



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

November 12, 2009

Vice President, Operations
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
450 Broadway, GSB
P.O. Box 249
Buchanan, NY 10511-0249

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 - REQUEST FOR
ADDITIONAL INFORMATION REGARDING RELIEF REQUEST RR-03
(TAC NO. ME1577)

Dear Sir or Madam:

On June 24, 2009, Agencywide Documents Access and Management System Accession No. ML091820325, Entergy Nuclear Operations, Inc. (Entergy), submitted Relief Request (RR)-03 for Indian Point Nuclear Generating Unit No. 3 to use the alternative qualification requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Code Case N-739-1, in lieu of the requirements in the ASME Code, Section IWA-2300.

The Nuclear Regulatory Commission staff is reviewing the submittal and has determined that additional information is needed to complete its review. The specific question is found in the enclosed request for additional information (RAI). On November 9, 2009, the Entergy staff indicated that a response to the RAI would be provided within 30 days of the date of this letter.

Please contact me at (301) 415-2901 if you have any questions on this issue.

Sincerely,

A handwritten signature in black ink that reads "John P. Boska".

John P. Boska, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure:
RAI

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION

REGARDING RELIEF REQUEST RR-03

ENERGY NUCLEAR OPERATIONS, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

DOCKET NO. 50-286

On June 24, 2009, Agencywide Documents Access and Management System Accession No. ML091820325, Entergy Nuclear Operations, Inc. (Entergy), submitted Relief Request (RR)-03 for Indian Point Nuclear Generating Unit No. 3 (IP3) to use the alternative qualification requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Code Case N-739-1, in lieu of the requirements in the ASME Code, Section IWA-2300. The Nuclear Regulatory Commission staff is reviewing the submittal and has the following question:

1. Based on the IP3 Updated Final safety Analysis Report Section 5.1.2.1, the reactor containment structure is a reinforced concrete vertical right cylinder with a flat base and hemispherical dome. No post-tensioning steel tendons are used in its design. RR-03, Section 1, states that the following ASME Code components are affected by the proposed alternative:
 - L1.11
 - L1.12
 - L2.30

However, Item L2.30 is included in the ASME Code, Section XI, 2001 Edition with 2003 Addenda, Division I, Table IWL-2500-1 as Examination Category L-B for the "Unbonded Post-Tensioning System." Specifically, Item L2.30 refers to anchorage hardware and surrounding concrete. Please clarify why Item L2.30 is included as an affected component under the ASME Code in the IP3 relief request, considering that IP3 does not have a post-tensioning system.

Enclosure

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ADAMS ACCESSION NO.: ML093140012 *See memo dated 11/6/09

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