

16-5, KONAN 2-CHOME, MINATO-KU TOKYO, JAPAN

October 28, 2009

Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555-0001

Attention: Mr. Jeffrey A. Ciocco

Docket No. 52-021 MHI Ref: UAP-HF-09499

Subject:

Transmittal of the Updated Chapter 14 of US-APWR DCD

Reference:

- "Request for Additional Information No. 3593 (CP RAI#86), SRP Section: 14.02 - Initial Plant Test Program - Design Certification and New License Applicants" dated September 26, 2009.
- Letter MHI Ref: UAP-HF-09490 from Y. Ogata (MHI) to U.S. NRC, "Submittal of US-APWR Design Control Document Revision 2 in Support of Mitsubishi Heavy Industries, Ltd.'s Application for Design Certification of the US-APWR Standard Plant Design" dated on October 27, 2009.

MHI and Luminant have been working to resolve a COLA Request for Additional Information (RAI). CPNPP-3 and 4, COL RAI #86, question 14.02-10 (Reference 1) was issued to request providing the description of the preoperational leak testing specified in Chapter 16, Subsection 5.5.2. In this activity, MHI concluded that the test description would be included in the Design Control Document because this could be applied to the standard plant.

With this letter, MHI transmits to the NRC Staff the proposed marked-up to be made to the DCD revision 2 (Reference 2). This update will be incorporated into future DCD revision.

Please contact Dr. C. Keith Paulson, Senior Technical Manager, Mitsubishi Nuclear Energy Systems, Inc. if NRC has questions concerning any aspect of this letter. His contact information is provided below.

Sincerely.

Yoshiki Ogata,

General Manager- APWR Promoting Department Mitsubishi Heavy Industries, LTD.

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Enclosure:

1. Update of Chapter 14 of US-APWR DCD

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CC: J. A. Ciocco C. K. Paulson

Contact Information

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Docket No.52-021 MHI Ref. UAP-HF-09499

Enclosure 1

UAP-HF-09499 Docket No. 52-021

Update of Chapter 14 of US-APWR DCD

October 2009

Subsection 14.2.12.1.1 Marked-up

	14.2.12.1.14	CVCS Preoperational Test - Letdown
	14.2.12.1.15	RCS Lithium Addition and Distribution Test
	14.2.12.1.19	Resistance Temperature Detectors (RTDs)/Thermocouple Cross-Calibration Preoperational Test
	14.2.12.1.21	Main Steam Supply System Preoperational Test
	14.2.12.1.22	Residual Heat Removal System (RHRS) Preoperational Test
	14.2.12.1.23	Main Steam Isolation Valve (MSIV), Main Feedwater Isolation Valve (MFIV) and Main Steam Check Valve Preoperational Test
	14.2.12.1.25	Turbine-Driven Emergency Feedwater System Preoperational Test
	14.2.12.1.50	Dynamic State Vibration Monitoring of Safety Related and High- Energy Piping
	14.2.12.1.51	Steady State Vibration Monitoring of Safety Related and High- Energy Piping
	14.2.12.1.52	Thermal Expansion Test
	14.2.12.1.54	Safety Injection System (SIS) Preoperational Test
	14.2.12.1.56	Safety Injection Check Valve Preoperational Test
	14.2.12.1.66	Reactor Cavity Cooling System Preoperational Test
	14.2.12.1.69	Containment Fan Cooler System Preoperational Test
	14.2.12.1.71	RCS Leak Rate Preoperational Test
	14.2.12.1.72	Loose Parts Monitoring System Preoperational Test
	14.2.12.1.76	Remote Shutdown Preoperational Test
1	14.2.12.1.83	Steam Generator Blowdown System Preoperational Test
	14.2.12.1.84	Sampling System Preoperational Test
	14.2.12.1.87	Component Cooling Water System Preoperational Test
	14.2.12.1.107	Pressurizer Heater and Spray Capability and Continuous Spray Flow Verification Test

Add item C.7 in Subsection 14.2.12.1.1. (CP RAI#86, 14.02-10)

Flow Verification Test

The Jackeds central program plant procedures which implement Technical

7. The leakage control program plant procedures which implement Technical Specifications program 5.5.2, Primary Coolant Sources Outside Containment, are performed while the plant is in hot standby.