# Crystal River Unit #3 Containment Delamination Update

November 20th 2009





# Agenda

- Introduction
- Plant Overview
- CR3 Containment Design Features
- SGR Opening Sequence & Identification of Delamination
- Investigative Approach
- Condition Assessment
- Root Cause Analysis (RCA)
- Operational Experience (OE)
- Design Basis Analysis (DBA)
- Repair Approach
- Summary Comments / Questions





# **Crystal River 3 Overview**

- Babcock and Wilcox Pressurized Water Reactor
- Location: Crystal River Florida
- 2609 MW<sub>th</sub>
- 838 MW<sub>e</sub>
- Commercial Operations began 1976







#### **2009 Crystal River 3 Outage Overview** Building a nuclear future for Florida customers

- Routine refueling scope
  - Off line maintenance and fuel for 2 years
- Steam Generator Replacement (SGR)
- Extended Power Uprate (EPU) Phase 2
  - Extensive steam plant work
  - Taking advantage of longer OTSGR duration
  - Steam plant efficiencies
  - Part of total ~15% Uprate





#### **Steam Generator Replacement (SGR)** Work Breakdown

- **Containment Opening**
- Lifting and Rigging
- Cutting and welding







#### Extended Power Uprate (EPU) Work Breakdown

- Generator Replacement
  - Stator, Rotor, Exciter
- Moisture Separators
- MSR Drain Coolers
- Lube Oil Coolers
- Feed Water Heaters
- Iso-Phase cooling







# **CRYSTAL RIVER #3 DESIGN FEATURES**







#### Fission Product Barriers Simplified Schematic



# CR3 Containment Dimensions

Dimension	Value
Containment Outside Dimension (OD)	137 ft 0.75 in
Dome Thickness	36 in
Basemat Thickness	12 ft 6 in
Liner Thickness	0.375 in
Wall Thickness	42 in
Buttress Wall Thickness	5 ft 10 in
Vertical & Hoop Conduit OD	5.25 in
# of Vertical Tendons	144
# of Tendon Hoops	94
# of Tendons per Hoop	3
# of Prestressed Dome Tendons	123





# SGR OPENING SEQUENCE & IDENTIFICATION OF DELAMINATION







# Steam Generator Replacement (SGR) Opening (between Buttresses 3 and 4)



#### SGR Opening Dimensions

@ Liner 23' 6" x 24' 9"

@ Concrete Opening 25' 0" x 27" 0"





#### **Concrete Removal**







# **Concrete & Liner Removal Sequence**







# **Delamination Close-up**







#### **Location of the Delamination**



# **INVESTIGATION APPROACH**









# **External Support**

- Condition Assessment & Laboratory Testing
  - NDT Construction Technology Laboratories (CTL)
  - Labs MacTec, Soil& Materials Engineers (S&ME)
  - Other Field Data Sensing Systems, Inc; Core Visual Inspection Services (Core VIS), Nuclear Inspection & Consulting, Inc; Precision Surveillance; Gulf West Surveying Inc; AREVA
- Root Cause Analysis
  - Lead Performance Improvement International (PII)
  - Owner's Support Worley Parsons, Bechtel





# **External Support (continued)**

#### Design Basis Analysis

- Lead MPR Associates, Inc.
- Owner's Support Worley Parsons

### Repair Analysis

- Lead Structural Preservation Systems (SPS)
- Owner's Support Wiss, Janney, Elstner, Inc (WJE)

## Industry Support

• Exelon, SCANA, and Southern Company





#### **Organization – Functional View**



#### Nuclear Safety Oversight Committee (NSOC) Containment Sub-Committee Membership

Member	Title
Bob Bazemore (PGN)	VP-Audit (Chairman)
Joe Donahue (PGN)	VP- Nuclear Oversight
Chris Burton (PGN)	VP – Harris
Greg Selby	Technical Director - EPRI
Dr. Shawn Hughes	VP - Shaw Stone and Webster
Dr. Paul Zia	Civil Engineering Professor, NCSU
Hub Miller	33 years industry oversight experience
Darrell Eisenhut	41 years industry operation and oversight experience





# **CONDITION ASSESSMENT**







#### Condition Assessment Activities Completed or Planned

## Determine Extent of Condition

- Characterize the extent of delamination at the SGR opening
- Determine condition of other portions of structure

# Non Destructive Testing (NDT) of Containment Wall Surfaces

- Use of Impulse Response (IR) Method
- Comprehensive on external exposed surfaces
- Accessible areas in adjacent buildings





# **Condition Assessment Activities**

#### **Completed or Planned**

#### Concrete Cores

- Used to confirm IR results (over 80 cores)
- Visual examination of core bore holes with boroscope to identify if delamination present
- ASME Section XI IWL visual inspection (affected areas)

## Containment Dome Inspections

- NDT IR scans in segment above the SGR opening
- Concrete cores with boroscope examination of bore holes
- Physical survey with established benchmarks





#### Condition Assessment Techniques Impulse Response (IR)





- IR Equipment
  - Primary test method used in this evaluation

• IR Performed in the Field





#### **Condition Assessment Techniques** *Ground Penetrating Radar (GPR)*



- Ground Penetrating Radar (GPR) Equipment
  - Locates internal features (rebar, tendon conduits, etc.)
- GPR Performed in the Field





#### **Condition Assessment Techniques** *Core Bores & Boroscopic Examination*



#### Examination – Inward View



#### Examination – Side View

Core 51, Gap 1 Depth 5-1/4" Gap 1 Width Less than 1/8"





#### Condition Assessment Techniques Impact Echo (IE)





- IE Equipment
  - Ability to determine depth of delamination
- IE Performed in the Field









# **Containment "Unfolded" – Buttress 5 to 2** Updated Nov 18<sup>th</sup> 2009





#### Core Bores Buttress Spans 2 - 3 - 4 - 5 (as of Nov 17<sup>th</sup> 2009)



#### Core Bores Buttress Spans 5 - 6 - 1 - 2 (as of Nov 17<sup>th</sup> 2009)








### Vertical Tendons



Additional tendons to be detensioned prior to closing SGR opening (preoutage plan)



### CR3 Typical Tendon Schematic and Photo (for horizontal tendon # 53H27)









### Tendon Pattern

### Tendon Pattern at time of cutting SGR Opening



Removed Tendon





### Tendon Pattern

### Tendon Pattern at time of cutting SGR Opening



Removed Tendon











### Equipment Hatch Opening Reinforcement Photo - 30 Nov 1972



# **ROOT CAUSE ANALYSIS**





## **Root Cause Analysis – PII Metrics**

Un-refuted Failure Modes as of Nov 17th 2009



### Root Cause Analysis Field Data Acquisition

- Impulse Response (IR) Scans
- Boroscopic Inspections
  - Core bore holes
  - Inside the delaminated gap
- Visual inspections
  - Delamination cracks at SGR Opening
  - Larger fragments from concrete removal process
  - Containment external surface





### Root Cause Analysis Field Data Acquisition (continued)

- Nearby energized tendons lift-off (vertical and horizontal)
- Containment dimension measurements
- Strain gauge measurements
- Linear variable displacement transducer (LVDT) gap monitoring
- Building natural frequency





### Root Cause Analysis Field Data Acquisition (continued)

- Core bores laboratory analysis
  - Petrographic Examination
  - Modulus of Elasticity and Poisson's Ratio
  - Density, Absorption, and Voids
  - Compressive Strength, Splitting Tensile Strength, and Direct Tensile Strength
  - Accelerated Creep test
  - Accelerated Alkali Silica Reaction (ASR) test
  - Chemistry and contamination test
  - Scanning Electron Microscope (SEM) examination of microcracking





# **OPERATIONAL EXPERIENCE (OE)**





### Steam Generator Replacement (SGR) OE Type of Information Collected from the Industry

- Architect Engineer and Constructor
- Type of Containment and design pressure
- # of Buttresses
- Concrete design strength requirement
- Dimensions
  - Internal containment diameter and wall height
  - Containment cylinder wall and dome thickness
  - Tendons details (# vertical, # horizontal, # dome, strand diameter)
  - Liner thickness





### Steam Generator Replacement (SGR) OE Type of Information Collected from the Industry (cont)

- Reinforcement details
- Whether concrete opening was made
  - Was hydro-excavation used
  - And if so, equipment operating parameters
- Detensioning details
  - # by cutting
  - # by relaxation
  - # of tendons removed/detensioned beyond the SGR opening





### **Concrete OE**

### **Worley Parsons**

1976 dome delamination investigation and repair (as Gilbert / Commonwealth)

### Structural Preservation Systems (SPS)

- Largest Concrete Repair Contractor in the US, 2<sup>nd</sup> largest Concrete Contractor (of any type) in the US
  - Defects, Damage, and Deterioration
- Performs > 4,000 repair projects per year
- 3,000 employees in 27 offices Nationwide, and London, Dubai & Singapore

### Wiss, Janney, Elstner, Inc (WJE)

- Structural engineering and materials science firm specializing in failure investigations and problem solving
- Specialist in structural condition assessments and design of repairs and retro-fits for reinforced and post tension concrete structures
- Conducted original CR3 Structural Integrity Test (SIT)
  - 450 employees in 20 offices nationwide





# 1976 Dome Delamination *Cause*<sup>(1)</sup>

- Compression tension interaction failure occurred
- Contributing Effects
  - Radial tension due to prestressing
  - Thermal effects
  - Tendon alignment
  - Stress concentrations
  - Shrinkage
- Combined with biaxial compressive stresses and lower than normal<sup>(2)</sup> direct tensile strength of concrete





 <sup>(1)</sup>Cause information taken from 1976 Final Report prepared by Gilbert / Commonwealth
<sup>(2)</sup>Lower than normal (or typical), but above design requirements

### 1976 Dome Delamination *Repair Approach*

- Tendons detensioned (18)
- Delaminated surface was removed
- Lower level cracks grouted with epoxy
- New reinforcement placed
- New cap poured and cured set
- Tendons partially retensioned (18)







# **DESIGN BASIS ANALYSIS**







### **Design Basis**

- Reinforced Post-Tensioned Concrete Structure
- Live and Dead Loads
- Wind (110mph @ 30' increasing to 179 mph @ 166'10")
- Tornado Wind (300 mph)
- Tornado pressure (external pressure of 3 psig)
- Tornado Missiles (35' utility pole or 1 ton car @ 150 mph)
- Seismic (OBE 0.05 and SSE 0.10)
- Temperature Loads
- Accident Pressure (55 psig)
- Accidental Containment Spray Actuation Press (- 6.0 psig)





### **CR3 FEA Model**



### 180 degree Symmetric model

- Symmetry plane @ 150 degrees midway Between Buttress 3 & 4 / 1 & 6
- ½ Opening, ½ Damage & ½ Hatch Modeled Explicitly

### Concrete Model

- Brick elements for all components
- Dome and Base modeled independently
- Simplified ring beam and buttress geometry
- Constraint equations used to join dome and ring girder for meshing efficiency
- Constraint equation used to model sloped surfaces of the hatch





### **CR3 FEA Model (continued)**

### Liner Model

- Shell mesh with variable thickness
- Shared nodes with containment inner surface

### Tendon Modeling

- Hoop tendons modeled explicitly for release and retensioning
- Vertical Tendons modeled explicitly for release and retensioning
- Dome tendons modeled independently with forces ported to global model





ELEMENTS MAT NUM

### **Concrete Geometry Based on Gilbert Associates Drawings**



NGG

60



### **Tendon Geometry Based on Prescon Drawings**



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### **Dome FEA Model**



# **Ring Girder Model**







### Core Building Geometry – FEA Mesh Hoop Tendon Locations Defined





### **Core Building Geometry - Buttresses**







### **Equipment Hatch Model**









### Liner



### **Tendon Loading**

- The tendons are preloaded to a prescribed load magnitude.
- The application of the tendon loads is achieved in the analysis using initial strain input
- An empirical formula has been developed to account for the loss of load as the distance from the anchor point increases:

$$\mathbf{P} = \mathbf{P}_0 \, \mathrm{e}^{-(\mathrm{ma} + \mathrm{ks})}$$

- Where:
  - P<sub>o</sub> = preload magnitude
  - m = friction coefficient
  - a = inflection angle (0.16)
  - k = wobble coefficient (0.0003)
  - s = distance from anchor point
- Tendon preloads used in analysis:
  - P<sub>0-dome</sub>
- = 1635 Kips (1,215,000 lb. 40 years) = 1635 Kips (1,252,000 lb. 40 years)
- P<sub>0-horizontal</sub> P<sub>0-vertical</sub>
- = 1635 Kips (1,252,000 lb. 40 years) = 1635 Kips (1,149,000 lb. 40 years)





### **Dome Force Vectors Ported to Global Model**



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### **FEA Model – Vertical and Hoop Tendons**

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### **FEA Model – Vertical and Hoop Tendon Supports**





### **FEA Model – Hoop Tendons Couples and Supports**



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#### **Hoop Tendon Forces**



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#### **Planned Analysis**

#### Existing Design Cases for Comparison

- Gravity (.95 G)
- Internal Dead Load (200 psf)
- Tendons (1635 kips / tendon)
  - Include losses
- Internal Pressure (55.0 psi)
- Wind Pressure (0.568 psi)
- Seismic
- Accident Thermal

#### Planned Analysis Steps

- Dead Load + Tendons
- Remove Hoop + Vertical Tendons in SGR Opening
- Remove SGR Opening
- Delamination<sup>(1)</sup>
- Remove Additional Hoop & Vertical Tendons
- Replace the SGR Plug<sup>(2)</sup>
- Repair<sup>(2)</sup>
- Re-tension Tendons
- SAVE Path Dependent Model for Starting point to Run 5 Controlling Design cases



 <sup>(1)</sup> Analysis will consider timing of delamination and specific concrete properties
<sup>(2)</sup> Sequence of replacing SGR concrete plug and repair may be adjusted



#### **Design Basis Controlling Load Steps**

- Restart the Re-tensioned Model and solve the following Controlling Load Steps
  - 1.5 Internal Pressure + Accident Thermal
  - 1.25 Wind + 1.25 Pressure + Accident Thermal
  - 1.25 Earthquake + 1.25 Pressure + Accident Thermal
  - 2.0 Wind + Pressure + Accident Thermal
  - SSE Earthquake + Pressure + Accident Thermal
- Run Comparison to original building elastic design results





#### Preliminary Comparison of FEA Results to Extent of Condition Measurements



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## **REPAIR APPROACH**





#### **Repair Attributes**

- Incorporates and is compatible with Root Cause Analysis findings
- Restores applicable design basis margins
- Incorporates Extended Life
  - Long Term Surveillance and/or Maintenance Requirements
  - License Renewal
- Constructability





### **Repair Alternatives Considered**

- Use-as-ls Rejected
- Anchorage Only Rejected
- **Cementitious Grout** *Rejected*
- Epoxy Resin Rejected
- Delamination Removal and Replacement Selected





#### Simplified Overview of Engineering & Repair Work Flow Tentative – Subject to RCA and DBA Results







#### **Post- Repair Testing** *Tentative – Subject to RCA Results*

- Approach ILRT and System Pressure Test
- ASME Section XI IWE for the liner and IWL for the concrete
- Concrete exterior will be visually examined prior to pressurization and following de-pressurization
- Evaluating other additional instrumentation based on the final repair that is implemented, and as driven by:
  - Root cause analysis
- NDE will be required for restored liner plate





### **Stakeholder Interactions**

- Prompt Notification of Regulator & Industry
- Engagement of Critical Industry Organizations
  - Nuclear Energy Institute (NEI)
    - Including Nuclear Safety Information Advisory Council (NSIAC)
  - Institute for Nuclear Power Operations (INPO)
  - Electric Power Research Institute (EPRI)
- Continued Transparency with Regulator
  - Special Inspection Team (SIT)
  - Region and NRR/RES technical discussions
- Periodic Updates with U.S. Licensees





#### Summary & Questions

# Questions



