

Tennessee Valley Authority, Post Office Box 2000, Decatur, Alabama 35609-2000 October 27, 2009

10 CFR 50.73

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D. C. 20555-0001

> Browns Ferry Nuclear Plant Unit 2 Facility Operating License No. DPR-52 NRC Docket No. 50-260

Subject: Licensee Event Report (LER) 50-260/2009-006

The enclosed Licensee Event Report (LER) provides details of an automatic Reactor Protection System scram while shutdown. At the time of the event Operations personnel believed the event to not be reportable due to the understanding of guidance provided in NUREG-1022, Event Reporting Guidelines 10 CFR 50.72 and 50.73. Station management reviewed the event and initially agreed with operations personnel that the event was not reportable. Subsequent to further review, TVA concluded that this event should have been reported. This resulted in the LER being submitted late. TVA is reporting this in accordance with 10 CFR 50.73(a)(2)(iv)(A), as an event that resulted in a manual or automatic actuation of the systems listed in paragraph 10 CFR 50.73(a)(2)(iv)(B) (i.e, Reactor Protection System including reactor scram or trip).

There are no new regulatory commitments contained in this letter. Should you have any questions concerning this submittal, please contact F. R. Godwin, Site Licensing and Industry Affairs Manager, at (256) 729-2636.

Respectfully,

R. G. West Vice President

cc: See page 2

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Enclosure cc (Enclosure):

NRC Regional Administrator - Region II

NRC Senior Resident Inspector - Browns Ferry Nuclear Plant

U.S. NUCLEAR REGUL	ATORY COMMISSION	PPROVED BY OMB NO. 3150-01	04	E	XPIRES 08	/31/2010
SEE EVENT REPORT	(LER)	Estimated burden per response to 80 hours. Reported lessons learned a back to industry. Send comments FOIA/Privacy Service Branch (T-5 Washington, DC 20555-0001, or by in Desk Officer, Office of Information an Office of Management and Budget, Wa an information collection does not disp may not conduct or sponsor, and a pe collection.	comply with regarding bu F52), U.S. Internet e-mai ad Regulatory ashington, D blay a current broon is not re	this manda ted into the li urden estima Nuclear R y Affairs, NE C 20503. If y valid OMB equired to res	atory collect censing produce the to the F legulatory C COB-10202, a means use control numi spond to, the	ion request: cess and fed Records and Commission, <i>y</i> , and to the (3150-0104), ed to impose ber, the NRC e information
2	2	DOCKET NUMBER 05000260	3. P	AGE 1	of 4	
c Reactor Protection	System Scram W	nile Shutdown				
6. LER NUMBER	7. REPORT DATE	8. OTHER F	ACILITIE	S INVOLV	'ED	
YEAR SEQUENTIAL REV NUMBER NO.	MONTH DAY YEAR	FACILITY NAME None			DOCKET NU	JMBER /A
2009 - 006 - 00	10 27 2009	FACILITY NAME None			DOCKET NU N	JMBER
11. THIS REPORT IS \$ 20.2201(b) 20.2201(d) 20.2203(a)(1) 20.2203(a)(2)(i)	CUBMITTED PURSUANT 20.2203(a)(3)(i) 20.2203(a)(3)(ii) 20.2203(a)(3)(ii) 20.2203(a)(4) 50.36(c)(1)(i)(A)	TO THE REQUIREMENTS OF 50.73(a)(2)(i)(C) 50.73(a)(2)(ii)(A) 50.73(a)(2)(ii)(B) 50.73(a)(2)(iii)	10 CFR §	(Check a 50.7 50.7 50.7 50.7 50.7	all that ap) 3(a)(2)(vii 3(a)(2)(vii 3(a)(2)(vii 3(a)(2)(ix)	ply) i) i)(A) i)(B))(A)
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Licensing Engineer			TELEPHONE	NUMBER (1)	nclude Area 9-7533	Code)
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FACILITY NAME (1)	DOCKET (2)		LER NUMBER (6		PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Browns Ferry Nuclear Plant Unit 2	05000260	2009	006	00	2 of 4

NARRATIVE

I. PLANT CONDITION(S)

Prior to the event, Unit 2 was shutdown in Mode 5 and in a refueling outage and Units 1 and 3 were in operating in Mode 1 at 100 percent thermal power (approximately 3458 megawatts thermal). Units 1 and 3 were unaffected by the event.

II. DESCRIPTION OF EVENT

A. Event:

On May 24, 2009, with Unit 2 in a refueling outage and all rods fully inserted, at approximately 0232 hours Central Daylight Time (CDT), Operations personnel inserted a "B" channel half scram to support a maintenance work activity on the Reactor Protection System Contactor Relay. At approximately 0247 hours CDT, Operations received an "A" channel half scram and a Reactor Protection System (RPS) [JC] actuation (full scram). A previous clearance was placed to support work on hydrolazing the Scram Discharge Volume header which had a caution order placed on the SDV hi level bypass switch to keep the switch in bypass with a warning that unbypassing would result in a full reactor scram. Restoration of the clearance supporting the SDV hydrolazing only removed the caution order from the by pass switch and did not reposition the switch. When the fuse was removed from the circuit that supplied power to the contact that bypassed the high level scram signal, a reactor scram signal was generated, which initiated an RPS actuation (full scram).

Operations reset the reactor scram per procedure 2-OI-99, Reactor Protection System, Section 6.1 by 0253 hours CDT.

Submission of this LER is late because at the time of the event Operations personnel believed the event to not be reportable due to the understanding of guidance provided in NUREG-1022, Event Reporting Guidelines 10 CFR 50.72 and 50.73.

TVA is submitting this report in accordance with 10 CFR 50.73(a)(2)(iv)(A). An event that resulted in a manual or automatic actuation of the systems listed in paragraph 10 CFR 50.73(a)(2)(iv)(B) (i.e., reactor protection system including reactor scram or trip).

B. Inoperable Structures, Components, or Systems that Contributed to the Event:

None.

C. Dates and Approximate Times of Major Occurrences:

May 24, 2009	0232 hours CDT	Operations personnel inserted a Unit 2 "B" channel half scram.
May 24, 2009	0247 hours CDT	Unit 2 reactor automatically scrammed.
May 24, 2009	0253 hours CDT	Operations personnel reset Unit 2 scram.

D. Other Systems or Secondary Functions Affected

None.

E. <u>Method of Discovery</u>

NRC FORM 366A (9-2007)	U.S. NUCLEAR REGULATORY COMMISSION				
LICENSI	EE EVENT F	REPORT	(LER)		
FACILITY NAME (1)	DOCKET (2)		LER NUMBER (6	5)	PAGE (3)
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Browns Ferry Nuclear Plant Unit 2	05000260	2009	006	00	3 of 4

NARRATIVE

The operations crew received main control room annunciators Reactor Channel "A" scram and Reactor Channel "B" scram.

F. Operator Actions

Operators responded in accordance with the alarm response procedures. Since the reactor
 was already in Mode 5 with all control rods inserted, no plant parameters were changed.

G. Safety System Responses

The RPS logic responded to the reactor scram. Since the reactor was already in Mode 5 with all control rods inserted, no reactor parameters were changed.

III. CAUSE OF THE EVENT

A. Immediate Cause

The immediate cause for the event was due to pulling a fuse, associated with bypassing an SDV high level scram signal, in combination with a half scram that had been inserted earlier in support of work on the Reactor Protection System Contactor Relay.

B. Root Cause

The root cause of this event was lack of awareness of the SDV system configuration. The control room operators were not cognizant of the fact that the SDV was isolated and full of water in combination with the associated SDV high level scram signal being bypassed.

C. Contributing Factors

Contributing causes were ineffective communication and ineffective pre-job brief.

IV. ANALYSIS OF THE EVENT

Prior to the event, an outage activity to hydrolaze the SDV Instrument Tank and Headers had been completed. The associated clearance had a caution order on the SDV High Level Bypass switch to keep the switch in "BYPASS" with a warning that taking the switch out of "BYPASS" would result in an RPS actuation (full scram). The hydrolazing was completed several days before this event, but the restoration of the clearance for that activity was only partially completed and had only removed the caution order from the "BYPASS" switch and did not return the switch to its "NORMAL" (un-bypassed) position.

Operations had inserted a "B" RPS Channel half-scram to support placement of a clearance to support RPS Channel "B" scram contactor maintenance. When fuse 2-FU1-99-5A/K27B was pulled a full scram signal was generated. The cause of the scram was the fact that the SDV vents and drains were closed and the volume had filled above the scram setpoint. Even though the High Level Scram signal for the SDV was in bypass, the pulling of the fuse also opened a contact in the "A" scram circuit. The SDV High Level Scram signal, in conjunction with the open contact, negated the key lock bypass contact and resulted in the receipt of a RPS actuation (full scram).

V. ASSESSMENT OF SAFETY CONSEQUENCES

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9-2007)		LICENSE	E EVENT R	EPORT	(LER)			
•		FACILITY NAME (1)	DOCKET (2)	L	ER NUMBER (6)	PAGE (3)	
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Browns	s Ferr	y Nuclear Plant Unit 2	05000260	2009	006	00	4 of 4	
IARRAT	VE							
	The s 5 with comp	afety consequences of this event a all control rods inserted, no react plicated. TVA concludes that the ev	were not signi or parameters vent did not af	ficant. Sin were cha fect the he	nce the react anged. The re ealth and safe	or was alre eactor scra ety of the p	eady in Mode am was not oublic.	
VI.	COR	RECTIVE ACTIONS						
	A .	Immediate Corrective Actions						
		Immediate corrective action was was held in the Unit 2 Main Contr	to reset the so ol Room, whic	ram per p ch include	procedure 2-C	0I-99. A sta ontrol roon	and-down n, Work	
		Control Center, and Outage Cont conditions.	rol Center stat	ff, to discu	iss all ongoin	g activities	and unit	
	В.	Corrective Actions to Prevent F the Browns Ferry Nuclear Plant c	Recurrence - orrective actio	The corre	ective actions	are being	managed by	
·		Corrective actions include a traini Licensed Operator Requalification	ing needs ana n training.	lysis of the	e event for po	ossible incl	usion into	
VII.	ADD	TIONAL INFORMATION						
	Α.	Failed Components						
		None.			-			
	В.	Previous LERs on Similar Even	<u>its</u>					
		None.						
	C.	Additional Information						
		Corrective action documents PER 172053, PER 178146 and SR 84741.						
	D.	Safety System Functional Failure Consideration:						
		This event is not a safety system	functional fail	ure in acc	ordance with	NEI 99-02		
	E.	Scram with Complications Consideration:						
		This event was not a complicated	scram accore	ding to NE	El 99-02.	. *		
VIII.	сом	MITMENTS						
	None							
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