UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

November 25, 2009

Mr. David J. Bannister
Vice President and CNO
Omaha Public Power District
Fort Calhoun Station
444 South 16th St. Mall
Omaha, NE 68102-2247

SUBJECT: FORT CALHOUN STATION, UNIT 1 – REVIEW OF THE 2008 REFUELING OUTAGE STEAM GENERATOR TUBE INSERVICE INSPECTION REPORT (TAC NO. ME0313)

Dear Mr. Bannister:

By letter dated December 9, 2008 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML083440629), and supplemented by letter dated October 16, 2009 (ADAMS Accession No. ML093000157), Omaha Public Power Distribution (the licensee), submitted information summarizing the results of the 2008 steam generator tube inspections performed at Fort Calhoun Station, Unit 1 during the 2008 refueling outage.

The Nuclear Regulatory Commission (NRC) staff has completed its review of the report and concludes that the licensee provided the information required by its technical specifications and no additional follow-up is required at this time. The NRC staff's review of the report is enclosed.

Sincerely,

Balwant K Singal for

Lynnea Wilkins, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure:
As stated

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REVIEW OF STEAM GENERATOR TUBE INSPECTION

REPORT FOR THE 2008 REFUELING OUTAGE

FORT CALHOUN STATION UNIT 1

DOCKET NO. 50-285

By letter dated December 9, 2008 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML083440629), and supplemented by letter dated October 16, 2009 (ADAMS Accession No. ML093000157), Omaha Public Power Distribution (the licensee), submitted information summarizing the results of the 2008 steam generator (SG) tube inspections performed at Fort Calhoun Station, Unit 1 (FCS) during the 2008 refueling outage.

FCS has two Mitsubishi Heavy Industries re-circulating model MHI-49TT-1 SGs. Each SG contains 5200 thermally-treated Alloy 690 tubes. Each tube has a nominal outside diameter of ¾ inches with a nominal wall thickness of 0.043 inches. The tubes were hydraulically expanded at both ends for the full length of the tubesheet and are supported by a series of type 405 stainless steel structures.

The licensee provided the scope, extent, methods, and results of its SG tube inspections in the documents referenced above. In addition, the licensee described corrective actions, such as tube plugging, taken in response to the inspection findings. The tubes in both SGs were inspected during the 2008 refueling outage.

After reviewing the information provided by the licensee, the Nuclear Regulatory Commission (NRC) staff has the following comments or observations:

- The FCS SGs were replaced during its 2006 refueling outage. The first inservice inspection was performed in 2008.

- Two new dent indications were identified in two different tubes in SG RC-2A and one new dent was identified in SG RC-2B with voltages of approximately 1 volt. The licensee reported that the cause of the dents is indeterminable due to the small magnitude of the indications.

- No wear was observed.

Based on a review of the information provided, the NRC staff concludes that the licensee provided the information required by the technical specifications. In addition, the NRC staff concludes that there are no technical issues that warrant follow-up action at this time since the inspections appear to be consistent with the objective of detecting potential tube degradation and the inspection results appear to be consistent with industry operating experience at similarly designed and operated units.

Enclosure

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/RA by Balwant K. Singal for/

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