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Administrative Topics Outline

Form ES-301-1

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Examination Level (circl	e one): R	O ASRO> Operating Test Number: <u>HLT 0801</u>
Administrative Topic (see Note)	Type Code *	Describe activity to be performed
Conduct of Operations	Μ	Determine Adequate Performance of License Reactivation JPM 541 (RO / SRO)
Conduct of Operations	Р	Evaluate Recombiner Performance JPM 510 (RO/SRO)
Equipment Control	D	Perform Jet Pump Surveillance 2-SR-3.4.2.1 JPM 120F (RO / SRO)
Radiation Control	N	Calculate the maximum permissible stay time within emerge dose limits and determine the approving authority JPM 528 (RO / SRO)
Emergency Plan	N, S	Classify the Event per the REP (Primary System Leakage (Torus Pressure) Exceeding PSP Curve) JPM 487TC (SRO Only)
NOTE: All items (5 tota are retaking only the ac	l) are requ Iministrati	uired for SROs. RO applicants require only 4 items unless the vertice of the second seco
* Type Codes & Criteria:	(C) (D) (N) (P) (S)	ontrol Room irect from bank (< 3 for ROs; < 4 for SROs and RO retakes) ew or (M)odified from bank (> 1) revious 2 exams (< 1; randomly selected) imulator

Submitted 5/21/09, RSS

SRO only JPMs shown in italics

Administrative Topics Narrative

Administrative Topics:

a. Determine Adequate Performance of License Reactivation

- Modified: OPDP-10 is a new procedure that removed information from OPDP-1, Conduct of Operations; and modified critical data in JPM
- OPDP-10, License Status Maintenance, Reactivation and Proficiency for Non-Licensed Positions, Rev. 0, Appendix E
- G2.1.4 Knowledge of individual licensed operator responsibilities related to shift staffing, such as medical requirements, "no-solo" operation, maintenance of active license status, 10CFR55, etc. (CFR: 41.10 / 43.2) IMPORTANCE: RO 3.3 SRO 3.8

b. Evaluate Recombiner Performance

- Previous: JPM revised to newest revision of 3-OI-66
- 3-OI-66, Off-Gas System, Rev 57
- G2.1.7 Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation. (CFR: 41.5 / 43.5 / 45.12 / 45.13) IMPORTANCE RO 4.4 SRO 4.7

c. Perform Jet Pump Surveillance 2-SR-3.4.2.1

- Direct: JPM revised to newest revision of 2-SR-3.4.2.1
- 2-SR-3.4.2.1, Jet Pump Operability, Rev 25
- G2.2.12 Knowledge of of Surveillance Procedures. (CFR: 41.10 / 43.3 / 45.13) IMPORTANCE: RO 3.7 SRO 4.1
- d. Calculate the maximum permissible stay time within emergency dose limits and determine the approving authority
 - New: JPM Classroom
 - EPIP-15, EMERGENCY EXPOSURE, Rev 9
 - G 2.3.4 Knowledge of radiation exposure limits under normal or emergency conditions. (CFR: 41.12 / 43.4 / 45.10) IMPORTANCE RO 3.2 SRO 3.7

Administrative Topics Narrative

Administrative Topics: (continued)

- e. Classify the Event per the REP (Primary Sys. Leakage (Torus Pressure) Exceeding PSP Curve)
 - New JPM Simulator
 - EPIP-1, Emergency Classification Procedure, Rev. 43, Primary Containment 2.0, Section 2.1
 - EPIP-4, Site Area Emergency, Rev. 31
 - SRO Only
 - G2.4.38 Ability to take actions called for in the facility emergency plan, including supporting or acting as emergency coordinator if required. (CFR: 41.10 / 43.5 / 45.11) IMPORTANCE: SRO 4.4

Form ES-301-1

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Facility: Browns Fer	ry NPP	Date of Examination: July 6 – 30, 2009						
Examination Level (circ	le one): R	O KSRO> Operating Test Number: HLT 0801						
Administrative Topic (see Note)	Type Code *	Describe activity to be performed						
Conduct of Operations	М	Determine Adequate Performance of License Reactivation JPM 541 (RO / SRO)						
Conduct of Operations	D	Perform Jet Pump Surveillance 2-SR-3.4.2.1 JPM 120F (RO / SRO)						
Equipment Control	Р	Determine Correct Method of Verification on a Given System JPM 550 (RO / SRO)						
Radiation Control	N	Calculate the maximum permissible stay time within emergency dose limits and determine the approving authority JPM 528 (RO / SRO)						
Emergency Plan	N, S	Classify the Event per the REP (Primary System Leakage (Torus Pressure) Exceeding PSP Curve) JPM 487TC (SRO Only)						
NOTE: All items (5 tota are retaking only the ad	al) are requ Iministrati	uired for SROs. RO applicants require only 4 items unless they ive topics, when all 5 are required.						
* Type Codes & Criteria	: (C) (D) (N) (P) (S)	ontrol Room irect from bank (≤ 3 for ROs; ≤ 4 for SROs and RO retakes) ew or (M)odified from bank (≥ 1) revious 2 exams (≤ 1; randomly selected) imulator						

Submitted on 5/11/09

SRO only JPMs shown in italics

Administrative Topics Narrative

Administrative Topics:

- a. Determine Adequate Performance of License Reactivation
 - Modified: OPDP-10 is a new procedure that removed information from OPDP-1, Conduct of Operations; and modified critical data in JPM
 - OPDP-10, License Status Maintenance, Reactivation and Proficiency for Non-Licensed Positions, Rev. 0, Appendix E
 - G2.1.4 Knowledge of individual licensed operator responsibilities related to shift staffing, such as medical requirements, "no-solo" operation, maintenance of active license status, 10CFR55, etc. (CFR: 41.10 / 43.2) IMPORTANCE: RO 3.3 SRO 3.8

b. Perform Jet Pump Surveillance 2-SR-3.4.2.1

- Direct: JPM revised to newest revision of 2-SR-3.4.2.1
- 2-SR-3.4.2.1, Jet Pump Operability, Rev 25
- G2.1.7 Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation. (CFR: 41.5 / 43.5 / 45.12 / 45.13) IMPORTANCE RO 4.4 SRO 4.7

c. Determine Correct Method of Verification on a Given System

- Previous NRC Exam: HLT 0707 (Modified slightly based on NRC feedback)
- SPP-10.3, Verification Program, Rev. 1, Sections 3.3.1, 3.4.2, and 3.4.3
- G2.2.14 Knowledge of the process for controlling equipment configuration or status. (CFR: 41.10 / 43.3 / 45.13) IMPORTANCE: RO 3.9 SRO 4.3
- d. Calculate the maximum permissible stay time within emergency dose limits and determine the approving authority
 - New: JPM Classroom
 - EPIP-15, EMERGENCY EXPOSURE, Rev 9
 - G 2.3.4 Knowledge of radiation exposure limits under normal or emergency conditions. (CFR: 41.12 / 43.4 / 45.10) IMPORTANCE RO 3.2 SRO 3.7

SRO only JPMs shown in italics

Administrative Topics Narrative

Administrative Topics: (continued)

- e. Classify the Event per the REP (Primary Sys. Leakage (Torus Pressure) Exceeding PSP Curve)
 - New JPM Simulator
 - EPIP-1, Emergency Classification Procedure, Rev. 43, Primary Containment 2.0, Section 2.1
 - EPIP-4, Site Area Emergency, Rev. 31
 - SRO Only
 - G2.4.38 Ability to take actions called for in the facility emergency plan, including supporting or acting as emergency coordinator if required. (CFR: 41.10 / 43.5 / 45.11) IMPORTANCE: SRO 4.4

Administrative Topics Outline

Form ES-301-1

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F	acility: E	Browns Ferr	y NPP	Date of Examination: July 6 – 30, 2009
E	Examination	Level (circl	e one): R	O KSRO> Operating Test Number: HLT 0801
A	dministrati (see No	ve Topic ote)	Type Code *	Describe activity to be performed
Co	onduct of Op	perations	Μ	Determine Adequate Performance of License Reactivation JPM 541 (RO / SRO)
Co	onduct of Op	perations	D	Conduct of Operations Question – Required Turnover Information JPM 512 (RO / SRO)
Ec	quipment Cc	ontrol	Ρ	Determine Correct Method øf Verification on a Given System JPM 550 (RO / SRO)
Ra	adiation Cor	ntrol	D	Determine the Dose Limitation for Declared and Undeclared Pregnant Female Employees and Their Eligibility for Overtime JPM 511 (RO / SRO)
Er	mergency Pl	lan	N, S	Classify the Event per the REP (Primary System Leakage (Torus Pressure) Exceeding PSP Curve) JPM 487TC (SRO Only)
N(ar	OTE: All ite e retaking o	ems (5 tota only the ad	l) are requ Iministrati	ired for SROs. RO applicants require only 4 items unless they ve topics, when all 5 are required.
*	Type Codes	s & Criteria:	(C)d (D)i (N)e (P)r (S)ii	ontrol Room rect from bank (≤ 3 for ROs; ≤ 4 for SROs and RO retakes) ew or (M)odified from bank (≥ 1) evious 2 exams (≤ 1; randomly selected) mulator

Administrative Topics Narrative

Administrative Topics:

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- a. Determine Adequate Performance of License Reactivation
 - Modified: OPDP-10 is a new procedure that removed information from OPDP-1, Conduct of Operations; and modified critical data in JPM
 - OPDP-10, License Status Maintenance, Reactivation and Proficiency for Non-Licensed Positions, Rev. 0, Appendix E
 - G2.1.4 Knowledge of individual licensed operator responsibilities related to shift staffing, such as medical requirements, "no-solo" operation, maintenance of active license status, 10CFR55, etc. (CFR: 41.10 / 43.2) IMPORTANCE: RO 3.3 SRO 3.8

b. Conduct of Operations Question - Required Turnover Information

- Direct from Bank
- OPDP-1, Conduct of Operations, Rev. 12, section 7.3.L
- G2.1.3 Knowledge of shift or short-term relief turnover practices. (CFR: 41.10 / 45.13) IMPORTANCE: RO 3.7 SRO 3.9
- c. Determine Correct Method of Verification on a Given System
 - Previous NRC Exam: HLT 0707 (Modified slightly based on NRC feedback)
 - SPP-10.3, Verification Program, Rev. 1, Sections 3.3.1, 3.4.2, and 3.4.3
 - G2.2.14 Knowledge of the process for controlling equipment configuration or status. (CFR: 41.10 / 43.3 / 45.13) IMPORTANCE: RO 3.9 SRO 4.3
- d. Determine the Dose Limitation for Declared and Undeclared Pregnant Female Employees and Their Eligibility for Overtime
 - Direct from Bank
 - SPP-5.1, Radiological Controls, Rev. 6, Section 3.4.1
 - G2.3.4 Knowledge of radiation exposure limits under normal or emergency conditions. (CFR: 41.12 / 43.4 / 45.10) IMPORTANCE: RO 3.2 SRO 3.7

Administrative Topics Narrative

Administrative Topics: (continued)

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- e. Classify the Event per the REP (Primary Sys. Leakage (Torus Pressure) Exceeding PSP Curve)
 - New JPM Simulator
 - EPIP-1, Emergency Classification Procedure, Rev. 43, Primary Containment 2.0, Section 2.1
 - EPIP-4, Site Area Emergency, Rev. 31
 - SRO Only
 - G2.4.38 Ability to take actions called for in the facility emergency plan, including supporting or acting as emergency coordinator if required. (CFR: 41.10 / 43.5 / 45.11) IMPORTANCE: SRO 4.4

Control Room/In-Plant Systems Outline

Fac	ility: Browns Ferry NPP Date of Exam	nination: July 6 – 3	30, 2009			
Exa	Im Level (circle one): RO (SRO-I)/ SRO-U Operating Te	st No.: HLT 0801	1			
Con	trol Room Systems [@] (8 for RO); (7 for SRO-I); (2 or 3 for	SRO-U, including	1 ESF)			
	System / JPM Title	Type Code*	Safety Function			
a.	Alternate RPV Injection – Standby Coolant	S, E, EN, L, N	2			
b.	RWM Functional Test for Startup 3-SR-3.3.2.1.2	S, L, N	7			
C.	Place RCIC in Test Mode From Standby for Alternate RPV Pressure Control	e S,A, D, E, L	3			
d.	Start and Inject SLC Solution Into the RPV	S, A, M, E, L	1			
e.	Emergency Ventilate Primary Containment	S, D, E, L	5			
f.	Parallel D/G With System 4KV S/D Board at Panel 9 23	S, A, D	6			
g.	Off-Gas Post-Treatment Radiation HI-HI-HI	S, A, D, E, L	9			
h.	Placing Standby Steam Jet Air Ejector in Operation	S, E, N	4			
In-P	lant Systems [@] (3 for RO); (3 for SRO-I); (3 or 2 for SRO-	U)				
i.	Shift From 'A' Set of CRD Stabilizing Valves to 'B' Set	et D, E, L, R	1			
j.	Place B2-3 ± 24V Neutron Monitoring Battery Charger in Service to Battery Board 3	tery A, N				
k.	Bypass RCIC Low Pressure Isolation 3-EOI Appendix-16A	EOI D, E, EN, L, 2 R				
@	All RO and SRO-I control room (and in-plant) systems must be functions; all 5 SRO-U systems must serve different safety fun may overlap those tested in the control room.	different and serve diff ctions; in-plant system	erent safety s and functions			
	* Type Codes C	riteria for RO / SRO-I / S	SRO-U			
(A)Ite	rnate path	4-6 / 4-6 / 2-3				
(C)on	itrol room	and the second second				
(D)ire	ct from bank	$\leq 9 / \leq 8 / \leq 4$				
(E)me	argency or abnormal in-plant	≥1/≥1/≥1				
(EN)g	Jineered safety feature	-/-/21(0	control room system,			
(L)OW	-Power / Shutdown	21/21/21				
(IV)CV	vious 2 avoms	<3/<3/<2	(randomly selected)			
(R)C/	A	>1/>1/>1	(Idiadonity adicates,			
(S)im	ulator					

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Control Room/In-Plant Systems JPM Narrative

Control Room Systems:

- a. Alternate RPV Injection Standby Coolant (JPM 347 U3)
 - New / Emergency or Abnormal In-Plant / Low-Power / ENgineered Safety Feature / Simulator
 - 3-EOI Appendix-7D, Alternate RPV Injection System Lineup Standby Coolant, Rev. 2
 - 203000 RHR/LPCI: Injection Mode (Plant Specific) A4.07 Ability to manually operate and/or monitor in the control room: Reactor Water Level IMPORTANCE: RO 4.5 SRO 4.5

b. RWM Functional Test for Startup 3-SR-3.3.2.1.2 (JPM 399 U3)

- New / Low-Power / Simulator
- 3-SR-3.3.2.1.2, RWM Functional Test for Startup, Rev. 3
- 201006 Rod Worth Minimizer System (RWM) (Plant Specific) A2.05 Ability to (a) predict the impacts of the following on the ROD WORTH MINIMIZER SYSTEM (RWM); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Out of sequence rod movement: Plant-Specific (Not BWR-6) IMPORTANCE: RO 3.1 SRO 3.5
- c. Place RCIC in Test Mode From Standby for Alternate RPV Pressure Control (JPM 43F U2)
 - Direct from bank / Emergency or Abnormal In-Plant / Low-Power / Simulator
 - Alternate Path: RCIC Trip Throttle Valve (71-9) is tripped and must be reset and opened. RCIC Flow Controller fails while in service and candidate must take manual control
 - 2-EOI Appendix-11B, Alternate RPV Pressure Control Systems RCIC Test Mode, Rev. 5
 - 241000 Reactor/Turbine Pressure Regulating System A4.02 Ability to manually operate and/or monitor in the control room: Reactor pressure IMPORTANCE: RO 4.1 SRO 4.1

d. Start and Inject SLC Solution Into the RPV (JPM 613F U2)

- Modified / Emergency or Abnormal In-Plant / Low-Power / Simulator
- Alternate Path: must start other pump due to no flow
- 2-EOI Appendix-3A, SLC Injection, Rev. 5
- 211000 Standby Liquid Control A4.08 Ability to manually operate and/or monitor in the control room: System initiation: Plant-specific IMPORTANCE: RO 4.2 SRO 4.2

- e. Emergency Ventilate Primary Containment (JPM 55 U2)
 - Direct from bank / Emergency or Abnormal In-Plant / Low-Power / Simulator
 - 2-EOI Appendix-13, Emergency Venting Primary Containment, Rev. 6
 - 223001 Primary Containment System and Auxiliaries A2.07 Ability to (a) predict the impacts of the following on the PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: High drywell pressure IMPORTANCE: RO 4.2 SRO 4.3
- f. Parallel D/G With System 4KV S/D Board at Panel 9-23 (JPM 104F U2)
 - Direct from bank / Simulator
 - Alternate Path: Once paralleled with offsite source, a voltage transient will occur requiring candidate to trip the output breaker
 - 0-OI-82, Standby Diesel Generator System, Rev. 99, Section 8.1
 - 264000 Emergency Generators (Diesel/Jet) A4.04 Ability to manually operate and/or monitor in the control room: Manual start, loading, and stopping of emergency generator: Plant-specific IMPORTANCE: RO 3.7 SRO 3.7
- g. Off-Gas Post-Treatment Radiation HI-HI-HI (JPM 390F U3)
 - Direct from bank / Emergency or Abnormal In-Plant / Low-Power / Simulator
 - Alternate Path: Offgas System Isolation Valve, 3-FCV-66-28 fails to close automatically requiring candidate to close manually
 - 3-ARP-9-4C, Panel 9-4 3-XA-55-4C, Rev. 29, Window 35 (Page 45 of 45) and 3-AOI-66-2, Offgas Post Treatment Radiation Hi Hi Hi, Rev. 9
 - 271000 Offgas System A2.04 Ability to (a) predict the impacts of the following on the OFFGAS SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Offgas system high radiation IMPORTANCE: RO 3.7 SRO 4.1
- h. Placing Standby Steam Jet Air Ejector in Operation (JPM 346 U3)
 - New / Emergency or Abnormal In-Plant / Simulator
 - 3-OI-66, Off-Gas System, Rev. 56, Section 8.4
 - 239001 Main and Reheat Steam System A2.08 Ability to (a) predict the impacts of the following on the MAIN AND REHEAT STEAM SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Low condenser vacuum IMPORTANCE: RO 3.6 SRO 3.6

Control Room/In-Plant Systems JPM Narrative

In-Plant Systems:

- i. Shift From 'A' Set of CRD Stabilizing Valves to 'B' Set (JPM 113 U2)
 - Direct from bank / Emergency or Abnormal In-Plant / Low-Power / RCA Entry
 - 2-OI-85, Control Rod Drive System, Rev. 112, Section 6.4
 - 201001 Control Rod Drive Hydraulic System A3.01 Ability to monitor automatic operations of the CONTROL ROD DRIVE HYDRAULIC SYSTEM including: Valve operation IMPORTANCE: RO 3.0 SRO 3.0
- j. Place B2-3 + 24V Neutron Monitoring Battery Charger in Service to Battery Board 3 (JPM 308F U3)
 - New
 - Alternate Path: When the Battery Charger is placed in service, voltage and amps will be out-of-specification requiring candidate to open breakers and notify the Unit Supervisor
 - 0-OI-57D, DC Electrical System, Rev. 121, Section 5.13
 - 263000 D.C. Electrical Distribution K1.02 Knowledge of the physical connections and/or cause-effect relationships between D.C. ELECTRICAL DISTRIBUTION and the following: Battery charger and battery IMPORTANCE: RO 3.2 SRO 3.3
- k. Bypass RCIC Low Pressure Isolation 3-EOI Appendix-16A (JPM 323 U3)
 - Direct from bank / Emergency or Abnormal In-Plant / Low-Power / ENgineered Safety Feature / RCA Entry
 - 3-EOI Appendix-16A, Bypassing RCIC Low RPV Pressure Isolation Interlocks, Rev. 1
 - 217000 Reactor Core Isolation Cooling System (RCIC) A2.03 Ability to (a) predict the impacts of the following on the REACTOR CORE ISOLATION COOLING SYSTEM (RCIC); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Valve closures IMPORTANCE: RO 3.4 SRO 3.3

Form ES-301-2

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Fac	cility: Browns Ferry NPP Da	te of Examinati	on: July 6 – 3	0, 2009		
Exa	am Level (circle one): RO (SRO-I)/ SRO-U Op	erating Test No	D.: HLT 0801			
Con	trol Room Systems [@] (8 for RO); (7 for SRO-I);	(2 or 3 for SRC	D-U, including	1 ESF)		
	System / JPM Title		Type Code*	Safety Function		
a.	Start and Inject SLC Solution Into the RPV	S	S, A, M, E, L	1		
b.	Alternate RPV Injection - Standby Coolant	S	, E, EN, L, N	2		
C.	Place RCIC in Test Mode From Standby fo RPV Pressure Control	or Alternate	S,A, D, E, L	3		
d.	Placing Standby Steam Jet Air Ejector in C	peration	S, E, N	4		
e.	Emergency Ventilate Primary Containment		S, D, E, L	5		
f.	Parallel D/G With System 4KV S/D Board a 23	at Panel 9-	S, A, D	6		
g.	RWM Functional Test for Startup 3-SR-3.3	.2.1.2	S, L, N	7		
h.	Off-Gas Post-Treatment Radiation HI-HI-H	1 8	6, A, D, E, L	9		
In-P	lant Systems [@] (3 for RO); (3 for SRO-I); (3 or 2	for SRO-U)				
i.	Shift From 'A' Set of CRD Stabilizing Valve	s to 'B' Set	D, E, L, R	1		
j.	Bypass RCIC Low Pressure Isolation 3-EC Appendix-16A	EOI D, E, EN, L, 2 R				
k.	Place B2-3 ± 24V Neutron Monitoring Batte Charger in Service to Battery Board 3	tery A, N 6				
@	All RO and SRO-I control room (and in-plant) system functions; all 5 SRO-U systems must serve different may overlap those tested in the control room.	ms must be differ at safety functions	ent and serve diffes; in-plant systems	erent safety s and functions		
	* Type Codes	Criteria	for RO / SRO-I / S	RO-U		
(A)Ite	rnate path		4-6 / 4-6 / 2-3			
(C)on	trol room					
(D)ire	ct from bank		≤9/≤8/≤4			
(E)me	ergency or abnormal in-plant		21/21/21	and and an array of the		
(EN)	Power / Shutdowe		-/-/21(C	ontrol room system,		
(L)OW	or (M)odified from bank including 1/A)		2/22/21			
(P)res	vious 2 exams		<3/<3/<2	(randomly selected		
(R)C			>1/>1/>1/>1	framounty selected		
(S)im	ulator					

Control Room/In-Plant Systems JPM Narrative

Control Room Systems:

- a. Start and Inject SLC Solution Into the RPV (JPM 613F U2)
 - Modified / Emergency or Abnormal In-Plant / Low-Power / Simulator
 - Alternate Path: must start other pump due to no flow
 - 2-EOI Appendix-3A, SLC Injection, Rev. 5
 - 211000 Standby Liquid Control A4.08 Ability to manually operate and/or monitor in the control room: System initiation: Plant-specific IMPORTANCE: RO 4.2 SRO 4.2
- b. Alternate RPV Injection Standby Coolant (JPM 347 U3)
 - New / Emergency or Abnormal In-Plant / Low-Power / ENgineered Safety Feature / Simulator
 - 3-EOI Appendix-7D, Alternate RPV Injection System Lineup Standby Coolant, Rev. 2
 - 203000 RHR/LPCI: Injection Mode (Plant Specific) A4.07 Ability to manually operate and/or monitor in the control room: Reactor Water Level IMPORTANCE: RO 4.5 SRO 4.5
- c. Place RCIC in Test Mode From Standby for Alternate RPV Pressure Control (JPM 43F U2)
 - Direct from bank / Emergency or Abnormal In-Plant / Low-Power / Simulator
 - Alternate Path: RCIC Trip Throttle Valve (71-9) is tripped and must be reset and opened. RCIC Flow Controller fails while in service and candidate must take manual control
 - 2-EOI Appendix-11B, Alternate RPV Pressure Control Systems RCIC Test Mode, Rev. 5
 - 241000 Reactor/Turbine Pressure Regulating System A4.02 Ability to manually operate and/or monitor in the control room: Reactor pressure IMPORTANCE: RO 4.1 SRO 4.1
- d. Placing Standby Steam Jet Air Ejector in Operation (JPM 346 U3)
 - New / Emergency or Abnormal In-Plant / Simulator
 - 3-OI-66, Off-Gas System, Rev. 56, Section 8.4
 - 239001 Main and Reheat Steam System A2.08 Ability to (a) predict the impacts of the following on the MAIN AND REHEAT STEAM SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Low condenser vacuum IMPORTANCE: RO 3.6 SRO 3.6

Control Room/In-Plant Systems JPM Narrative

- e. Emergency Ventilate Primary Containment (JPM 55 U2)
 - Direct from bank / Emergency or Abnormal In-Plant / Low-Power / Simulator
 - 2-EOI Appendix-13, Emergency Venting Primary Containment, Rev. 6
 - 223001 Primary Containment System and Auxiliaries A2.07 Ability to (a) predict the impacts of the following on the PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: High drywell pressure IMPORTANCE: RO 4.2 SRO 4.3
- f. Parallel D/G With System 4KV S/D Board at Panel 9-23 (JPM 104F U2)
 - Direct from bank / Simulator
 - Alternate Path: Once paralleled with offsite source, a voltage transient will occur requiring candidate to trip the output breaker
 - 0-OI-82, Standby Diesel Generator System, Rev. 99, Section 8.1
 - 264000 Emergency Generators (Diesel/Jet) A4.04 Ability to manually operate and/or monitor in the control room: Manual start, loading, and stopping of emergency generator: Plant-specific IMPORTANCE: RO 3.7 SRO 3.7

g. RWM Functional Test for Startup 3-SR-3.3.2.1.2 (JPM 399 U3)

- New / Low-Power / Simulator
- 3-SR-3.3.2.1.2, RWM Functional Test for Startup, Rev. 3
- 201006 Rod Worth Minimizer System (RWM) (Plant Specific) A2.05 Ability to (a) predict the impacts of the following on the ROD WORTH MINIMIZER SYSTEM (RWM); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Out of sequence rod movement: Plant-Specific (Not BWR-6) IMPORTANCE: RO 3.1 SRO 3.5
- h. Off-Gas Post-Treatment Radiation HI-HI-HI (JPM 390F U3)
 - Direct from bank / Emergency or Abnormal In-Plant / Low-Power / Simulator
 - Alternate Path: Offgas System Isolation Valve, 3-FCV-66-28 fails to close automatically requiring candidate to close manually
 - 3-ARP-9-4C, Panel 9-4 3-XA-55-4C, Rev. 29, Window 35 (Page 45 of 45) and 3-AOI-66-2, Offgas Post Treatment Radiation Hi Hi Hi, Rev. 9
 - 271000 Offgas System A2.04 Ability to (a) predict the impacts of the following on the OFFGAS SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Offgas system high radiation IMPORTANCE: RO 3.7 SRO 4.1

Control Room/In-Plant Systems JPM Narrative

In-Plant Systems:

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- i. Shift From 'A' Set of CRD Stabilizing Valves to 'B' Set (JPM 113 U2)
 - Direct from bank / Emergency or Abnormal In-Plant / Low-Power / RCA Entry
 - 2-OI-85, Control Rod Drive System, Rev. 112, Section 6.4
 - 201001 Control Rod Drive Hydraulic System A3.01 Ability to monitor automatic operations of the CONTROL ROD DRIVE HYDRAULIC SYSTEM including: Valve operation IMPORTANCE: RO 3.0 SRO 3.0
- j. Bypass RCIC Low Pressure Isolation 3-EOI Appendix-16A (JPM 323 U3)
 - Direct from bank / Emergency or Abnormal In-Plant / Low-Power / ENgineered Safety Feature / RCA Entry
 - 3-EOI Appendix-16A, Bypassing RCIC Low RPV Pressure Isolation Interlocks, Rev. 1
 - 217000 Reactor Core Isolation Cooling System (RCIC) A2.03 Ability to (a) predict the impacts of the following on the REACTOR CORE ISOLATION COOLING SYSTEM (RCIC); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Valve closures IMPORTANCE: RO 3.4 SRO 3.3
- k. Place B2-3 + 24V Neutron Monitoring Battery Charger in Service to Battery Board 3 (JPM 308F U3)
 - New
 - Alternate Path: When the Battery Charger is placed in service, voltage and amps will be out-of-specification requiring candidate to open breakers and notify the Unit Supervisor
 - 0-OI-57D, DC Electrical System, Rev. 121, Section 5.13
 - 263000 D.C. Electrical Distribution K1.02 Knowledge of the physical connections and/or cause-effect relationships between D.C. ELECTRICAL DISTRIBUTION and the following: Battery charger and battery IMPORTANCE: RO 3.2 SRO 3.3

ES-401

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	10.0	wns F	erry	NPP	,	_	D	ate o	of Ex	am:	,	July	06 - 3	30, 2009				_							
				RO K/A Category Points														SRO-Only Points							
Tier	Gr	oup	K 1	K 2	К 3	K 4	K 5	К 6	A 1	A 2	A 3	A 4	G.	Total	A	2	c	3*	Tota						
1.		1	4	3	4				3	2			4	20		1		3	7						
Emergenc &	y	2	2	2	2	2	2	2	2	1	1				1	1		1/4	1	7	2		1		3
Abnormal Plant Evolutions	Т	ier stals	6	4	5		10/0		4	3			5	27		5		4	10						
		1	4	2	3	2	2 2 3 2 2 2 2 2							26	1	3		2	5						
2. Plant		2	2	1	1	1	1	1	1	1	1	1	1	12	0	2	1		3						
Systems	T To	ier stals	6	3	4	3	3	4	3	3	3	3	3	38		5		3	8						
3	Generic	Know	ledae	and			1	1	2	3	3		4		1	2	3	4							
0.	Abiliti	es Cat	egori	es		2		2		3			3	10	2	2	2	1	7						
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Form ES-401-1

ES-401 BWR Examination Outline Form E Emergency and Abnormal Plant Evolutions – Tier 1/Group 1 (RO / SRO)											
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#		
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4				1			AA1.02 (10CFR 55.41.7) Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION: • RPS	3.3	1		
295003 Partial or Complete Loss of AC / 6	1					1	 AK1.03 (10CFR 55.41.8) Knowledge of the operational implications of the following concepts as they apply to PARTIAL OR COMPLETE LOSS OF AC POWER: Under voltage / degraded voltage effects on electrical loads 	2.9	2		
							G2.4.50 (10CFR 55.43.5 - SRO Only) Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.	4.0	76		
295004 Partial or Total Loss of DC Pwr / 6			1				 AK3.02 (10CFR 55.41.5) Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF DC POWER: Ground isolation / fault determination 	2.9	3		
295005 Main Turbine Generator Trip / 3						1	G2.4.21 (10CFR 55.41.7) Knowledge of the parameters and logic used to assess the status of safety functions, such as reactivity control, core cooling and heat removal, reactor coolant system integrity, containment conditions, radioactivity release control, etc.	4.0	4		
295006 SCRAM / 1		1					 AK2.02 (10CFR 55.41.7) Knowledge of the interrelations between SCRAM and the following: Reactor water level control system 	3.8	5		

ES-401							BWR Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1/Group 1 (RO / SRO)	Form E	S-401-1
E/APE # / Name / Safety Function	К 1	K 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
295016 Control Room Abandonment / 7						1	G2.4.30 (10CFR 55.41.10) Knowledge of events related to system operation/status that must be reported to internal organizations or external agencies, such as the State, the NRC, or the transmission system operator.	2.7	6
						1	G2.4.3 (10CFR 55.43.5 - SRO Only) Ability to identify post-accident instrumentation.	3.9	77
295018 Partial or Total Loss of CCW / 8				1	1		 AA1.02 (10CFR 55.41.7) Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF COMPONENT COOLING WATER: System loads 	3.3	7
							 AA2.01 (10CFR 55.43.5 - SRO Only) Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF COMPONENT COOLING WATER: Component temperatures 	3.4	78
295019 Partial or Total Loss of Inst. Air / 8		1					 AK2.18 (10CFR 55.41.7) Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR and the following: ADS: Plant-Specific 	3.5	8
295021 Loss of Shutdown Cooling / 4						1	G2.2.4 (10CFR 55.41.7) (multi-unit license) Ability to explain the variations in control board/control room layouts, systems, instrumentation, and procedural actions between units at a facility.	3.6	9

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ES-401							BWR Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1/Group 1 (RO / SRO)	Form E	ES-401-1
E/APE # / Name / Safety Function	К 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
295023 Refueling Acc / 8				1	1		 AA1.04 (10CFR 55.41.7) Ability to operate and/or monitor the following as they apply to REFUELING ACCIDENTS: Radiation monitoring equipment 	3.4	10
							AA2.01 (10CFR 55.43.5 - SRO Only) Ability to determine and/or interpret the following as they apply to REFUELING ACCIDENTS: • Area radiation levels	4.0	79
295024 High Drywell Pressure / 5					1		EA2.03 (10CFR 55.41.10) Ability to determine and/or interpret the following as they apply to HIGH DRYWELL PRESSURE: • Suppression pool level	3.8	11
295025 High Reactor Pressure / 3		1			1		EK2.04 (10CFR 55.41.7) Knowledge of the interrelations between HIGH REACTOR PRESSURE and the following: ARI/RPT/ATWS: Plant-Specific	3.9	12
							EA2.01 (10CFR 55.43.5 - SRO Only) Ability to determine and/or interpret the following as they apply to HIGH REACTOR PRESSURE: • Reactor pressure	4.3	80
295026 Suppression Pool High Water Temp. / 5			1				EK3.02 (10CFR 55.41.5) "Details in Record of Rejected K/As" Knowledge of the reasons for the following responses as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE: • Suppression pool cooling	3.9	13
295027 High Containment Temperature / 5	-	-	-	-	-	-	N/A for BFN - K/A for Mark III Containments ONLY		

ES-401		<u> </u>					BWR Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1/Group 1 (RO / SRO)	Form E	S-401-1
E/APE # / Name / Safety Function	К 1	K 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
295028 High Drywell Temperature / 5	1				1		EK1.01 (10CFR 55.41.8)	1	
							Knowledge of the operational implications of the following concepts as they apply to HIGH DRYWELL TEMPERATURE:	3.5	14
							 Reactor water level measurement 		
							EA2.01 (10CFR 55.43.5 - SRO Only) **Details in Record of Rejected K/As**		
							Ability to determine and/or interpret the following as they apply to HIGH DRYWELL TEMPERATURE:	4.1	81
							Drywell temperature		
295030 Low Suppression Pool Wtr Lvl / 5			1				EK3.03 (10CFR 55.41.5)		
							Knowledge of the reasons for the following responses as they apply to LOW SUPPRESSION POOL WATER LEVEL:	3.6	15
							RCIC operation: Plant-Specific		
295031 Reactor Low Water Level / 2					1	· .	EA2.03 (10CFR 55.41.10)		
							Ability to determine and/or interpret the following as they apply to REACTOR LOW WATER LEVEL:	4.2	16
							Reactor pressure		
295037 SCRAM Condition Present and						1	G2.2.12 (10CFR 55.41.10)	07	477
Unknown / 1							Knowledge of surveillance procedures.	3.7	17
295038 High Off-site Release Rate / 9	1						EK1.03 (10CFR 55.41.10) **Details in Record of Rejected K/As**		
							Knowledge of the operational implications of the following concepts as they apply to HIGH OFF-SITE RELEASE RATE:	2.8	18
							 Meteorological effects on off-site release 		

ES-401							BWR Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1/Group 1 (RO / SRO)	Form E	S-401-1
E/APE # / Name / Safety Function	к 1	K 2	к 3	A 1	A 2	G	K/A Topic(s)	IR	#
600000 Plant Fire On Site / 8			1				 AK3.04 (10CFR 55.41.5) Knowledge of the reasons for the following responses as they apply to PLANT FIRE ON SITE: Actions contained in the abnormal procedure for plant fire on site 	2.8	19
700000 Generator Voltage and Electric Grid Disturbances / 6	1					1	 AK1.01 (10CFR 55.41.5) Knowledge of the operational implications of the following concepts as they apply to GENERATOR VOLTAGE AND ELECTRIC GRID DISTURBANCES: Definition of terms: volts, watts, amps, VARs, power factor 	3.3	20
							G2.4.31 (10CFR 55.43.5 - SRO Only) Knowledge of annunciator alarms, indications, or response procedures.	4.1	82
K/A Category Totals: (RO)	4	3	4	3	2	4	Group Point Total: (RO)		20
K/A Category Totals: (SRO)					4	3	Group Point Total: (SRO)		7

ES-401							BWR Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1/Group 2 (RO / SRO)	Form E	S-401-1
E/APE # / Name / Safety Function	К 1	К 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
295002 Loss of Main Condensor Vac / 3	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
295007 High Reactor Pressure / 3	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
295008 High Reactor Water Level / 2	-	-	-		-	-	NOT RANDOMLY SELECTED		
295009 Low Reactor Water Level / 2		1					AK2.04 (10CFR 55.41.7) Knowledge of the interrelations between LOW REACTOR WATER LEVEL and the following: Reactor water cleanup	2.6	21
295010 High Drywell Pressure / 5						1	G2.4.49 (10CFR 55.41.10) Ability to perform without reference to procedures those actions that require immediate operation of system components and controls.	4.6	22
295011 High Containment Temp / 5	-	-	-	1	-	-	N/A for BFN - K/A for Mark III Containments ONLY		
295012 High Drywell Temperature / 5	1					1	AK1.01 (10CFR 55.41.9) Knowledge of the operational implications of the following concepts as they apply to HIGH DRYWELL TEMPERATURE: • Pressure / temperature relationship	3.3	23
							G2.4.34 (10CFR 55.43.5 - SRO Only) Knowledge of RO tasks performed outside the main control room during an emergency and the resultant operational effects.	4.1	83
295013 High Suppression Pool Temp. / 5					1		AA2.02 (10 CFR 55.43.5 - SRO Only) Ability to determine and/or interpret the following as they apply to HIGH SUPPRESSION POOL TEMPERATURE: • Localized heating / stratification	3.5	84
295014 Inadvertent Reactivity Addition / 1	-	-	-	-	-	-	NOT RANDOMLY SELECTED		

SRO only K/As shown in italics

Form ES-401-1

ES-401							BWR Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1/Group 2 (RO / SRO)	Form E	S-401-1
E/APE # / Name / Safety Function	К 1	K 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
295015 Incomplete SCRAM / 1	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
295017 High Off-site Release Rate / 9			1				AK3.03 (10CFR 55.41.5) Knowledge of the reasons for the following responses as they apply to HIGH OFF-SITE RELEASE RATE: Implementation of the site emergency plan	3.3	24
295020 Inadvertent Cont. Isolation / 5 & 7	-	-	-	+	-	+	NOT RANDOMLY SELECTED		
295022 Loss of CRD Pumps / 1					1		AA2.03 (10CFR 55.41.10) Ability to determine and/or interpret the following as they apply to LOSS OF CRD PUMPS: CRD mechanism temperatures	3.1	25
295029 High Suppression Pool Wtr Lvl / 5				1			EA1.04 (10CFR 55.41.7) Ability to operate and/or monitor the following as they apply to HIGH SUPPRESSION POOL WATER LEVEL: • RCIC: Plant-Specific	3.4	26
295032 High Secondary Containment Area Temperature /-5	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
295033 High Secondary Containment Area Radiation Levels / 9					1		EA2.03 (10 CFR 55.43.5 - SRO Only) Ability to determine and/or interpret the following as they apply to HIGH SECONDARY CONTAINMENT AREA RADIATION LEVELS: Cause of high area radiation	4.2	85
295034 Secondary Containment Ventilation High Radiation / 9	-	-	-	-		-	NOT RANDOMLY SELECTED		
295035 Secondary Containment High Differential Pressure / 5	+	-	-	-	-	1	NOT RANDOMLY SELECTED		

3

ES-401							BWR Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1/Group 2 (RO / SRO)	Form E	S-401-1
E/APE # / Name / Safety Function	К 1	K 2	к 3	A 1	A 2	G	K/A Topic(s)	IR	#
295036 Secondary Containment High Sump/Area Water Level / 5	1						 EK1.01 (10CFR 55.41.10) Knowledge of the operational implications of the following concepts as they apply to SECONDARY CONTAINMENT HIGH SUMP/AREA WATER LEVEL: Radiation releases 	2.9	27
500000 High CTMT Hydrogen Conc. / 5	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
		-	-						-
	-	-	-	-		-			
	-	-	-			1			
		-							
K/A Category Totals: (RO)	2	1	1	1	1	1	Group Point Total: (RO)		7
K/A Category Totals: (SRO)					2	1	Group Point Total: (SRO)		3

ES-401										Plant	BWR	R Examination Outline terns – Tier 2/Group 1 (RO / SRO)	Form E	S-401-1
System # / Name	К 1	K 2	К 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
203000 RHR/LPCI: Injection Mode					1							 K5.02 (10CFR 55.41.5) **Details in Record of Rejected K/As** Knowledge of the operational implications of the following concepts as they apply to RHR / LPCI: INJECTION MODE (PLANT SPECIFIC): ♦ Core cooling methods 	3.5	28
205000 Shutdown Cooling		1								1		 K2.02 (10CFR 55.41.7) Knowledge of electrical power supplies to the following: Motor operated valves 	2.5	29
												 A4.03 (10CFR 55.41.7) **Details in Record of Rejected K/As** Ability to manually operate and/or monitor in the control room: SDC / RHR discharge valves 	3.6	30
206000 HPCI				1				1				 K4.03 (10CFR 55.41.7) **Details in Record of Rejected K/As** Knowledge of HIGH PRESSURE COOLANT INJECTION (HPCI) SYSTEM design feature(s) and/or interlocks which provide for the following: Resetting turbine trips: BWR-2,3,4 	4.2	31
												 A2.16 (10CFR 55.43.5 - SRO Only) Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION (HPCI) SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: High drywell pressure: BWR-2,3,4 	4.1	86
207000 Isolation (Emergency) Condenser	+	-	-	-		-	-	+	-	-	-	N/A for BFN - No Isolation Condenser(s)		

SRO only K/As shown in italics

ES-401	-									Plant	BWR Syst	Examination Outline ems – Tier 2/Group 1 (RO / SRO)	Form E	S-401-1
System # / Name	к 1	K 2	К 3	K 4	K 5	К 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
209001 LPCS	1											 K1.12 (10CFR 55.41.5) Knowledge of the physical connections and/or cause-effect relationships between the LOW PRESSURE CORE SPRAY (LPCS) SYSTEM and the following: ECCS room coolers 	2.9	32
209002 HPCS		1	-	-	-	1	-	+	-	-	-	N/A for BFN - No HPCS System		
211000 SLC										1		 A4.08 (10CFR 55.41.7) Ability to manually operate and/or monitor in the control room: ♦ System initiation: Plant-Specific 	4.2	33
212000 RPS			1					1				 K3.11 (10CFR 55.41.7) Knowledge of the effect that a loss or malfunction of the REACTOR PROTECTION SYSTEM (RPS) will have on the following: Recirculation system 	3.0	34
												A2.20 (10CFR 55.43.5 - SRO Only) Ability to (a) predict the impacts of the following on the REACTOR PROTECTION SYSTEM (RPS); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: • Full system activation (full-SCRAM)	4.2	87
215003 IRM				1								 K4.04 (10CFR 55.41.7) Knowledge of INTERMEDIATE RANGE MONITOR (IRM) SYSTEM design feature(s) and/or interlocks which provide for the following: Varying system sensitivity levels using range switches 	2.9	35

SRO only K/As shown in italics

ES-401										l Plant	3WR Syst	Examination Outline ems – Tier 2/Group 1 (RO / SRO)	Form ES	6-401-1
System # / Name	К 1	K 2	К 3	K 4	К 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
215004 Source Range Monitor									1			 A3.01 (10CFR 55.41.7) Ability to monitor automatic operations of the SOURCE RANGE MONITOR (SRM) SYSTEM including: Meters and recorders 	3.2	36
215005 APRM / LPRM								1				 A2.07 (10CFR 55.41.5) Ability to (a) predict the impacts of the following on the AVERAGE POWER RANGE MONITOR / LOCAL POWER RANGE MONITOR (APRM / LPRM) SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Recirculation flow channels flow mismatch 	3.2	37

ES-401										Plant	BWR Syst	Examination Outline ems – Tier 2/Group 1 (RO / SRO)	Form E	S-401-1
System # / Name	к 1	K 2	К 3	К 4	K 5	К 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
217000 RCIC			2								1	 K3.02 (10CFR 55.41.7) Knowledge of the effect that a loss or malfunction of the REACTOR CORE ISOLATION COOLING (RCIC) SYSTEM will have on the following: ◆ Reactor vessel pressure 	3.6	38
												 K3.04 (10CFR 55.41.7) (Unit Differences) Knowledge of the effect that a loss or malfunction of the REACTOR CORE ISOLATION COOLING (RCIC) SYSTEM will have on the following: ♦ Adequate core cooling 	3.6	39
												G2.4.41 (10CFR 55.43.5 - SRO Only) Knowledge of the emergency action level thresholds and classifications.	4.6	88
218000 ADS	1											 K1.02 (10CFR 55.41.5) Knowledge of the physical connections and/or cause-effect relationships between the AUTOMATIC DEPRESSURIZATION SYSTEM (ADS) and the following: ♦ Low pressure core spray: Plant-Specific 	4.0	40

ES-401									<u> 1127</u>	l Plant	3WR Syste	Examination Outline ems – Tier 2/Group 1 (RO / SRO)	Form E	S-401-1
System # / Name	К 1	K 2	К 3	К 4	K 5	К 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
223002 PCIS/Nuclear Steam Supply Shutoff	2											K1.03 (10CFR 55.41.5) Knowledge of the physical connections and/or cause-effect relationships between the PRIMARY CONTAINMENT ISOLATION SYSTEM (PCIS) / NUCLEAR STEAM SUPPLY SHUT-OFF and the following:	3.0	41
			and a substant of the second o									 K1.13 (10CFR 55.41.7) Knowledge of the physical connections and/or cause-effect relationships between the PRIMARY CONTAINMENT ISOLATION SYSTEM (PCIS) / NUCLEAR STEAM SUPPLY SHUT-OFF and the following: Traversing in-core probe system 	2.7	42
239002 SRVs							1				1	 A1.01 (10CFR 55.41.5) Ability to predict and/or monitor changes in parameters associated with operating the RELIEF / SAFETY VALVES (SRVs) controls including: Tail pipe temperature 	3.3	43
											-	G2.4.45 (10CFR 55.43.5 - SRO Only) Ability to prioritize and interpret the significance of each annunciator or alarm.	4.3	89
259002 Reactor Water Level Control											1	G2.4.45 (10CFR 55.41.10) Ability to prioritize and interpret the significance of each annunciator or alarm.	4.1	44

ES-401			200				1.06.0			Plant	BWR Syst	Examination Outline ems – Tier 2/Group 1 (RO / SRO)	Form E	S-401-1
System # / Name	К 1	K 2	К 3	К 4	K 5	К 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
261000 SGTS						1		1				 K6.05 (10CFR 55.41.7) **Details in Record of Rejected K/As** Knowledge of the effect that a loss or malfunction of the following will have on the STANDBY GAS TREATMENT SYSTEM (SGTS): Reactor protection system: Plant-Specific 	3.1	45
				an and a substant and a substant and a substant a substant a substant a substant a substant a substant a substa								 A2.11 (10CFR 55.43.5 - SRO Only) Ability to (a) predict the impacts of the following on the STANDBY GAS TREATMENT SYSTEM (SGTS); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: High containment pressure 	3.3	90
262001 AC Electrical Distribution									1		1	 A3.04 (10CFR 55.41.7) **Details in Record of Rejected K/As** Ability to monitor automatic operations of the A.C. ELECTRICAL DISTRIBUTION SYSTEM including: Load sequencing 	3.4	46
												G2.2.39 (10CFR 55.41.10) Knowledge of less than or equal to one hour Technical Specification action statements for systems.	3.9	47

ES-401										Plant	BWR Syst	Examination Outline ems – Tier 2/Group 1 (RO / SRO)	Form E	S-401-1
System # / Name	К 1	K 2	К 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
262002 UPS (AC/DC)						2						 K6.02 (10CFR 55.41.7) Knowledge of the effect that a loss or malfunction of the following will have on the UNINTERRUPTABLE POWER SUPPLY (UPS) (A.C. / D.C.): D.C. electrical power 	2.8	48
												 K6.03 (10CFR 55.41.7) Knowledge of the effect that a loss or malfunction of the following will have on the UNINTERRUPTABLE POWER SUPPLY (UPS) (A.C. / D.C.): Static inverter 	2.7	49
263000 DC Electrical Distribution								1				 A2.01 (10CFR 55.41.5) Ability to (a) predict the impacts of the following on the D.C. ELECTRICAL DISTRIBUTION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Grounds 	2.8	50
264000 EDGs					1							 K5.05 (10CFR 55.41.5) Knowledge of the operational implications of the following concepts as they apply to EMERGENCY GENERATORS (DIESEL / JET): Paralleling A.C. power sources 	3.4	51
300000 Instrument Air		1										 K2.01 (10CFR 55.41.7) Knowledge of electrical power supplies to the following: Instrument air compressor 	2.8	52

SRO only K/As shown in italics

ES-401										Plant	BWR Syst	Examination Outline ems – Tier 2/Group 1 (RO / SRO)	Form E	S-401-1
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
400000 Component Cooling Water							1					 A1.01 (10CFR 55.41.5) Ability to predict and/or monitor changes in parameters associated with operating the COMPONENT COOLING WATER SYSTEM controls including: CCW flow rate 	2.8	53
K/A Category Point Totals: (RO)	4	2	3	2	2	3	2	2	2	2	2	Group Point Total: (RO)		26
K/A Category Point Totals: (SRO)								3			2	Group Point Total: (SRO)	1	5

ES-401									Pla	BV ant S	VR E ysten	xamination Outline ns – Tier 2/Group 2 (RO / SRO)	Form E	S-401-1
System # / Name	K 1	K 2	К 3	К 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
201001 CRD Hydraulic						1						 K6.04 (10CFR 55.41.7) Knowledge of the effect that a loss or malfunction of the following will have on the CONTROL ROD DRIVE HYDRAULIC SYSTEM: ♦ RPS 	3.6	54
201002 RMCS	-	-	-	-	-		-	-	-	-		NOT RANDOMLY SELECTED		
201003 Control Rod and Drive Mechanism	-	-	1	-	-	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
201004 RSCS	-	-	-	-		-	-	-	-	-		N/A for BFN - BWR 4/5 ONLY		
201005 RCIS	-		-	1	-	-	-	-	-	-	-	N/A for BFN - BWR 6 ONLY		
201006 RWM			-	-	-	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
202001 Recirculation								1	2			A2.10 (10CFR 55.43.5 - SRO Only) Ability to (a) predict the impacts of the following on the RECIRCULATION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: • Recirculation pump seal failure	3.9	91
202002 Recirculation Flow Control	-	-	-	-	-	-	1	-	-	-	++	NOT RANDOMLY SELECTED		
204000 RWCU	-	-	-	-	-	-	-	-	-		-	NOT RANDOMLY SELECTED		
214000-RPIS	-	+	-	+	+	-	+	-	-	-	-	NOT RANDOMLY SELECTED-		
215001 Traversing In-core Probe									1			 A3.03 (10CFR 55.41.7) Ability to monitor automatic operations of the TRAVERSING IN-CORE PROBE including: ◆ Valve operation: Not-BWR1 	2.5	55

SRO only K/As shown in italics

Form ES-401-1

ES-401 BWR Examination Outline For Plant Systems – Tier 2/Group 2 (RO / SRO)							Form E	m ES-401-1						
System # / Name	к 1	K 2	К 3	K 4	K 5	К 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
215002 RBM	1							1				 K1.02 (10CFR 55.41.6) Knowledge of the physical connections and/or cause-effect relationships between ROD BLOCK MONITOR SYSTEM and the following: LPRM: BWR-3,4,5 	3.2	56
												A2.05 (10CFR 55.43.5 - SRO Only) Ability to (a) predict the impacts of the following on the ROD BLOCK MONITOR SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: • RBM high or inoperable: BWR-3,4,5	3.3	92
216000 Nuclear Boiler Inst.	-	-	-	-			-	-	+		-	NOT RANDOMLY SELECTED-		
219000 RHR/LPCI: Torus/Pool Cooling Mode		1										 K2.02 (10CFR 55.41.7) Knowledge of electrical power supplies to the following: Pumps 	3.1	57
223001 Primary CTMT and Aux-		-	-	-	-	-		-	+	-	1	NOT RANDOMLY SELECTED		
226001 RHR/LPCI: CTMT Spray Mode	-	1	-	-	-	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
230000 RHR/LPCI: Torus/Peel Spray Mode	-	-	-	-	-	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
233000 Fuel Pool Cooling/Cleanup											1	G2.4.35 (10CFR 55.43.5 - SRO Only) Knowledge of local auxiliary operator tasks during an emergency and the resultant operational effects.	4.0	93
234000 Fuel Handling Equipment	-	I	-	-	-	-	-	-	1	-	-	NOT RANDOMLY SELECTED-		
239001 Main and Reheat Steam	-	1	-	-	+		1	-	-	-	+	NOT RANDOMLY SELECTED		

SRO only K/As shown in italics

ES-401 BWR Examination Outline For Plant Systems – Tier 2/Group 2 (RO / SRO)							Form E	S-401-1						
System # / Name	к 1	K 2	К 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
239003 MSIV Leakage Control				1								 K4.02 (10CFR 55.41.7) Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following: Performance of intended safety function following any single active component failure: BWR-4,5,6(P-Spec) 	3.0	58
241000 Reactor/Turbine Pressure Regulator							1					A1.01 (10CFR 55.41.5) **Details in Record of Rejected K/As** Ability to predict and/or monitor changes in parameters associated with operating the REACTOR / TURBINE PRESSURE REGULATING SYSTEM controls including: • Reactor pressure	3.9	59
245000 Main Turbine Gen. / Aux.					1							 K5.07 (10CFR 55.41.5) Knowledge of the operational implications of the following concepts as they apply to MAIN TURBINE GENERATOR AND AUXILIARY SYSTEMS: Generator operations and limitations 	2.6	60
256000 Reactor Condensate											1	G2.1.30 (10CFR 55.41.7) Ability to locate and operate components, including local controls.	4.4	61
259001-Reactor-Feedwater	1	-	-	-	-	-	1	-	-	-	I	NOT RANDOMLY SELECTED		
268000-Radwaste	-	+	-	+	-	-	-	+	-	-	-	NOT RANDOMLY SELECTED-		
271000 Offgas										1		 A4.06 (10CFR 55.41.7) Ability to manually operate and/or monitor in the control room: ♦ System indicating lights and alarms 	3.3	62

Form ES-401-1

ES-401									Pl	BV ant S	VR E ysten	xamination Outline ns – Tier 2/Group 2 (RO / SRO)	Form E	S-401-1
System # / Name	к 1	к 2	к 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
272000 Radiation Monitoring	-	-	-	-		-	-	-	-	-	-	NOT RANDOMLY SELECTED		
286000 Fire Protection		-	-		-	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
288000 Plant Ventilation								1				 A2.05 (10CFR 55.41.5) Ability to (a) predict the impacts of the following on the PLANT VENTILATION SYSTEMS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Extreme outside weather conditions: Plant-Specific 	2.6	63
290001 Secondary CTMT			1									 K3.01 (10CFR 55.41.7) Knowledge of the effect that a loss or malfunction of the SECONDARY CONTAINMENT will have on the following: Off-site radioactive release rates 	4.0	64
290003 Control Room HVAC	-	-	-	-	-	-	-	-	-	-	-	NOT RANDOMLY SELECTED		
290002 Reactor Vessel Internals	1											 K1.04 (10CFR 55.41.5) Knowledge of the physical connections and/or cause-effect relationships between REACTOR VESSEL INTERNALS and the following: HPCI: Plant-Specific 	3.4	65
K/A Category Point Totals: (RO)	2	1	1	1	1	1	1	1	1	1	1	Group Point Total: (RO)		12
K/A Category Point Totals: (SRO)								2			1	Group Point Total: (SRO)		3

Generic Knowledge and Abilities Outline (Tier 3)

0-1-1-1-1	-	Tanta	R	0	SRO	Onl
Category	N/A #	Горіс	IR	#	IR	#
ş	2.1.	G2.1.39 (10CFR 55.41.10) Knowledge of conservative decision making practices. G2.1.13 (10CFR 55.43.5 – SRO Only) Knowledge of facility requirements for controlling vital/controlled access.	3.6	66	3.2	9
1. Conduct of Operation	2.1.	 G2.1.6 (10CFR 55.41.10) Ability to manage the control room crew during transients. G2.1.4 (10CFR 55.43.2 – SRO Only) Knowledge of individual licensed operator responsibilities related to shift staffing, such as medical requirements, "no-solo" operation, maintenance of active license status, 10CFR55, etc. 	3.8	67	3.8	9
	2.1.	NOT RANDOMLY SELECTED				-
•	2.1.	NOT RANDOMLY SELECTED				
	2.1.	NOT RANDOMLY SELECTED				
	2.1.	NOT RANDOMLY SELECTED				
	Subtota	al		2		2
lo	2.2.	G2.2.20 (10CFR 55.41.10) Knowledge of the process for managing troubleshooting activities. G2.2.19 (10CFR 55.43.5 – SRO Only) Knowledge of maintenance work order requirements.	2.6	68	3.4	9
2. pment Cont	2.2.	 G2.2.22 (10CFR 55.41.5) Knowledge of limiting conditions for operations and safety limits. G2.2.23 (10CFR 55.43.2 – SRO Only) Ability to track Technical Specification limiting conditions for operations. 	4.0	69	4.6	9
inb	2.2.	NOT RANDOMLY SELECTED				
Щ	2.2.	NOT RANDOMLY SELECTED				
	2.2.	NOT RANDOMLY SELECTED				
	2.2.	NOT RANDOMLY SELECTED				
	Subtota	al		2		2

Generic Knowledge and Abilities Outline (Tier 3)

Catanani	WIA #	Tonio	R	0	SRO	Onl
Category	NA#	Торіс	IR	#	IR	#
	2.3	G2.3.14 (10CFR 55.41.12) Knowledge of radiation or contamination hazards that may arise during normal, abnormal, or emergency conditions or activities. G2.3.11 (10CFR 55.43.4 – SRO Only) Ability to control radiation releases.	3.4	70	4.3	98
3. Radiation Control	2.3	 G2.3.4 (10CFR 55.41.12) Knowledge of radiation exposure limits under normal or emergency conditions. G2.3.7 (10CFR 55.43.4/5 – SRO Only) Ability to comply with radiation work permit requirements during normal or abnormal conditions. 	3.2	71	3.6	99
	2.3	G2.3.7 (10CFR 55.41.12) Ability to comply with radiation work permit requirements during normal or abnormal conditions.	3.5	72	-	-
	2.3	NOT RANDOMLY SELECTED				
	2.3	NOT RANDOMLY SELECTED				
	2.3	NOT RANDOMLY SELECTED				
	Subtota	al		3		2
ss / Plan	2.4.	G2.4.18 (10CFR 55.41.10) Knowledge of the specific bases for EOPs. G2.4.8 (10CFR 55.43.5 – SRO Only) Knowledge of how abnormal operating procedures are used in conjunction with EOPs.	3.3	73	4.5	10
4. icy Procedures	2.4.	G2.4.26 (10CFR 55.41.10) Knowledge of facility protection requirements including fire brigade and portable fire fighting equipment usage.	3.1	74	-	11 - M
	2.4.	G2.4.6 (10CFR 55.41.10) Knowledge of EOP mitigation strategies.	3.7	75		F
ger	2.4.	NOT RANDOMLY SELECTED				
Jer	2.4.	NOT RANDOMLY SELECTED				
ш	2.4.	NOT RANDOMLY SELECTED				
	Subtota	al		3		1

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ES-401, Rev. 9

BWR Examination Outline

Tier Gr 1. Emergency & Abnormal Plant Evolutions Tier 1 2. Plant Systems Tier 3. Generic Knowled Catego 1. Ensur- and Sl in eac 2. The po The fin based 3. System not ap not inc	roup	K 1 4 2 6 4 2 6 Abilit	K 2 3 1 4 2 1 3	K 3 4 1 5 3 1 4	R 4 2 1 3	O K/ K 5 N/A 2 1 3	A Ca K 6	A 1 3 1 4 2	ory F A 2 1 3 2	Point A 3 N/	A 4 A	G * 4 1 5	Total 20 7	A	SRC 4 2	O-Only G	/ Poin i* 3	ts Total	
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Emergency & Abnormal Plant Evolutions Tier 1 2. Plant Systems Tier 3. Generic Knowled Catego 1. Ensur- and Sl in eac 2. The po The fir based 3. System not ap not inc	2 Totals 1 2 Totals 2 Totals 2 dge and <i>i</i> ories re that at le RO-only of	2 6 2 6 Abilit	1 2 1 3 ities	1 5 3 1 4	2 1 3	N/A 2 1 3	3	1 4 2 1	1 3 2	N /	A	1 5	7		2		4 3		
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2. Plant Systems 3. Generic Knowler Catego 1. Ensur- and Sl in eac 2. The po The fin based 3. System not ap not inc	1 2 Totals edge and <i>i</i> ories re that at legRO-only or	4 2 6 Abilit	2 1 3 ities	3 1 4	2 1 3	2 1 3	3	2	2	2			27		6		4	10	
2. Plant Systems 3. Generic Knowled Catego 1. Ensur- and Sl in eac 2. The po The fin based 3. System not ap not inc	2 Totals edge and <i>i</i> ories re that at le sRO-only or	2 6 Abilit	1 3 ities	1 4	1 3	1 3	1	1	1 4 2 3 2 2 3 2 2 2 2 2 2 2 6									5	
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ES-401, R	EV 9		T10	31 BWR EXAMINATION OUTLINE	FORM ES-401-1
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC)	
295001AA1.	Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4	3.3	3.3	Ability to operate and / or monitor the following as they apply to (ABNORMAL PLANT EVOLUTION):(CFR: 41.7 / 45.5 / 45.6)	RPS
295003AK1.	Partial or Complete Loss of AC / 6	2.9	3.2	Knowledge of the operational implications of the following concepts as they apply to the (ABNORMAL PLANT EVOLUTION):(CFR: 41.8 to 41.10 / 45.3)	Under voltage/degraded voltage effects on electrical loads
295004AK3.	Partial or Total Loss of DC Pwr / 6	2.9	3.3	Knowledge of the reasons for the following responses as they apply to (ABNORMAL PLANT EVOLUTION):(CFR: 41.5 / 41.10 / 45.6 / 45.13)	Ground isolation/fault determination
295005G2.4	Main Turbine Generator Trip / 3	4.0	4.6	This is a Generic, no stem statement is associated.	Knowledge of the parameters and logic used to assess the status of safety functions
295006AK2.	SCRAM / 1	3.8	3.8	Knowledge of the interrelations between (ABNORMAL PLANT EVOLUTION) and the following:(CFR: 41.7 / 45.7 / 45.8)	Reactor water level control system
295016G2.4	Control Room Abandonment / 7	2.7	4.1	This is a Generic, no stem statement is associated.	Knowledge of events related to system operations/status that must be reported to internal orginizations or outside agencies.





T1G1 BWR EXAMINATION OUTLINE



IR K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G TOPIC: KA NAME / SAFETY FUNCTION: RO SRO 295018AA1. Partial or Total Loss of CCW / 8 3.3 3.4 System loads..... Ability to operate and / or monitor the following as they apply to (ABNORMAL PLANT EVOLUTION):(CFR: 41.7 / 45.5 / 45.6) 295019AK2. Partial or Total Loss of Inst. Air / 8 3.5 3.5 ADS: Plant-Specific..... Knowledge of the interrelations between (ABNORMAL PLANT EVOLUTION) and the following:(CFR: 41.7 / 45.7 / 45.8) (multi-unit) Ability to explain the variations in control 295021G2.2 Loss of Shutdown Cooling / 4 3.6 3.6 board layouts, systems, instrumentation and procedural This is a Generic, no stem statement is actions between units at a facility. associated. 295023AA1. Refueling Acc Cooling Mode / 8 3.4 3.7 Radiation monitoring equipment..... Ability to operate and / or monitor the following as they apply to (ABNORMAL PLANT EVOLUTION):(CFR: 41.7 / 45.5 / 45.6) 295024EA2. High Drywell Pressure / 5 3.8 3.8 Suppression pool level..... Ability to determine and interpret the following as they apply to (EMERGENCY PLANT EVOLUTION):(CFR: 41.10 / 43.5 / 45.13) 295025EK2. High Reactor Pressure / 3 3.9 4.1 ARI/RPT/ATWS: Plant-Specific..... Knowledge of the interrelations between (EMERGENCY PLANT EVOLUTION) and

the following:(CFR: 41.7 / 45.7 / 45.8)





T1G1 BWR EXAMINATION OUTLINE



NAME / SAFETY FUNCTION: IR K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G TOPIC: KA RO SRO 295027EK3. High Containment Temperature / 5 Containment spray: Plant-Specific..... 3.2 3.2 Knowledge of the reasons for the following responses as they apply to (EMERGENCY PLANT EVOLUTION):(CFR: 41.5 / 41.10 / 45.6 / 45.13) 295028EK1. High Drywell Temperature / 5 3.5 3.7 Reactor water level measurement..... Knowledge of the operational implications of the following concepts as they apply to the EMERGENCY PLANT EVOLUTION):(CFR: 41.8 to 41.10 / 45.3) 295030EK3. Low Suppression Pool Wtr Lvl / 5 3.6 3.7 RCIC operation: Plant-Specific..... Knowledge of the reasons for the following responses as they apply to (EMERGENCY PLANT EVOLUTION);(CFR: 41.5 / 41.10 / 45.6 / 45.13) 295031EA2. Beactor Low Water Level / 2 4.2 4.2 Reactor pressure..... Ability to determine and interpret the following as they apply to (EMERGENCY PLANT EVOLUTION):(CFR: 41.10 / 43.5 / 45.13) 295037G2.2 SCRAM Condition Present and Power 3.7 4.1 Knowledge of surveillance procedures. Above APRM Downscale or Unknown This is a Generic, no stem statement is /1 associated. 295038EK1. High Off-site Release Rate / 9 2.5 3.1 Biological effects of radioisotope ingestion..... Knowledge of the operational implications of the following concepts as they apply to the EMERGENCY PLANT EVOLUTION):(CFR: 41.8 to 41.10 / 45.3)







T1G1 BWR EXAMINATION OUTLINE

KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRO		
600000AK3.	Plant Fire On Site / 8	2.8	3.4	Knowledge of the reasons for the following responses as they apply to (ABNORMAL PLANT EVOLUTION):(CFR: 41.5 / 41.10 / 45.6 / 45.13)	Actions contained in the abnormal procedure for plant fire on site
700000AK1.	Generator Voltage and Electric Grid Distrurbancecs	3.3	3.5	Knowledge of the operational implications of the following concepts as they apply to the (ABNORMAL PLANT EVOLUTION):(CFR: 41.8 to 41.10 / 45.3)	Definition of the terms: volts, watts, amps, VARS, power factor







ES-401, R	EV 9		T10	32 BWR EXAMINATION OUTLINE	FORM ES-401-1
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC)	
295009AK2.	Low Reactor Water Level / 2	2.6	2.6	Knowledge of the interrelations between (ABNORMAL PLANT EVOLUTION) and the following:(CFR: 41.7 / 45.7 / 45.8)	Reactor water cleanup
295010G2.4	High Drywell Pressure / 5	4.6	4.4	This is a Generic, no stem statement is associated.	Ability to perform without reference to procedures those actions that require immediate operation of system components and controls.
295012AK1.	High Drywell Temperature / 5	3.3	3.5	Knowledge of the operational implications of the following concepts as they apply to the (ABNORMAL PLANT EVOLUTION):(CFR: 41.8 to 41.10 / 45.3)	Pressure/temperature relationship
295017AK3.	High Off-site Release Rate / 9	3.3	4.5	Knowledge of the reasons for the following responses as they apply to (ABNORMAL PLANT EVOLUTION):(CFR: 41.5 / 41.10 / 45.6 / 45.13)	Implementation of site emergency plan
295022AA2.	Loss of CRD Pumps / 1	3.1	3.2	Ability to determine and interpret the following as they apply to ABNORMAL PLANT EVOLUTION):(CFR: 41.10 / 43.5 / 45.13)	CRD mechanism temperatures
295029EA1.	High Suppression Pool Wtr Lvl / 5	3.4	3.5	Ability to operate and / or monitor the following as they apply to (EMERGENCY PLANT EVOLUTION):(CFR: 41.7 / 45.5 / 45.6)	RCIC: Plant-Specific







T1G2 BWR EXAMINATION OUTLINE

KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRO		
295036EK1.	Secondary Containment High Sump/Area Water Level / 5	2.9	3.1	Knowledge of the operational implications of the following concepts as they apply to the EMERGENCY PLANT EVOLUTION):(CFR: 41.8 to 41.10 / 45.3)	Radiation releases





T2G1 BWR EXAMINATION OUTLINE



KA NAME / SAFETY FUNCTION: IR K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G TOPIC: RO SRO 203000K5.0 **RHR/LPCI:** Injection Mode Testable check valve operation 2.7 2.9 Knowledge of the operational implications of the following concepts as they apply to the (SYSTEM):(CFR: 41.5 / 45.7) 205000K2.0 Shutdown Cooling 2.5 2.7 Motor operated valves Knowledge of electrical power supplies to the following:(CFR: 41.7) 206000K4.1 HPCI 3.1 3.3 Minimizing fission product concentration in the condensate storage tank (valve closures on system Knowledge of (SYSTEM) design feature(s) initiation): BWR-2,3,4(P-Spec) and or interlock(s) which provide for the following:(CFR: 41.7) 207000A4.0 Isolation (Emergency) Condenser 3 3.2 Primary and shell sidetemperatures: BWR-2,3 Ability to manually operate and/or monitor in the control room:(CFR: 41.7 / 45.5 to 45.8) 209001K1.1 LPCS ECCS room coolers 2.9 3.1 Knowledge of the physical connections and/or cause-effect relationships between (SYSTEM) and the following:(CFR: 41.2 to 41.9 / 45.7 to 45.8) 209002A3.0 HPCS 3.7 3.7 System flow: BWR-5,6 Ability to monitor automatic operations of the (SYSTEM) including:(CFR: 41.7 / 45.5) 211000A4.0 SLC System initiation: Plant-Specific 4.2 4.2 Ability to manually operate and/or monitor in the control room:(CFR: 41.7 / 45.5 to 45.8)







T2G1 BWR EXAMINATION OUTLINE

KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRO		
212000K3.1	RPS	3.0	3.3		Recirculation system
				Knowledge of the effect that a loss or malfunction of the (SYSTEM) will have on the following:(CFR: 41.7 / 45.6)	
215003K4.0	IRM	2.9	2.9		Varying system sensitivity levels using range switches
				Knowledge of (SYSTEM) design feature(s) and or interlock(s) which provide for the following:(CFR: 41.7)	
215004A3.0	Source Range Monitor	3.2	3.2		Meters and recorders
				Ability to monitor automatic operations of the (SYSTEM) including:(CFR: 41.7 / 45.5)	
215005A2.0	APRM / LPRM	3.2	3.4		Recirculation flow channels flow mismatch
				Ability to (a) predict the impacts of the following on the (SYSTEM) and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal operation:(CFR: 41.5 / 43.5 / 45.3 / 45.13)	
217000K3.0	RCIC	3.6	3.6		Adequate core cooling
				Knowledge of the effect that a loss or malfunction of the (SYSTEM) will have on the following:(CFR: 41.7 / 45.6)	
217000K3.0	RCIC	3.6	3.6		Reactor vessel pressure
				Knowledge of the effect that a loss or malfunction of the (SYSTEM) will have on the following:(CFR: 41.7 / 45.6)	



KA





V 9 T2G1 BWR EXAMINATION OUTLINE NAME / SAFETY FUNCTION: IR K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G TOPIC:

		RO	SRC	•	
218000K1.0	ADS	4.0	4.1	Knowledge of the physical connections and/or cause-effect relationships between (SYSTEM) and the following:(CFR: 41.2 to 41.9 / 45.7 to 45.8)	Low pressure core spray: Plant-Specific
223002K1.0	PCIS/Nuclear Steam Supply Shutoff	3.0	3.2	Knowledge of the physical connections and/or cause-effect relationships between (SYSTEM) and the following:(CFR: 41.2 to 41.9 / 45.7 to 45.8)	Plant ventilation
223002K1.1	PCIS/Nuclear Steam Supply Shutoff	2.7	2.9	Knowledge of the physical connections and/or cause-effect relationships between (SYSTEM) and the following:(CFR: 41.2 to 41.9 / 45.7 to 45.8)	Traversing in-core probe system
239002A1.0	SRVs	3.3	3.4	Ability to predict and/or monitor changes in parameters associated with operating the (SYSTEM) controls including:(CFR: 41.5 / 45.5)	Tail pipe temperature
259002G2.4	Reactor Water Level Control	4.1	4.3	This is a Generic, no stem statement is associated.	Ability to prioritize and interpret the significance of each annunciator or alarm.
261000K6.0	SGTS	3.1	3.3	Knowledge of the effect that a loss or malfunction of the following will have on the (SYSTEM):(CFR: 41.7/45.7)	Primary containment high pressure: Plant-Specific

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ES-401, REV 9 **T2G1 BWR EXAMINATION OUTLINE** TOPIC: NAME / SAFETY FUNCTION: IR K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G KA RO SRO CCW flow rate 400000A1.0 Component Cooling Water 2.8 2.8 Ability to predict and/or monitor changes in parameters associated with operating the (SYSTEM) controls including:(CFR: 41.5 / 45.5)







ES-401, REV 9			T2G	2 BWR EXAMINATION OUTLINE	FORM ES-401-	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
201001K6.0	CRD Hydraulic	3.6	3.7		RPS	
				Knowledge of the effect that a loss or malfunction of the following will have on the (SYSTEM):(CFR: 41.7 / 45.7)		
201004A1.0	RSCS	3.3	3.3		Reactor manual control system: BWR-4,5	
				Ability to predict and/or monitor changes in parameters associated with operating the (SYSTEM) controls including:(CFR: 41.5 / 45.5)		
215001A3.0	Traversing In-core Probe	2.5	2.6		Valve operation: Not-BWR1	
				Ability to monitor automatic operations of the (SYSTEM) including:(CFR: 41.7 / 45.5)		
215002K1.0	RBM	3.2	3.1		LPRM: BWR-3,4,5	
				Knowledge of the physical connections and/or cause-effect relationships between (SYSTEM) and the following:(CFR: 41.2 to 41.9 / 45.7 to 45.8)		
219000K2.0	RHR/LPCI: Torus/Pool Cooling Mode	3.1	3.3		Pumps	
				Knowledge of electrical power supplies to the following:(CFR: 41.7)		
239003K4.0	MSIV Leakage Control	3	3.4		Performance of intended safety function following any	
				Knowledge of (SYSTEM) design feature(s) and or interlock(s) which provide for the following:(CFR: 41.7)	single active component failure: BWR-4,5,6(P-Spec)	
245000K5.0	Main Turbine Gen. / Aux.	2.6	2.9		Generator operations and limitations	
				Knowledge of the operational implications of the following concepts as they apply to the (SYSTEM):(CFR: 41.5 / 45.7)		







K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G NAME / SAFETY FUNCTION: IR TOPIC: KA RO SRO 256000G2.1 **Reactor Condensate** 4.4 4.0 Ability to locate and operate components, including local controls. This is a Generic, no stem statement is associated. 271000A4.0 Offgas 3.3 3.2 System indicating lights and alarms Ability to manually operate and/or monitor in the control room: (CFR: 41.7 / 45.5 to 45.8) 288000A2.0 **Plant Ventilation** 2.6 2.7 Extreme outside weather conditions: Plant-Specific Ability to (a) predict the impacts of the following on the (SYSTEM) and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal operation:(CFR: 41.5 / 43.5 / 45.3 / 45.13) 290001K3.0 Secondary CTMT 4.0 4.4 Off-site radioactive release rates Knowledge of the effect that a loss or malfunction of the (SYSTEM) will have on the following:(CFR: 41.7 / 45.6) 290002K1.0 **Reactor Vessel Internals** 3.4 3.5 **HPCI: Plant-Specific** Knowledge of the physical connections and/or cause-effect relationships between (SYSTEM) and the following:(CFR: 41.2 to 41.9 / 45.7 to 45.8)







ES-401, REV 9			ТЗ	BWR EXAMINATION OUTLINE	FORM ES-401-	
KA	NAME / SAFETY FUNCTION:	I	R	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
G2.1.39	Conduct of operations	3.6	4.3		Knowledge of conservative decision making practices	
G2.1.6	Conduct of operations	3.8	4.8		Ability to manage the control room crew during plant transients.	
G2.2.20	Equipment Control	2.6	3.8		Knowledge of the process for managing troubleshooting activities.	
G2.2.22	Equipment Control	4.0	4.7		Knowledge of limiting conditions for operations and safety limits.	
62.2.14	Padiation Control		20		Knowledge of rediction or contamination beyonds that	
G2.0.14		3.4	3.0		may arise during normal, abnormal, or emergency conditions or activities	
G2.3.4	Radiation Control	3.2	3.7		Knowledge of radiation exposure limits under normal and emergency conditions	
G2.3.7	Radiation Control	3.5	3.6		Ability to comply with radiation work permit requirements during normal or abnormal conditions	







ES-401, REV 9			ТЗ	BWR EXAMINATION OUTLINE	FORM ES-401-1
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC)	
G2.4.18	Emergency Procedures/Plans	3.3	4.0		Knowledge of the specific bases for EOPs.
G2.4.26	Emergency Procedures/Plans	3.1	3.6		Knowledge of facility protection requirements including fire brigade and portable fire fighting equipment usage.
6246	Emergency Procedures/Plans	37	47		Knowledge symptom based EOP mitigation strategies
G2.4.0	Emergency Frocedures/Plans	3.7	4./		Knowledge symptom based EOP miligation strategies.





SRO T1G1 BWR EXAMINATION OUTLINE



ES-401, REV 9 IR K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G TOPIC: KA NAME / SAFETY FUNCTION: RO SRO 295003G2.4.50 Partial or Complete Loss of AC / 6 4.2 4.0 Ability to verify system alarm setpoints and operate controls identified in the alarm response manual. This is a Generic, no stem statement is associated. Control Room Abandonment / 7 295016G2.4.3 3.7 3.9 Ability to identify post-accident instrumentation. This is a Generic, no stem statement is associated. 295018AA2.01 Partial or Total Loss of CCW / 8 3.3 3.4 Component temperatures..... Ability to determine and interpret the following as they apply to ABNORMAL PLANT EVOLUTION):(CFR: 41.10 / 43.5 / 45.13) 295023AA2.01 Refueling Acc Cooling Mode / 8 3.6 4.0 Area radiation levels..... Ability to determine and interpret the following as they apply to ABNORMAL PLANT EVOLUTION):(CFR: 41.10 / 43.5 / 45.13) 295025EA2.01 High Reactor Pressure / 3 4.3 4.3 Reactor pressure..... Ability to determine and interpret the following as they apply to (EMERGENCY PLANT EVOLUTION):(CFR: 41.10 / 43.5 / 45.13) 295027EA2.01 High Containment Temperature / 5 3.7 3.7 Containment temperature: Mark-III..... Ability to determine and interpret the following as they apply to (EMERGENCY PLANT EVOLUTION):(CFR: 41.10 / 43.5 / 45.13)







ES-401, REV 9 SRO T1G1 BW

SRO T1G1 BWR EXAMINATION OUTLINE

KA	NAME / SAFETY FUNCTION:	I	R	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRO		
700000G2.4.31	Generator Voltage and Electric Grid Distrurbancecs	4.2	4.1	This is a Generic, no stem statement is associated.	Knowledge of annunciators alarms, indications or response procedures







ES-401, REV 9			RO T	1G2 BWR EXAMINATION OUTLINE	FORM ES-401	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRO)		
295012G2.4.34	High Drywell Temperature / 5	4.2	4.1	This is a Generic, no stem statement is associated.	Knowledge of RO tasks performed outside the main control room during an emergency and the resultant operational effects	
295013AA2.02	High Suppression Pool Temp. / 5	3.2	3.5	Ability to determine and interpret the following as they apply to ABNORMAL PLANT EVOLUTION):(CFR: 41.10 / 43.5 / 45.13)	Localized heating/stratification	
295033EA2.03	High Secondary Containment Area Radiation Levels / 9	3.7	4.2	Ability to determine and interpret the following as they apply to (EMERGENCY PLANT EVOLUTION):(CFR: 41.10 / 43.5 / 45.13)	Cause of high area radiation	

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NAME / SAFETY FUNCTION:

KA



IR

SRO T2G1 BWR EXAMINATION OUTLINE

K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G

TOPIC:



RO SRO 206000A2.16 HPCI High drywell pressure: BWR-2,3,4 4 4.1 Ability to (a) predict the impacts of the following on the (SYSTEM) and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal operation:(CFR: 41.5 / 43.5 / 45.3 / 45.13) 212000A2.20 RPS 4.1 4.2 Full system activation (full-SCRAM) Ability to (a) predict the impacts of the following on the (SYSTEM) and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal operation:(CFR: 41.5 / 43.5 / 45.3 / 45.13) 217000G2.4.41 RCIC 2.9 4.6 Knowledge of the emergency action level thresholds and classifications. This is a Generic, no stem statement is associated. 239002G2.4.45 SRVs Ability to prioritize and interpret the significance of each 4.1 4.3 annunciator or alarm. This is a Generic, no stem statement is associated. 261000A2.11 SGTS 3.2 3.3 High containment pressure Ability to (a) predict the impacts of the following on the (SYSTEM) and (b) based on those predictions, use procedures to correct, control, or mitigate the

consequences of those abnormal operation:(CFR: 41.5 / 43.5 / 45.3 / 45.13)









ES-401, REV 9			SRO	T3 BWR EXAMINATION OUTLINE	FORM ES-401	
KA	NAME / SAFETY FUNCTION:	RO	IR SRC	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
G2.1.13	Conduct of operations	2.5	3.2		Knowledge of facility requirements for controlling vital / controlled access.	
G2.1.4	Conduct of operations	3.3	3.8		Knowledge of individual licensed operator responsibilities related to shift staffing, such as medical requirements, "no-solo" operation, maintenance of active license statur, 10CFR55 etc.	
G2.2.19	Equipment Control	2.3	3.4		Knowledge of maintenance work order requirements.	
G2.2.23	Equipment Control	3.1	4.6		Ability to track Technical Specification limiting conditions for operations.	
G2.3.11	Radiation Control	3.8	4.3		Ability to control radiation releases	
G2.3.7	Radiation Control	3.5	3.6		Ability to comply with radiation work permit requirements during normal or abnormal conditions	
G2.4.8	Emergency Procedures/Plans	3.8	4.5		Knowledge of how abnormal operating procedures are used in conjunction with EOPs.	