 41.7/45.7)	3.5	1
W/E11 Loss of Emergency Coolant Recirc. / 4				x			EA1 Ability to operate and/or monitor the following as they apply to the Loss of Emergency Coolant Recirculation EA1.2 Operating behavior characteristics of the facility (CFR 41.7/45.5/45.6)	3.5	1
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4					X		EA2 Ability to determine and interpret the following as they apply to the Loss of Secondary Heat Sink EA2.1 Facility conditions and the selection of appropriate procedures during abnormal and emergency operations (CFR 43.5/45.13)	3.4	1
000077 Generator Voltage and Electric Grid Disturbances / 6									
K/A Category Totals:	3	2	3	4	3	3	Group Point Total:		18/ 6

ES-401 Emergency and A	P' bnorm	WR al P	Exa lant	mina Evol	ation lution	Outlir s - Ti	ne Fc ier 1/Group 2 (RO / SRO)	rm ES-	401-2
E/APE # / Name / Safety Function	K 1	к 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
000001 Continuous Rod Withdrawal / 1									
000003 Dropped Control Rod / 1				x			AA1 Ability to operate and/or monitor the following as they apply to the Dropped Control Rod AA1.02 Controls and components necessary to recover rod (CFR 41.7/45.5/45.6)	3.6	1
000005 Inoperable/Stuck Control Rod / 1									
000024 Emergency Boration / 1									
000028 Pressurizer Level Malfunction / 2	x				angrafia Sangraf		AK1 Knowledge of the operational implications of the following concepts as they apply to Pressurizer Level Control Malfunctions AK1.01 PZR reference leg abnormalities (CER 41 8/41 10/45 3)	2.8	
000032 Loss of Source Range NI / 7					en de la composition Carlos e administration Carlos e administration Carlos e administration				
000033 Loss of Intermediate Range NI / 7									
000036 (BW/A08) Fuel Handling Accident / 8		x					AK2 Knowledge of the interrelationships between Fuel Handling Accidents and the following: AK2.02 Radiation monitoring equipment (portable and installed)	3.4	1
000037 Steam Generator Tube Leak / 3				x			AA1 Ability to operate and/or monitor the following as they apply to the Steam Generator Tube Leak: AA1.11 PZR level indication (CFR 41.7/45.5/45.6)	3.4	1
000051 Loss of Condenser Vacuum / 4			x				AK3 Knowledge of the reasons for the following responses as they apply to the Loss of Condenser Vacuum: AK3.01 Loss of steam dump capability upon loss of condenser vacuum (CFR 41.5/41.10/45.6/45.13)	2.8	1
000059 Accidental Liquid RadWaste Rel. / 9									
000060 Accidental Gaseous Radwaste Rel. / 9									
000061 ARM System Alarms / 7						سمر واروز			
000067 Plant Fire On-site / 8					X		AA2 Ability to determine and interpret the following as they apply to the Plant Fire on site: AA2.06 Need for pressurizing control room (recirculation mode) (CFR 43.5/45.13)	3.3	1
000068 (BW/A06) Control Room Evac. / 8									

ES-401 Emergency and Abi	P' norm	WR al P	Exa lant	mina Evol	ntion ution	Outlir s - Ti	ne Fo ier 1/Group 2 (RO / SRO)	rm ES-	401-2
E/APE # / Name / Safety Function	К 1	К 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
000069 (W/E14) Loss of CTMT Integrity / 5									
000074 (W/E06&E07) Inad. Core Cooling / 4									
000076 High Reactor Coolant Activity / 9									
W/EO1 & E02 Rediagnosis & SI Termination / 3									
W/E13 Steam Generator Over-pressure / 4									
W/E15 Containment Flooding / 5					X		EA2 Ability to determine and interpret the following as they apply to the Containment Flooding: EA2.1 Facility conditions and selection of appropriate procedures during abnormal and emergency conditions	2.7	1
W/E16 High Containment Radiation / 9				x			(CFR 43.5/45.13) EA1 Ability to operate and/or monitor the following as they apply to the High Containment Radiation:	3.1	1
							EA1.1 Components and functions of control and safety systems including instrumentation, signals, interlocks, failure modes, and automatic and manual features.		
BW/A01 Plant Runback / 1									
BW/A02&A03 Loss of NNI-X/Y / 7									
BW/A04 Turbine Trip / 4					¥. Serie				
BW/A05 Emergency Diesel Actuation / 6									
BW/A07 Flooding / 8									
BW/E03 Inadequate Subcooling Margin / 4					eży				
BW/E08; W/E03 LOCA Cooldown - Depress. / 4									
BW/E09; CE/A13; W/E09&E10 Natural Circ. / 4									
BW/E13&E14 EOP Rules and Enclosures									
CE/A11; W/E08 RCS Overcooling - PTS / 4						x	2.4.18 Knowledge for the specific bases for EOP's	3.3	1
						અલેલ્લુલ્સ્ સંસ્કૃતિવર્શન	(CFK 41.10/43.1/45.13)		
			L		ni. Tarah				
									0/4
K/A Category Point Totals:		1	1	3	2	ा	Group Point Total:		9/4

ES-401				Plar	it Sy	PW yster	RE ns-	xamir Tier	natio 2/Gr	n O oup	utline 1 (R	Fo O / SRO)	rm ES-4	401-2
System # / Name	К 1	к 2	К 3	К 4	к 5	К 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
003 Reactor Coolant Pump		x										K2 Knowledge of bus power supplies to the following: K2.01 RCPs (CFR 41.7)	3.1	1
004 Chemical and Volume Control						x						K6 Knowledge of the effect of a loss or malfunction on the following CVCS components: K6.07 Heat exchangers and condensers (CFR 41.7/45.7) 2.1.23 Ability to perform specific system and integrated plant procedures during all modes of operation (CFR 41.10/43.5/45.2/45.6)	2.7 3.9	2
005 Residual Heat Removal				x								K4 Knowledge of RHRS design feature (s) and/or interlock (s) which provide the following: K4.03 RHR heat exchanger bypass flow control (CFR 41.7) 2.1.32 Ability to explain and apply system limits and precautions (CFR 41.10/43.2/45.12)	2.9 3.8	2
006 Emergency Core Cooling										x		A4 Ability to manually operate and/or monitor from the control room A4.11 Over pressure protection system.	4.2	1
007 Pressurizer Relief/Quench Tank			x									K3 Knowledge of the effect that a loss or malfunction of the PRTS will have on the following: K3.01 Containment (CFR 41.7/45.6)	3.3	1

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ES-401				Plar	nt Sy	PW /ster	RE ns-	xamii Tier	natio 2/Gr	n O oup	utline 1 (R	FC FC FC	orm ES-4	401-2
System # / Name	к 1	К 2	к 3	к 4	к 5	к 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
008 Component Cooling Water	x											K1 Knowledge of the physical connections and/or cause-effect relationships between the CCWS and the following systems: K1.02 Loads cooled by CCWS (CFR 41.2 to 41.9/45.7 to 45.9) A2 Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the CCWS controls including: A2.01 Loss of CCW pump	3.3 3.3	2
010 Pressurizer Pressure Control				x								K4 Knowledge of PZR PCS design features and/or interlocks which provide for the following: K4.03 Over pressure control (CFR 41.7)	3.8	1
012 Reactor Protection						x						K6 Knowledge of the effect of loss or malfunction that the following will have on the RPS: K6.10 Permissive circuits (CFR 41.7/45.7) A2 Ability to (a) predict the impacts of the following malfunctions or operations on the RPS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.02 Loss of instrument power (CFR 41.5/43.5/45.3/45.5)	3.3 3.6	2
013 Engineered Safety Features Actuation										x		A4 Ability to manually operate and/or monitor in the control room: A4.01 ESFAS –initiation equipment which fails to actuate (CFR 41.7/45.5 to 45.8)	4.5	1
022 Containment Cooling 025 Ice Condenser		x										K2 Knowledge of power supplies for the following: K2.01 Containment cooling fans (CFR 41.7/45.6)	3.0	1

ES-401				Plar	nt Sy	PW /ster	RE	ixamir - Tier	natio 2/Gr	n O oup	utline 1 (R	Fo O / SRO)	rm ES-	401-2
System # / Name	К 1	К 2	К 3	К 4	K 5	К 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
026 Containment Spray				x								K4 Knowledge of the CSS design feature(s) and/or interlock(s) which provide for the following: K4.08 Automatic swapover to containment sump suction for recirculation phase after LOCA(RWST low-low level alarm) (CFR 41.7)	4.1	1
039 Main and Reheat Steam					x							K5 Knowledge of the operational implications of the following concepts as they apply to the MRSS: K5.05 Bases for RCS cooldown limits (CFR 41.5/45.7)	2.7	1
059 Main Feedwater			x									K3 Knowledge of the effect that a loss or malfunction of the MFW will have on the following: K3.02 AFW system (CFR 41.7/45.6)	3.6	1
061 Auxiliary/Emergency Feedwater		X	x									K2 Knowledge of bus power supplies to the following: K2.02 AFW electric drive pumps (CFR 41.7) K4 Knowledge of AFW design feature(s) and/or interlock(s) which provide for the following: K4.08 AFW recirculation (CFR 41.7)	3.7 2.7	2
062 AC Electrical Distribution							x		x			A1 Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the ac distribution system controls including: A1.01 Significance of D/G load limits (CFR 41.5/45.5) A3 Ability to monitor automatic operation of the ac distribution system, including: A3.05 Safety related indicators and controls (CFR 41.7/45.5)	3.4	2

ES-401				Plar	nt Sy	PW /ster	RE ns-	xamir Tier	natio 2/Gr	n O oup	utline 1 (Ri	For O / SRO)	m ES-4	401-2
System # / Name	к 1	к 2	к 3	К 4	к 5	к 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
063 DC Electrical Distribution	x											K1 Knowledge of the physical connections and/or cause-effect relationships between the DC electrical system and the following systems: K1.02 AC electrical system (CFR 41.2 to 41.9/45.7 to 45.8)	2.7	1
064 Emergency Diesel Generator											X	2.1.7 Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation. (CFR 41.5/43.5/45.12/45.13)	4.4	1
073 Process Radiation Monitoring							x					A1 Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the PRM system controls including: A1.01 Radiation levels (CFR 41.5/45.7)	3.2	1
076 Service Water			x									K3 Knowledge of the effect that a loss or malfunction of the SWS will have on the following: K3.01 Closed cooling water. (CFR 41.7/45.6)	3.4	1
078 Instrument Air	x		x									K1 Knowledge of the physical connections and/or cause-effect relationships between the IAS and the following systems: K1.02 Service air (CFR 41.2 to 41.9/45.7 to 45.8) K3 Knowledge of the effect that a loss or malfunction of the IAS will have on the following: K3.02 Systems having pneumatic valves and controls (CFR 41.7/45.6)	3.4	2
103 Containment				x								K4 Knowledge of the containment system design feature(s) and/or interlock(s) which provide for the following: K4.06 Containment isolation system (CFR 41.7)	3.1	1
K/A Category Point Totals:	3	3	5	4	1	2	2	2	1	2	3	Group Point Total:		28/ 5

ES-401				Pla	nt S	PV Syste	VR ems	Exan - Tie	nina er 2/	tion Gro	Outli up 2 (ne Fo (RO / SRO)	rm ES-4	401-2
System # / Name	K 1	к 2	K 3	K 4	к 5	к 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
001 Control Rod Drive				x								K4 Knowledge of CRDS design feature(s) and/or interlock(s) which provide for the following: K4.03 Rod control logic (CFR 41.7)	3.5	1
002 Reactor Coolant											al			
011 Pressurizer Level Control									x			A3 Ability to monitor automatic operation of the PZR LCS, including: A3.03 Charging and letdown	3.2	1
014 Rod Position Indication							x					A1 Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the RPIS controls, including: A1.03 PDIL,PPDIL (CFR 41.5/45.5)	3.6	1
015 Nuclear Instrumentation												**************************************		
016 Non-nuclear Instrumentation														
017 In-core Temperature Monitor				x								K4 Knowledge of the ITM system design features an/or interlocks which provide for the following: K4.01 Input to subcooling monitors. (CFR 41.7)	3.4	1
027 Containment Iodine Removal								11 2ú						
028 Hydrogen Recombiner and Purge Control											100			
029 Containment Purge														
033 Spent Fuel Pool Cooling														
034 Fuel Handling Equipment														
035 Steam Generator					x							K5 Knowledge of the operational implications of the following concepts as they apply to the S/Gs: K5.01 Effect of secondary parameters, pressure, and temperature on reactivity (CFR 41.5/45.7)	3.4	1

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ES-401				Pla	int S	P۱ Syst	NR ems	Exan 3 - Tie	nina er 2/	tior Gro	n C our	Dutlii b 2 (ne Foi RO / SRO)	rm ES-4	401-2
System # / Name	К 1	К 2	к 3	K 4	к 5	к 6	A 1	A 2	A 3	A 4		G	K/A Topic(s)	IR	#
041 Steam Dump/Turbine Bypass Control													A2 Ability to (a) predict the impacts of the following malfunctions or operations on the SDS; and (b) based on those predictions use procedures to correct, control, or mitigate the consequences of those malfunctions or operations A2.02 Steam valve stuck open (CFR 41.5/43.5/45.3/45.13)	3.6	1
045 Main Turbine Generator												Си-т. Т			
055 Condenser Air Removal	x											B at	K1 Knowledge of the physical connections and/or cause-effect relationships between the CARS and the following systems: K1.06 PRM (process radiation monitoring) system	2.6	1
													(CER 41 2 to 41 9/45 7 to 45 8)		
056 Condensate	X												K1 Knowledge of the physical connections and/or cause-effect relationships between the condensate system and the following systems: K1.03 MFW (CFR 41.2 to 41.9/ 45.7 to 45.8)	2.6	1
068 Liquid Radwaste											1 A.				
071 Waste Gas Disposal											1945 1945				
072 Area Radiation Monitoring															
075 Circulating Water		x											K2 Knowledge of bus power supplies to the following: K2.03 Emergency/essential SWS pumps (CFR 41.7)	2.6	1
079 Station Air				x							A MAN		K4 Knowledge of SAS design feature(s) and/or interlock(s) which provide for the following: K4.01 Cross-connect with IAS (CFR 41.7)	2.9	1
086 Fire Protection															
K/A Category Point Totals:	2	1	0	3	1	0	1	1	1	0		0	Group Point Total:		10/ 3

Facility: SEABF	ROOK	Date of Exam: 6/15/09				
Category	K/A #	Торіс	R	0	SRO-	Only
			IR	#	IR	#
	2.1.3	Knowledge of shift or short-term relief turnover practices.	3.7	1		
Conduct		(CFR 41.10/45.13)				
of Operations	2.1.25	Ability to interpret reference materials, such as graphs, curves, tables, etc.	3.9	1		
		(CFR 41.10/43.5/45.12)				
	2.1.26	Knowledge of industrial safety procedures (such as rotating equipment, electrical, high temperature, high pressure, caustic, chlorine, oxygen, and hydrogen.	3.4	1		
		(CFR 41.10/45.12)				
	2.1.					
	2.1.					
	2.1.					
	Subtotal	P		3	10.00	
2	2.2.2	Ability to manipulate the console controls as required to operate the facility between shutdown and designated power levels.	4.6	1		
Equipment		(CFR 4160/41.7/45.2)				
Control	2.2.22	Knowledge of limiting conditions for operation and safety limits.	4.0	1		
		(CFR 41.5/43.2/45.2)				
	2.2.					
	2.2.					
	2.2.					
	2.2.					
	Subtotal			2		
3	2.3.5	Ability to use radiation monitoring systems, such as fixed radiation monitors and alarms, portable survey instruments, personal monitoring equipment, etc.	2.9	1		
Radiation		(CFR 41.11/41.12/43.4/45.9)				
Control	2.3.11	Ability to control radiation releases.	3.8	1		
		(CFR 41.11/43.4/45.10)				
	2.3.					
	2.3.					
	2.3.					

Facility: SEABR	OOK	Date of Exam: 6/15/09				
Category	K/A #	Торіс	R	0	SRO	Only
			IR	#	IR	#
	2.3.					
	Subtotal			2		
	2.4.6	Knowledge of EOP mitigation strategies.	3.7	1		
4.		(CFR 41.10/43.5/45.13)				
Emergency Procedures /	2.4.11	Knowledge of abnormal condition procedures.	4.0	1		
Plan		(CFR 41.10/43.5/45.13)				
	2.4.42	Knowledge of emergency response facilities.	2.6	1		
		(CFR 41.10/45.11)				
	2.4.					
	2.4.			~		
	2.4.					
	Subtotal			3		
Tier 3 Point Total				10		7

PWR Examination Outline

Facility: SEABROOK SRO section Date of Exam: 10/1/9																		
		RO K/A Category Points SRO-Only Points K K K A A A														ts		
Tier	Group	К 1	к 2	К 3	К 4	К 5	к 6	A 1	A 2	A 3	A 4	G *	Total	Å	42	(G*	Total
1.	1												18		4		2	6
Emergency & Abnormal	2					N/A							9		2		2	4
Plant Evolutions	Tier Totals												27	1	6		4	10
	1												28		2		3	5
2. Plant	2												10		3		0	3
Systems	Tier Totals												38		5		3	8
3. Generic K	Knowledge and	Abili	ities			1		2		3	4	4	10	1	2	3	4	7
	Categories																	
Note: 1. 2. 3. 4. 5. 6. 5. 7.* 8. 9.	Ensure that at le and SRO-only or in each K/A cate The point total for The final point to The final RO exa Systems/evolutio at the facility sho included on the or of inappropriate Select topics from selecting a secon Absent a plant-s Use the RO and Select SRO topic The generic (G) H must be relevant On the following for the applicable for each categor SRO-only exam, pages for RO and For Tier 3, select and point totals (vast tv utline gory or eaco tal fo am m sns wi bould b outlin K/A s m as nd top pecifit SRO ss for t to til page: e lice t to til page: e lice d SR t to pil (#) or	wo top s (i.e shall ch grcd shall ch grcd and shall be dell e sho staten many pic fo c price ratin Tiers in Tiers in Tiers s, enfines he tall er it or Co-on c s from b Form	bics fit , exc not b bup an h groo ttal 7 sch g eted uid b nents / syst r any gs fo 1 an ers 1 a plical er tha evel, a ble ab n the ly exa m Se m Se	rom (cept fi- ce les nd tie up all 5 poin group froup for and 5 poin group and 15 r the all 2 fi and 2 fi and 2 fi and 2 fi and 2 fi left s assst cove; left s ams.	every or ond s that er in the nd tie are ic ustified ded. and e em or those RO a com the shall volution if fue ide of 0.2 of 1.2 of 3. Li	appli e catu n two ne pr r may nd the dentified; op Refe evolu r evol K/As ne sh l be s on or bers, bint to l har f Colu the H mit S	icable egory y device s SRC ied or operati r to S itions lutions RO-o aded select syste , a bri itals (udling umn A	e K/A in Ti- ed ou iate b D-only the a onality ection as po ng ar nly p syste ed for em. F ef de et #) for equij	categer 3 c tline r y ±1 1 r example assoc r importion assible i importion arms a except Refer script each ormen Tier	gory a go	are sa SRO matcl that s ist tot outlin t, site ES-4 mple ce rat spect /A ca 1 2 of f eacl tem a ample oup 2 or the	mpled with -only outlin pecified in al 25 point e; systems -specific s D1 for guid every syste ing (IR) of ively. tegories. the K/A Cz D.1.b of E in topic, the nd categor ed in other (Note #1	hin each ne, the cified in the ta s. or evo ystems ance r 2.5 or 2.5 or atalog, S-401 a topics y. En than C does r ers, de	ch tier o "Tier T in the tal ble bas plutions s/evolutions evolution higher but the for the s' importer the g Categor not applion	f the R otals" ole. ed on that do ions th g the c on in th shall b topics applic tance group a y A2 o y). Us	O NRC re- not app at are n eliminati e group e select able K// ratings (and tier r G* on se duplic	visions. Ny iot on before red. As. (IRs) totals the cate

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B-401 PWR Examination Outline Emergency and Abnormal Plant Evolutions - Tier 1/Group 1 (RO / SRO) Form ES-401-2 APE # / Name / Safety Function K K A G K/A Topic(s) IR #													
E/APE # / Name / Safety Function	к 1	К 2	к 3	A 1	A 2	G	K/A Topic(s)	IR	#				
000007 (BW/E02&E10 CE/E02) Reactor Trip - Stabilization - Recovery / 1													
000008 Pressurizer Vapor Space Accident / 3													
000009 Small Break LOCA / 3					x	1	EA2 Ability to determine or interpret the following as they apply to a small break LOCA:	4.2	1				
							EA2.34 Conditions for throttling or stopping HPI						
000011 Large Break LOCA / 3							(CFR 43.5/45.13)						
000015/17 RCP Malfunctions / 4						x	2.1.23 Ability to perform specific system and integrated plant procedures during all modes of operation.	4.4	1				
							(CFR 41.10/43.5/45.2/45.6)						
000022 Loss of Rx Coolant Makeup / 2					X		AA2 Ability to determine and interpret the following as they apply to the Loss of Reactor Coolant Makeup:	3.6	1				
							AA2.03 Failures of flow control valve or controller (CER 43 5/45 13)						
000025 Loss of RHR System / 4													
000026 Loss of Component Cooling Water / 8													
000027 Pressurizer Pressure Control System Malfunction / 3					X		AA2 Ability to determine and interpret the following as they apply to the Pressurizer Pressure Control Malfunctions:	3.9	1				
							AA2.06 Conditions requiring plant shutdown						
							(CFR 43.5/45.13)						
000029 ATWS / 1													
000038 Steam Gen. Tube Rupture / 3					X 101		EA2 Ability to determine or interpret the following as they apply to a SGTR:	3.3	1				
					-+4] S./12		EA2.10 Flowpath for charging and letdown flows						
					24. 1987) 1997 - 1997 1997 - 1997 - 1997	See 9	(CFR 43.5/45.13)						
Steam Line Rupture - Excessive Heat Transfer / 4													
000054 (CE/E06) Loss of Main Feedwater / 4					158 28								
000055 Station Blackout / 6													
000056 Loss of Off-site Power / 6													
000057 Loss of Vital AC Inst. Bus / 6													

ES-401 Emergenc	y an	id A	bnoi	PW rma	/R E> I Plar	amin t Evo	ation Outline Fo Iutions - Tier 1/Group 1 (RO / SRO)	rm ES-4	401-2
E/APE # / Name / Safety Function	К 1	К 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
000058 Loss of DC Power / 6						3			
000062 Loss of Nuclear Svc Water / 4									
000065 Loss of Instrument Air / 8						x	2.1.32 Ability to explain and apply system limits and precautions.	4.0	1
						1000	(CFR 41.10/43.2/45.12)		
W/E04 LOCA Outside Containment / 3									
W/E11 Loss of Emergency Coolant Recirc. / 4									
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4									
000077 Generator Voltage and Electric Grid Disturbances / 6									
					. N				
K/A Category Totals:					4	2	Group Point Total:		18/ 6

3

ES-401 Emergency and	F Abnom	PWF nal	R Ex Plar	ami nt Ev	inatic voluti	n Oul ons -	tline Fo Tier 1/Group 2 (RO / SRO)	orm ES-	401-2
E/APE # / Name / Safety Function	К 1	K 2	к 3	A 1	A 2	G	K/A Topic(s)	IR	#
000001 Continuous Rod Withdrawal / 1					Ju				
000003 Dropped Control Rod / 1									
000005 Inoperable/Stuck Control Rod / 1					X		AA2 Ability to determine and interpret the following as they apply to the Inoperabel/Stuck Rod: AA2.03 Required actions if more than one rod is stuck or inoperable	4.4	1
000024 Emergency Boration / 1					4		(011(45.045.15)		
000028 Pressurizer Level Malfunction / 2									
000032 Loss of Source Range NI / 7					1 21.32	24			
000033 Loss of Intermediate Range NI / 7						Nation.			
000036 (BW/A08) Fuel Handling Accident / 8					•				
000037 Steam Generator Tube Leak / 3					Çati î.	- 14			
000051 Loss of Condenser Vacuum / 4									
000059 Accidental Liquíd RadWaste Rel. / 9						X	2.3.11 Ability to control radiation releases. (CFR 41.11/43.4/45.10)	4.3	1
000060 Accidental Gaseous Radwaste Rel. / 9						65 19			
000061 ARM System Alarms / 7									
000067 Plant Fire On-site / 8					ni 1940 - E				
000068 (BW/A06) Control Room Evac. / 8					ana a	Kiiki 2x			
000069 (W/E14) Loss of CTMT Integrity / 5									
000074 (W/E06&E07) Inad. Core Cooling / 4									
000076 High Reactor Coolant Activity / 9									
W/EO1 & E02 Rediagnosis & SI Termination / 3									
W/E13 Steam Generator Over-pressure / 4							an		
W/E15 Containment Flooding / 5									
W/E16 High Containment Radiation / 9									
BW/A01 Plant Runback / 1									
BW/A02&A03 Loss of NNI-X/Y / 7					Sel Cre Concella				
BW/A04 Turbine Trip / 4					(44)-2-15 (146-15)	ų			
BW/A05 Emergency Diesel Actuation / 6						. NH			
BW/A07 Flooding / 8									

ES-401 PWR Examination Outline Form ES-401-2 Emergency and Abnormal Plant Evolutions - Tier 1/Group 2 (RO / SRO)												
E/APE # / Name / Safety Function	К 1	K 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#			
BW/E03 Inadequate Subcooling Margin / 4												
BW/E08; W/E03 LOCA Cooldown - Depress. / 4						x	2.4.20 Knowledge of operational implcations of EOP warnings, cautions, and notes. (CFR 41.10/43.5/45.13)	4.3	1			
BW/E09; CE/A13; W/E09&E10 Natural Circ. / 4					X		E09 EA2 Ability to determine and interpret the following as they apply to the Natural Circulation Operations: EA2.2 Adherence to appropriate procedures and operation within the limitations in the facilities license and amendments. (CFR 43.5/45.13)	3.8	1			
BW/E13&E14 EOP Rules and Enclosures												
CE/A11; W/E08 RCS Overcooling - PTS / 4												
CE/A16 Excess RCS Leakage / 2												
CE/E09 Functional Recovery												
K/A Category Point Totals:					2	2	Group Point Total:		9/4			

4

Form ES-401-2

ES-401	-401 PWR Examination Outline Form ES-401-2 Plant Systems - Tier 2/Group 1 (RO / SRO)													
System # / Name	К 1	к 2	к 3	К 4	к 5	к 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
003 Reactor Coolant Pump														
004 Chemical and Volume Control											10 (1) X	2.2.22 Knowledge of limiting conditions for operations and safety limits. (CER 41 5/43 2/45 2)	4.7	1
005 Residual Heat Removal														
006 Emergency Core Cooling								1						
007 Pressurizer Relief/Quench Tank														
008 Component Cooling Water											.	2.4.11 Knowledge of abnormal condition procedures. (CFR 41.10/43.5/45.13)	4.2	1
010 Pressurizer Pressure Control														
012 Reactor Protection												A2 Ability to (a) predict the impacts of the following malfunctions or operations on the RPS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.06 Failure of RPS signal to trip the reactor. (CFR 41.5/43.5/45.3/45.5)	4.7	1
013 Engineered Safety Features Actuation											(10.90) 1972 1972			
022 Containment Cooling														
025 Ice Condenser											27 YA			
026 Containment Spray														
039 Main and Reheat Steam											and and a second se			
059 Main Feedwater														
061 Auxiliary/Emergency Feedwater								an a						
062 AC Electrical Distribution														
063 DC Electrical Distribution														

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ES-401				Pla	ant S	PV	/VR em:	Exar s - Ti	ninat er 2/0	tion Grou	Ou . qu	ıtline 1 (R	For O / SRO)	m ES-4	401-2
System # / Name	stem # / Name K K K 1 2 3									A 4		G	K/A Topic(s)	IR	#
064 Emergency Diesel Generator									8			X	2.2.40 Ability to apply Technical Specifications for a system.	4.7	1
													(CFR41.10/43.2/43.5/45.3)		
073 Process Radiation Monitoring									1.						
076 Service Water															
078 Instrument Air								Sec. 3							
103 Containment													A2 Ability to (a) predict the impacts of the following malfunctions or operations on the containment system; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.03 Phase A and B isolation (CFR 41.5/43.5/45.3/45.13)	3.8	1
				_	Ļ	<u> </u>	L	12.00		ļ					
								438500 HUREPH	N						
K/A Category Point Totals: 2 3 Group Point Total: 28/ 5															

5

ES-401	-401 PWR Examination Outline Form ES-401-2 Plant Systems - Tier 2/Group 2 (RO / SRO)													
System # / Name	к 1	к 2	к 3	К 4	K 5	К 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
001 Control Rod Drive											с., Я С., Я С., Я			
002 Reactor Coolant								X				A2 Ability to (a) predict the impacts of the following malfunctions or operations on the RCS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.02 Loss of coolant pressure (CFR 41.5/43.5/45.3/45.5)	4.4	1
011 Pressurizer Level Control														
014 Rod Position Indication														
015 Nuclear Instrumentation												A2 Ability to (a) predict the impacts of the following malfunctions or operations on the NIS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.02 Faulty or erratic operation of detectors or compensating components (CFR 41.5/43.5/45.3/45.5)	3.5	1
016 Non-nuclear Instrumentation								121 121 121 121 121 121 121 121 121 121						
017 In-core Temperature Monitor														
027 Containment Iodine Removal														
028 Hydrogen Recombiner and Purge Control														
029 Containment Purge														
033 Spent Fuel Pool Cooling												A2 Ability to (a) predict the impacts of the following malfunctions or operations on the Spent Fuel Pool Cooling System; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.03 Abnormal spent fuel pool water level or loss of water level (CFR 41.5/43.5/45.3/45.13)	3.5	1
034 Fuel Handling Equipment			339.9 								24 			
035 Steam Generator														

ES-401				Dutlin p 2 (l	ne Form ES- RO / SRO)	401-2							
System # / Name	к 1	К 2	к 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	#
041 Steam Dump/Turbine Bypass Control													
045 Main Turbine Generator											a .		
055 Condenser Air Removal													
056 Condensate											8880 - 10 17		
068 Liquid Radwaste													
071 Waste Gas Disposal													
072 Area Radiation Monitoring													
075 Circulating Water													
079 Station Air													
086 Fire Protection											8 1		
K/A Category Point Totals:								3			0	Group Point Total:	10/ 3

ES-401 Generic Knowledge and Abilities Outline (Tier 3)

Facility: SEABF	ROOK	Date of Exam: 6/15/09				
Category	K/A #	Торіс	R	0	SRO	-Only
			IR	#	IR	#
	2.1.20	Ability to interpret and execute procedure steps.			4.6	1
1.		(CFR 41.10/43.5/45.12)				
Conduct	2.1.41	Knowledge of refueling process.			3.7	1
of operations		(CFR 41.2/41.10/43.6/45.13)				
	2.1.					
	2.1.					
	2.1.					
	2.1.					
	Subtotal					2
	2.2.5	Knowledge of the process for making design or operating changes to the facility.			3.2	1
2.		(CFR 41.10/43.3/45.13)				
Equipment Control	2.2.38	Knowledge of conditions and limitations in the facility license.			4.5	1
		(CFR 41.7/41.10/43.1/45.13)				
	2.2.					
	2.2.					
	2.2.					
	2.2.					
	Subtotal	p	19			2
	2.3.4	Knowledge of radiation exposure limits under normal or emergency conditions.			3.7	1
3.		(CFR 41.12/43.4/45.10)				
Control	2.3.					
	2.3.		_			
	2.3.					
	2.3.					
	2.3.		_			
	Subtotal					1
4. Emergency	2.4.5	Knowledge of the organization of the operating procedure network for normal, abnormal, and emergency evolutions.			4.3	1
		(CFR 41.10/43.5/45.13)				

Facility: SEABROOK		Date of Exam: 6	6/15/09				
Category	K/A#	Торіс		RO		SRO-Only	
				IR	#	IR	#
Procedures / Plan	2.4.38	Ability to take actions called for in emergency plan, including suppo emergency coordinator if required (CFR 41.10/43.5/45.13)	the facility rt or acting as d.			4.4	1
	2.4.						
	2.4.						
	2.4.						
	2.4.						
	Subtotal						2
Tier 3 Point Total						81	7

Tier / Group	Randomly Selected K/A	Reason for Rejection
Tier 2/ Group 1	025 Ice Condenser	During exam outline generation the K/A's associated with Ice Condenser (025) were suppressed.
RO and SRO section		This system is N/A to Seabrook.
Tier 2/ Group 2	016 Non-nuclear Instrumentation	During exam outline generation the K/A's associated with Non-Nuclear Instrumentation (016) were suppressed.
RO and SRO section		This system is N/A to Seabrook
Tier 2/ Group 2	027 Containment Iodine Removal	During exam outline generation the K/A's associated with Containment Iodine Removal (027) were suppressed.
RO and SRO section		This system is included as part of the Containment Building Spray System at Seabrook
Tier 1/ Group 1 RO section	007 Reactor Trip- Stabilization- Recovery	Originally selected K/A EK1.05, Decay power as a function of time. Rejected K/A as this subject is considered a generic fundamental topic.
		Selected K/A EK1.06, Relationship of emergency feedwater flow to S/G and decay heat removal following reactor trip.
Tier 1/ Group 1	040 Steam Line Rupture-	Originally selected K/A AA1.15, T-avg protection indicators. Rejected K/A as this is not applicable to Seabrook. The facility utilizes core/vessel temperature parameters with
RO section	Transfer	regard to excessive cooldown conditions.
		PORV.
Tier 1/ Group 2 RO section	061 ARM System Alarms	Originally selected K/A AA1.01, Ability to operated and/or monitor the following as they apply to the ARM system alarms: Automatic actuation. Rejected this K/A as it presents too similar a topic to the K/A selected for Tier 1/ Group 2 036, Fuel Handling Accident. Seabrook Station does not have automatic actuations from area rad monitors with the exception of the refueling crane monitor that is placed in
		service during refueling. This topic is covered by the 036, Fuel Handling Accident associated K/A. Selected K/A AA2.05, Need for area evacuation.

Tier / Group	Randomly Selected K/A	Reason for Rejection
Tier 2/ Group 1 RO section	007 Pressurizer Relief/Quench Tank	Originally selected K/A A1.03, Monitoring quench tank temperature. Rejected this K/A as the subject matter appears to overlap that of Question 1 on the original 2009 written exam.
		Selected K/A 3.01, Knowledge of the effect that a loss or malfunction of the PRTS will have on the following: Containment
Tier 2/ Group 1 RO section	022 Containment Cooling	Originally selected K/A A2.05, Major leak in CCS. Rejected K/A as Seabrook Station does not have a Service Water interface for it's CCS. Any major leak associated with the CCS system would be associated with the Component Cooling System, which is covered by another category.
		Selected new K/A K2.01, Knowledge of power supplies for the following: Containment Cooling Fans
Tier 2/ Group 1 RO section	063 DC Electrical System	Originally selected K/A K1.03, Knowledge of the physical connections and/or cause-effect relationships between the DC electrical system and the following: Battery charger and battery. Rejected the K/A as it appeared too close to the subject matter for the K/A selected for Tier 1/ Group 1, 058, Loss of DC power.
		Selected new K/A K1.02, Knowledge of the physical connections and/or cause-effect relationships between the DC electrical system and the following: AC electrical system
Tier 1/ Group 2 SRO section	059 Accidental Liquid Rad Waste Release	Originally selected K/A 2.3.7, Ability to comply with radiation work permits during normal or abnormal conditions. K/A was rejected as the topic pertains to accidental rad releases vice a planned release associated with rad permits.
		Selected new K/A 2.3.1, Ability to control radiation releases.
Tier 2/ Group 1 SRO section	063 DC Electrical System	Originally selected category 063 DC Electrical System. Rejected K/A category 063 as the same category appears multiple times on the exam. Change made pursuant to achieving diversity of subject matter.
		Selected new K/A category 064 Emergency Diesel Generator

Seabrook Station 2009 NRC Remediation Written Exam Record of Rejected K/As

Form	ES-40	1-4

Tier / Group	Randomly Selected K/A	Reason for Rejection
Tier 2/ Group 2 SRO section	002 Reactor Coolant	Originally selected K/A A2.04, Ability to a) predict the impacts of the following malfunctions or operations on the RCS; and b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Loss of heat sinks. Rejected the K/A as the subject matter was too similar to the K/A selected for RO section, Tier 1/Group1, BW/E04, Inadequate Heat Transfer-Loss of Secondary Heat Sink, K/A EA2.1. Selected new K/A A2.02, Loss of coolant pressure.
Tier 2/Group 2	017 In-Core Temperature Monitoring	Originally selected K/A K1.02, Knowledge of the physical connections and/or cause effect relationships between ITM and the following systems: RCS
		with submitted test question #2.
		Selected new K/A K4.01, Knowledge if the ITM system design features and/or interlocks which provide for the following: Input to subcooling monitors.
Tier 2/ Group 1	076 Service Water	Originally selected K/A K3.07 Knowledge of the effect that a loss or malfunction of the SWS will have on the following: ESF Loads.
		Rejected the K/A as the main ESF load is the Emergency Diesel Generator. The exam already has questions pertaining to the Emergency Diesel Generator and another discriminatory question could not be found.
		Selected new K/A K3.01, Knowledge of the effect that a loss or malfunction of the SWS will have on the following: Closed cooling water.

Seabrook Station 2009 NRC Remediation Written Exam Record of Rejected K/As

FORM E3-401-4	Form	ES-401-4
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Tier / Group	Randomly Selected K/A	Reason for Rejection
Tier 1/ Group 2	061 Area Rad Monitoring System Alarms	Originally selected K/A AA2.05 Ability to determine and interpret the following as they apply to the Area Radiation Monitoring (ARM) System alarms: Need for area evacuation, check against existing limits.
		Rejected the K/A based on a) could not select/create a discriminatory question. The facilities actions in the Area High Radiation abnormal procedure appeared too simplistic to author a valid question and b) the exam already contains questions pertaining to Area Rad Monitor interface with Containment Ventilation Isolation equipment and also Refueling/Spent Fuel abnormal procedures.
		Reselected new Category and K/A from same tier. Selected W/E16 High Containment Radiation, K/A EA1.1 Ability to operate and/or monitor the following as they apply to the High Containment Radiation, Components and functions of control and safety systems including instrumentation, signals, interlocks, failure modes, and automatic and manual features.