



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 30, 2009

Mr. J. R. Morris
Site Vice President
Catawba Nuclear Station
Duke Energy Carolinas, LLC
4800 Concord Road
York, SC 29745

SUBJECT: CATAWBA NUCLEAR STATION, UNITS 1 AND 2, ISSUANCE OF
AMENDMENTS REGARDING CHANGES TO THE LICENSES AND
TECHNICAL SPECIFICATIONS (TAC NOS. MD9871 AND MD9872)

Dear Mr. Morris:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 253 to Renewed Facility Operating License NPF-35 and Amendment No. 248 to Renewed Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2, respectively. The amendments consist of changes to the licenses and Technical Specifications (TSs) in response to your application dated October 8, 2008, as supplemented by letter dated May 5, 2009.

The amendments would revise the licenses and TSs by removing and updating portions of the licenses and TSs which are out of date or are obsolete including footnotes and references.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

If you have any questions, please call me at 301-415-1119.

Sincerely,

A handwritten signature in black ink that reads "Jon Thompson".

Jon Thompson, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures:

1. Amendment No. 253 to NPF-35
2. Amendment No. 248 to NPF-52
3. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CAROLINAS, LLC

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 253
Renewed License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Renewed Facility Operating License No. NPF-35 filed by the Duke Energy Carolinas, LLC, acting for itself, and North Carolina Electric Membership Corporation (licensees), dated October 8, 2008, as supplemented by letter dated May 5, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

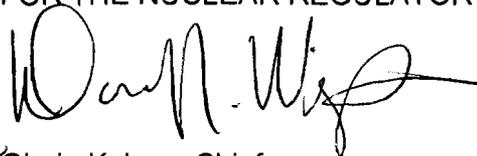
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-35 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 253 , which are attached hereto, are hereby incorporated into this license. Duke Energy Carolinas, LLC, shall operate the facility in accordance with the Technical Specifications.

3. Furthermore, this amendment makes a change to update a reference in NPF-35, Appendix B, page 1.
4. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for Gloria Kulesa, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to License No. NPF-35
and the Technical Specifications

Date of Issuance: October 30, 2009



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CAROLINAS, LLC

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 248
Renewed License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Renewed Facility Operating License No. NPF-52 filed by the Duke Energy Carolinas, LLC, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees), dated October 8, 2008, as supplemented by letter dated May 5, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

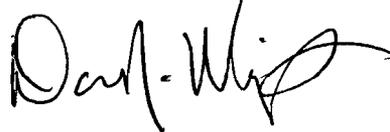
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 248 , which are attached hereto, are hereby incorporated into this license. Duke Energy Carolinas, LLC, shall operate the facility in accordance with the Technical Specifications.

3. Furthermore, this amendment makes a change to update a reference in NPF-52, Appendix B, page 1.
4. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for Gloria Kulesa, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to License No. NPF-52
and the Technical Specifications

Date of Issuance: October 30, 2009

ATTACHMENT TO
LICENSE AMENDMENT NO. 253
RENEWED FACILITY OPERATING LICENSE NO. NPF-35
DOCKET NO. 50-413
AND LICENSE AMENDMENT NO. 248
RENEWED FACILITY OPERATING LICENSE NO. NPF-52
DOCKET NO. 50-414

Replace the following pages of the Renewed Facility Operating Licenses (Licenses), the Appendix A Technical Specifications (TSs), and the Appendix B Additional Conditions with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Licenses

NPF-35, page 4
 NPF-35, page 5
 NPF-35, Appendix B, page 1
 NPF-52, page 4
 NPF-52, page 5
 NPF-52, Appendix B, page 1

TSs

3.5.2-1
 3.6.6-1
 3.6.9-1
 3.6.9-2
 3.6.17-1
 3.7.5-1
 3.7.7-1
 3.7.12-1
 3.8.1-1
 3.8.1-3
 3.8.4-2
 3.8.4-3
 3.8.6-1
 3.8.6-2
 3.8.6-3
 3.8.6-4
 5.2-1
 5.6-2

Insert

Licenses

NPF-35, page 4
 NPF-35, page 5
 NPF-35, Appendix B, page 1
 NPF-52, page 4
 NPF-52, page 5
 NPF-52, Appendix B, page 1

TSs

3.5.2-1
 3.6.6-1
 3.6.9-1
 3.6.9-2
 3.6.17-1
 3.7.5-1
 3.7.7-1
 3.7.12-1
 3.8.1-1
 3.8.1-3
 3.8.4-2
 3.8.4-3
 3.8.6-1
 3.8.6-2
 3.8.6-3
 3.8.6-4
 5.2-1
 5.6-2

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 253, which are attached hereto, are hereby incorporated into this renewed operating license. Duke Energy Carolinas, LLC shall operate the facility in accordance with the Technical Specifications.

(3) Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than December 6, 2024, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

(4) Antitrust Conditions

Duke Energy Carolinas, LLC shall comply with the antitrust conditions delineated in Appendix C to this renewed operating license.

(5) Fire Protection Program (Section 9.5.1, SER, SSER #2, SSER #3, SSER #4, SSER #5)*

Duke Energy Carolinas, LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report, as amended, for the facility and as approved in the SER through Supplement 5, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

*The parenthetical notation following the title of this renewed operating license condition denotes the section of the Safety Evaluation Report and/or its supplement wherein this renewed license condition is discussed.

(6) Mitigation Strategies

Develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

- (a) Fire fighting response strategy with the following elements:
 - 1. Pre-defined coordinated fire response strategy and guidance
 - 2. Assessment of mutual aid fire fighting assets
 - 3. Designated staging areas for equipment and materials
 - 4. Command and control
 - 5. Training of response personnel

- (b) Operations to mitigate fuel damage considering the following:
 - 1. Protection and use of personnel assets
 - 2. Communications
 - 3. Minimizing fire spread
 - 4. Procedures for implementing integrated fire response strategy
 - 5. Identification of readily-available pre-staged equipment
 - 6. Training on integrated fire response strategy
 - 7. Spent fuel pool mitigation measures

- (c) Actions to minimize release to include consideration of:
 - 1. Water spray scrubbing
 - 2. Dose to onsite responders

(7) Additional Conditions

The Additional Conditions contained in Appendix B, as revised through Amendment No. 253 are hereby incorporated into this renewed operating license. Duke Energy Carolinas, LLC shall operate the facility in accordance with the Additional Conditions.

- D. The facility requires exemptions from certain requirements of Appendix J to 10 CFR Part 50, as delineated below and pursuant to evaluations contained in the referenced SER and SSERs. These include, (a) partial exemption from the requirement of paragraph III.D.2(b)(ii) of Appendix J, the testing of containment airlocks at times when the containment integrity is not required (Section 6.2.6 of the SER, and SSERs # 3 and #4), (b) exemption from the requirement of paragraph III.A.(d) of Appendix J, insofar as it requires the venting and draining of lines for type A tests (Section 6.2.6 of SSER #3), and (c) partial exemption from the requirements of paragraph III.B of Appendix J, as it relates to bellows testing (Section 6.2.6 of the SER and SSER #3). These exemptions are authorized by law, will not present an undue risk to the public health and safety, are consistent with the common defense and security, and are consistent with certain special circumstances as discussed in the referenced SER and SSERs. These exemptions are, therefore, hereby granted pursuant to 10 CFR 50.12. With the granting of these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

APPENDIX B

ADDITIONAL CONDITIONS

FACILITY OPERATING LICENSE NO. NPF-35

Duke Energy Carolinas, LLC shall comply with the following conditions on the schedules noted below:

<u>Amendment Number</u>	<u>Additional Condition</u>	<u>Implementation Date</u>
159	This amendment requires the licensee to use administrative controls, as described in the licensee's letter of March 7, 1997, and evaluated in the staff's safety evaluation dated April 29, 1997, to restrict the dose-equivalent iodine levels to 0.46 microCurie per gram (in lieu of the limit in TS Section 3.4.16.a), and to 26 microCurie per gram (in lieu of the limit of TS Figure 3.4.16-1), until this license condition is removed by a future amendment.	Immediately upon issuance of the amendment
173	The licensee is authorized to relocate certain requirements included in appendix A to licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the appropriate documents as described in the licensee's letters dated May 27, 1997, as amended by letters dated March 9, March 20, April 20, June 3, June 24, July 7, July 21, August 5, September 8, and September 15, 1998, and evaluated in the NRC staff's Safety Evaluation associated with this amendment.	All relocation to be completed by January 31, 1999.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 248, which are attached hereto, are hereby incorporated into this renewed operating license. Duke Energy Carolinas, LLC shall operate the facility in accordance with the Technical Specifications.

(3) Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than February 24, 2026, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

(4) Antitrust Conditions

Duke Energy Carolinas, LLC shall comply with the antitrust conditions delineated in Appendix C to this renewed operating license.

(5) Fire Protection Program (Section 9.5.1, SER, SSER #2, SSER #3, SSER #4, SSER #5)*

Duke Energy Carolinas, LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report, as amended, for the facility and as approved in the SER through Supplement 5, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

*The parenthetical notation following the title of this renewed operating license condition denotes the section of the Safety Evaluation Report and/or its supplements wherein this renewed license condition is discussed.

(6) Mitigation Strategies

Develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

- (a) Fire fighting response strategy with the following elements:
 - 1. Pre-defined coordinated fire response strategy and guidance
 - 2. Assessment of mutual aid fire fighting assets
 - 3. Designated staging areas for equipment and materials
 - 4. Command and control
 - 5. Training of response personnel

- (b) Operations to mitigate fuel damage considering the following:
 - 1. Protection and use of personnel assets
 - 2. Communications
 - 3. Minimizing fire spread
 - 4. Procedures for implementing integrated fire response strategy
 - 5. Identification of readily-available pre-staged equipment
 - 6. Training on integrated fire response strategy
 - 7. Spent fuel pool mitigation measures

- (c) Actions to minimize release to include consideration of:
 - 1. Water spray scrubbing
 - 2. Dose to onsite responders

(7) Additional Conditions

The Additional Conditions contained in Appendix B, as revised through Amendment No. 248 are hereby incorporated into this renewed operating license. Duke Energy Carolinas, LLC shall operate the facility in accordance with the Additional Conditions.

- D. The facility requires exemptions from certain requirements of Appendix J to 10 CFR Part 50, as delineated below and pursuant to evaluations contained in the referenced SER and SSERs. These include, (a) partial exemption from the requirement of paragraph III.D.2(b)(ii) of Appendix J, the testing of containment airlocks at times when the containment integrity is not required (Section 6.2.6 of the SER, and SSERs # 3 and #4), (b) exemption from the requirement of paragraph III.A.(d) of Appendix J, insofar as it requires the venting and draining of lines for type A tests (Section 6.2.6 of SSER #3), and (c) partial exemption from the requirements of paragraph III.B of Appendix J, as it relates to bellows testing (Section 6.2.6 of the SER and SSER #3). These exemptions are authorized by law, will not present an undue risk to the public health and safety, are consistent

APPENDIX B

ADDITIONAL CONDITIONS

FACILITY OPERATING LICENSE NO. NPF-52

Duke Energy Carolinas, LLC shall comply with the following conditions on the schedules noted below:

<u>Amendment Number</u>	<u>Additional Condition</u>	<u>Implementation Date</u>
151	This amendment requires the licensee to use administrative controls, as described in the licensee's letter of March 7, 1997, and evaluated in the staff's safety evaluation dated April 29, 1997, to restrict the dose-equivalent iodine levels to 0.46 microCurie per gram (in lieu of the limit in TS Section 3.4.16.a), and to 26 microCurie per gram (in lieu of the limit of TS Figure 3.4.16-1), until this license condition is removed by a future amendment.	Immediately upon issuance of the amendment.
165	The licensee is authorized to relocate certain requirements included in appendix A to licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the appropriate documents, as described in the licensee's letters dated May 27, 1997, as amended by letters dated March 9, March 20, April 20, June 3, June 24, July 7, July 21, August 5, September 8, and September 15, 1998, and evaluated in the NRC staff's Safety Evaluation associated with this amendment.	All relocation to be completed by January 31, 1999.

3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)

3.5.2 ECCS — Operating

LCO 3.5.2 Two ECCS trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

-----NOTE-----
 In MODE 3, both safety injection (SI) pump flow paths may be isolated by closing the isolation valves for up to 2 hours to perform pressure isolation valve testing per SR 3.4.14.1.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more trains inoperable. <u>AND</u> At least 100% of the ECCS flow equivalent to a single OPERABLE ECCS train available.	A.1 Restore train(s) to OPERABLE status.	72 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3. <u>AND</u>	6 hours
	B.2 Be in MODE 4.	12 hours

3.6 CONTAINMENT SYSTEMS

3.6.6 Containment Spray System

LCO 3.6.6 Two containment spray trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One containment spray train inoperable.	A.1 Restore containment spray train to OPERABLE status.	72 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	84 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.6.6.1 Verify each containment spray manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in the correct position.	31 days

(continued)

3.6 CONTAINMENT SYSTEMS

3.6.9 Hydrogen Ignition System (HIS)

LCO 3.6.9 Two HIS trains shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One HIS train inoperable.	A.1 Restore HIS train to OPERABLE status.	7 days
	<u>OR</u> A.2 Perform SR 3.6.9.1 on the OPERABLE train.	Once per 7 days
B. One containment region with no OPERABLE hydrogen ignitor.	B.1 Restore one hydrogen ignitor in the affected containment region to OPERABLE status.	7 days
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3.	6 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.6.9.1 Energize each HIS train power supply breaker and verify ≥ 34 ignitors are energized in each train.	92 days
SR 3.6.9.2 Verify at least one hydrogen ignitor is OPERABLE in each containment region.	92 days
SR 3.6.9.3 Energize each hydrogen ignitor and verify temperature is $\geq 1700^{\circ}\text{F}$.	18 months

3.6 CONTAINMENT SYSTEMS

3.6.17 Containment Valve Injection Water System (CVIWS)

LCO 3.6.17 Two CVIWS trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One CVIWS train inoperable.	A.1 Restore CVIWS train to OPERABLE status.	7 days
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.6.17.1 Verify system surge tanks pressure is ≥ 36.4 psig.	31 days
SR 3.6.17.2 Verify valve injection flow rate is < 1.29 gpm (Unit 1) < 1.21 gpm (Unit 2) for Train A and < 1.16 gpm for Train B with a surge tank pressure ≥ 36.4 psig.	18 months
SR 3.6.17.3 Verify each automatic valve actuates to its correct position on an actual or simulated actuation signal.	18 months

3.7 PLANT SYSTEMS

3.7.5 Auxiliary Feedwater (AFW) System

LCO 3.7.5 Three AFW trains shall be OPERABLE.

-----NOTE-----
Only one AFW train, which includes a motor driven pump, is required to be OPERABLE in MODE 4.

APPLICABILITY: MODES 1, 2, and 3,
MODE 4 when steam generator is relied upon for heat removal.

ACTIONS

-----NOTE-----
LCO 3.0.4.b is not applicable when entering MODE 1.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One steam supply to turbine driven AFW pump inoperable.	A.1 Restore steam supply to OPERABLE status.	7 days <u>AND</u> 10 days from discovery of failure to meet the LCO
B. One AFW train inoperable in MODE 1, 2 or 3 for reasons other than Condition A.	B.1 Restore AFW train to OPERABLE status.	72 hours <u>AND</u> 10 days from discovery of failure to meet the LCO

(continued)

3.7 PLANT SYSTEMS

3.7.7 Component Cooling Water (CCW) System

LCO 3.7.7 Two CCW trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. One CCW train inoperable.</p>	<p>A.1 -----NOTE----- Enter applicable Conditions and Required Actions of LCO 3.4.6, "RCS Loops—MODE 4," for residual heat removal loops made inoperable by CCW. ----- Restore CCW train to OPERABLE status.</p>	<p>72 hours</p>
<p>B. Required Action and associated Completion Time of Condition A not met.</p>	<p>B.1 Be in MODE 3. <u>AND</u> B.2 Be in MODE 5.</p>	<p>6 hours 36 hours</p>

3.7 PLANT SYSTEMS

3.7.12 Auxiliary Building Filtered Ventilation Exhaust System (ABFVES)

LCO 3.7.12 Two ABFVES trains shall be OPERABLE.

-----NOTE-----
The ECCS pump rooms pressure boundary may be opened intermittently under administrative controls.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One ABFVES train inoperable.	A.1 Restore ABFVES train to OPERABLE status.	7 days
B. Two ABFVES trains inoperable due to inoperable ECCS pump rooms pressure boundary.	B.1 Restore ECCS pump rooms pressure boundary to OPERABLE status.	24 hours
C. Required Action and associated Completion Time of Condition A or B not met.	C.1 Be in MODE 3. <u>AND</u> C.2 Be in MODE 5.	6 hours 36 hours
D. One or more ABFVES train(s) heater inoperable.	D.1 Restore ABFVES train(s) heater to OPERABLE status. <u>OR</u> D.2 Initiate action in accordance with Specification 5.6.6.	7 days 7 days

3.8 ELECTRICAL POWER SYSTEMS

3.8.1 AC Sources—Operating

LCO 3.8.1 The following AC electrical sources shall be OPERABLE:

- a. Two qualified circuits between the offsite transmission network and the Onsite Essential Auxiliary Power System; and
- b. Two diesel generators (DGs) capable of supplying the Onsite Essential Auxiliary Power Systems;

AND

The automatic load sequencers for Train A and Train B shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

-----NOTE-----

LCO 3.0.4.b is not applicable to DGs.

CONDITION		REQUIRED ACTION	COMPLETION TIME
A.	One offsite circuit inoperable.	A.1 Perform SR 3.8.1.1 for OPERABLE offsite circuit.	1 hour
		<u>AND</u>	<u>AND</u>
		A.2 Declare required feature(s) with no offsite power available inoperable when its redundant required feature(s) is inoperable.	24 hours from discovery of no offsite power to one train concurrent with inoperability of redundant required feature(s)
		<u>AND</u>	(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. (continued)	B.4 Restore DG to OPERABLE status.	72 hours <u>AND</u> 6 days from discovery of failure to meet LCO
C. Two offsite circuits inoperable.	C.1 Declare required feature(s) inoperable when its redundant required feature(s) is inoperable. <u>AND</u> C.2 Restore one offsite circuit to OPERABLE status.	12 hours from discovery of Condition C concurrent with inoperability of redundant required features 24 hours

(continued)

ACTIONS (continued)

CONDITION		REQUIRED ACTION	COMPLETION TIME
D.	A and/or D channel of DC electrical power subsystem inoperable. <u>AND</u> Associated train of DG DC electrical power subsystem inoperable.	D.1 Enter applicable Condition(s) and Required Action(s) of LCO 3.8.9, "Distribution Systems-Operating", for the associated train of DC electrical power distribution subsystem made inoperable.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.8.4.1	Verify DC channel and DG battery terminal voltage is ≥ 125 V on float charge.	7 days
SR 3.8.4.2	Not used.	
SR 3.8.4.3	Verify no visible corrosion at the DC channel and DG battery terminals and connectors. <u>OR</u> Verify battery connection resistance of these items is $\leq 1.5 \text{ E-4}$ ohm.	92 days

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
SR 3.8.4.4 Verify DC channel and DG battery cells, cell plates, and racks show no visual indication of physical damage or abnormal deterioration that could degrade battery performance.	18 months
SR 3.8.4.5 Remove visible terminal corrosion, verify DC channel and DG battery cell to cell and terminal connections are clean and tight, and are coated with anti-corrosion material.	18 months
SR 3.8.4.6 Verify DC channel and DG battery connection resistance is $\leq 1.5 \text{ E-4 ohm}$.	18 months
SR 3.8.4.7 Verify each DC channel battery charger supplies ≥ 200 amps and the DG battery charger supplies ≥ 75 amps with each charger at $\geq 125 \text{ V}$ for ≥ 8 hours.	18 months
SR 3.8.4.8 -----NOTES----- 1. The modified performance discharge test in SR 3.8.4.9 may be performed in lieu of the service test in SR 3.8.4.8. 2. This Surveillance shall not be performed for the DG batteries in MODE 1, 2, 3, or 4. ----- Verify DC channel and DG battery capacity is adequate to supply, and maintain in OPERABLE status, the required emergency loads for the design duty cycle when subjected to a battery service test.	18 months

(continued)

3.8 ELECTRICAL POWER SYSTEMS

3.8.6 Battery Cell Parameters

- LCO 3.8.6
- a. Battery cell parameters for the channels of DC batteries shall be within the limits of Table 3.8.6-1; and
 - b. Battery cell parameters for the Diesel Generator (DG) Train A and Train B batteries shall be within the limits of Table 3.8.6-1.

APPLICABILITY: When associated DC electrical power subsystems are required to be OPERABLE.

ACTIONS

-----NOTE-----

Separate Condition entry is allowed for each battery.

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. One or more batteries (channel(s) of DC batteries, DG batteries, or both) with one or more battery cell parameters not within Category A or B limits.</p>	<p>A.1 Verify pilot cells electrolyte level and float voltage meet Table 3.8.6-1 Category C limits.</p>	<p>1 hour</p>
	<p><u>AND</u></p> <p>A.2 Verify battery cell parameters meet Table 3.8.6-1 Category C limits.</p>	<p>24 hours</p> <p><u>AND</u></p> <p>Once per 7 days thereafter</p>
	<p><u>AND</u></p> <p>A.3 Restore battery cell parameters to Category A and B limits of Table 3.8.6-1.</p>	<p>31 days</p>

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.8.6.1 Verify battery cell parameters of the channels of DC and DG batteries meet Table 3.8.6-1 Category A limits.	7 days
SR 3.8.6.2 Not used.	
SR 3.8.6.3 Verify battery cell parameters of the channels of DC and DG batteries meet Table 3.8.6-1 Category B limits.	92 days <u>AND</u> Once within 7 days after a battery discharge < 110 V <u>AND</u> Once within 7 days after a battery overcharge > 150 V
SR 3.8.6.4 Verify average electrolyte temperature for the channels of DC and DG batteries of representative cells is $\geq 60^{\circ}\text{F}$.	92 days

5.0 ADMINISTRATIVE CONTROLS

5.2 Organization

5.2.1 Onsite and Offsite Organizations

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be defined and established throughout highest management levels, intermediate levels, and all operating organization positions. These relationships shall be documented and updated, as appropriate, in organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the UFSAR;
- b. The Station Manager shall be responsible for overall safe operation of the plant and shall have control over those onsite activities necessary for safe operation and maintenance of the plant;
- c. The Vice President of Catawba Nuclear Site shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety;
- d. The Chief Nuclear Officer will be the Senior Nuclear Executive and have corporate responsibility for overall nuclear safety; and
- e. The individuals who train the operating staff, carry out radiation protection, or perform quality assurance functions may report to the appropriate onsite manager; however, these individuals shall have sufficient organizational freedom to ensure their independence from operating pressures.

5.2.2 Unit Staff

The unit staff organization shall include the following:

- a. A non-licensed operator shall be assigned to each reactor containing fuel and an additional non-licensed operator shall be assigned for each control room from which a reactor is operating in MODES 1, 2, 3, or 4.

A total of three non-licensed operators are required for the two units.

(continued)

5.6 Reporting Requirements (continued)

5.6.4 Not used.

5.6.5 CORE OPERATING LIMITS REPORT (COLR)

- a. Core operating limits shall be established prior to each reload cycle, or prior to any remaining portion of a reload cycle, and shall be documented in the COLR for the following:
1. Illustration of Reactor Core Safety Limits for Specification 2.1.1,
 2. Moderator Temperature Coefficient BOL and EOL limits and 60 ppm and 300 ppm surveillance limits for Specification 3.1.3,
 3. Shutdown Bank Insertion Limit for Specification 3.1.5,
 4. Control Bank Insertion Limits for Specification 3.1.6,
 5. Axial Flux Difference limits for Specification 3.2.3,
 6. Heat Flux Hot Channel Factor for Specification 3.2.1,
 7. Nuclear Enthalpy Rise Hot Channel Factor for Specification 3.2.2,
 8. Overtemperature and Overpower Delta T setpoint parameter values for Specification 3.3.1,
 9. Reactor Coolant System Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits for Specification 3.4.1,
 10. Accumulator and Refueling Water Storage Tank boron concentration limits for Specification 3.5.1 and 3.5.4,
 11. Reactor Coolant System and refueling canal boron concentration limits for Specification 3.9.1,
 12. Spent fuel pool boron concentration limits for Specification 3.7.15,
 13. SHUTDOWN MARGIN for Specification 3.1.1,
 14. 31 EFPD Surveillance Penalty Factors for Specifications 3.2.1 and 3.2.2, and
 15. Reactor Makeup Water Pumps Combined Flow Rates limit for Specification 3.3.9.

(continued)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO

AMENDMENT NO. 253 TO RENEWED FACILITY OPERATING LICENSE NPF-35

AND

AMENDMENT NO. 248 TO RENEWED FACILITY OPERATING LICENSE NPF-52

DUKE ENERGY CAROLINAS, LLC

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By application dated October 8, 2008 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML082880098), as supplemented by letter dated May 5, 2009 (ADAMS Accession No. ML091270418), Duke Energy Carolinas, LLC (Duke), requested changes to the Technical Specifications (TSs) and license conditions for the Catawba Nuclear Station, Units 1 and 2 (Catawba 1 and 2). The supplement dated May 5, 2009, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published the *Federal Register* on April 7, 2009 (74 FR 15769).

The amendments would revise the licenses and TSs by removing and updating portions of the licenses and TSs which are out of date or are obsolete, including footnotes and references, which reflect equipment that has been replaced, and which contain typographical errors or inconsistencies with other TSs.

2.0 REGULATORY EVALUATION

Pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Section 50.36, "Technical Specifications," the U.S. Nuclear Regulatory Commission (NRC) established its regulatory requirements related to the content of TSs. Pursuant to 10 CFR 50.36, TSs are required to include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operations (LCOs); (3) surveillance requirements; (4) design features; and (5) administrative controls. The rule does not specify the particular requirements to be included in a plant's TSs. The NRC staff examined the proposed changes with respect to the requirements of 10 CFR 50.36 and determined that these requirements were met.

3.0 TECHNICAL EVALUATION

Duke submitted this license amendment request (LAR) seeking the following changes to the following License Additional Conditions and TSs:

Catawba 1, Appendix B, "Additional Conditions," Amendment No. 159

Renewed Facility Operating License NPF-35 for Catawba 1, Appendix B, "Additional Conditions," Amendment Number 159, incorrectly cites "TS Section 3.4.8.a," which should be replaced with "TS Section 3.4.16.a," and "Figure 3.4-1," which should be replaced with "Figure 3.4.16-1."

The NRC staff examined this change to the license conditions. Renewed Facility Operating License NPF-35 for Catawba 1, Appendix B, "Additional Conditions," Amendment Number 159, incorrectly cites "TS Section 3.4.8.a," which does not exist (the nearest citation is "3.4.8 RCS Loops-MODE 5, Loops Not Filled"). This license condition was added to ensure that administrative controls described by the licensee in a letter dated March 7, 1997, were reflected in the license. The proposed change does not change the limitations described in this license condition. The proper citation should be "TS Section 3.4.16.a," for the condition of "DOSE EQUIVALENT, I-131 > 1.0 $\mu\text{Ci/gm}$." In this same license condition, the reference to "Figure 3.4-1," which does not exist, should be replaced with a reference to "Figure 3.4.16-1" which properly shows "Reactor Coolant DOSE EQUIVALENT 1-131 Specific Activity, Limit Versus Percent of RATED THERMAL POWER"

The NRC staff has reviewed this proposed change and approves it based on the fact that the uncorrected text references a section and a figure which do not exist and based on the fact that the corrections suggested properly identify the correct references relevant to the license condition.

Catawba 2, Appendix B, "Additional Conditions," Amendment No. 151

Renewed Facility Operating License NPF-52 for Catawba 2, Appendix B, "Additional Conditions," Amendment Number 151, incorrectly cites "TS Section 3.4.8.a," which should be replaced with "TS Section 3.4.16.a," and "Figure 3.4-1," which should be replaced with "Figure 3.4.16-1."

The NRC staff examined this change to the license conditions. Renewed Facility Operating License NPF-52 for Catawba 2, Appendix B, "Additional Conditions," Amendment Number 151, incorrectly cites "TS Section 3.4.8.a," which does not exist (the nearest citation is "3.4.8 RCS Loops-MODE 5, Loops Not Filled"). This license condition was added to ensure that administrative controls described by the licensee in a letter dated March 7, 1997, were reflected in the license. The proposed change does not change the limitations described in this license condition. The proper citation should be "TS Section 3.4.16.a," for the condition of "DOSE EQUIVALENT, I-131 > 1.0 $\mu\text{Ci/gm}$." In this same license condition, the reference to "Figure 3.4-1," which does not exist, should be replaced with a reference to "Figure 3.4.16-1" which properly shows "Reactor Coolant DOSE EQUIVALENT 1-131 Specific Activity, Limit Versus Percent of RATED THERMAL POWER"

The NRC staff has reviewed this proposed change and approves it based on the fact that the uncorrected text references a section and a figure which do not exist and based on the fact that the suggested corrections properly identify the correct references relevant to the license condition.

TS LCO 3.5.2 and associated footnote

LCO 3.5.2 on page 3.5.2-1 contains a footnote which allows extension of completion times for Required Action A.1 for the 1B Emergency Core Cooling System train during repair and restoration of the 1B centrifugal charging pump (CCP). This footnote and the associated markings will be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that the 1B CCP was repaired and restored and the footnote is no longer applicable under its own terms.

TS LCO 3.6.6 and associated footnote

LCO 3.6.6 on page 3.6.6-1 contains a footnote which allows extension of completion times for Required Action A.1 for one train of the Containment Spray system during Nuclear Service Water System (NSWS) upgrades. This footnote and the associated markings will be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that NSWS upgrades were completed and the footnote is no longer applicable under its own terms.

TS LCO 3.6.9 and associated footnote

LCO 3.6.9 on page 3.6.9-1 contains a footnote which allows each Hydrogen Ignition System train's ignitor located beneath the reactor vessel missile shield to be inoperable without entering into Condition A or B for Cycle 11 of Catawba 2, or until the next repair opportunity. This footnote and the associated markings will be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that Cycle 11 of Unit 2 has passed.

TS Surveillance Requirements (SRs) 3.6.9.1, 3.6.9.2, and 3.6.9.3 and associated footnote

SRs 3.6.9.1, 3.6.9.2, and 3.6.9.3 on page 3.6.9-2 contain footnotes which allow these SR to be not applicable for Cycle 11 of Catawba 2, or until the next repair opportunity. This footnote and the associated markings will be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that Cycle 11 of Catawba 2 has passed.

TS LCO 3.6.17 and associated footnote

LCO 3.6.17 on page 3.6.17-1 contains a footnote which allows extension of completion times for Required Action A.1 for one train of the Containment Valve Injection Water System during NSWS system upgrades. This footnote and the associated markings will be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that NSWS upgrades were completed and thus the footnote is no longer applicable.

TS LCO 3.7.5 and associated footnote

LCO 3.7.5 on page 3.7.5-1 contains a footnote which allows extension of completion times for Required Action B.1 for one train of the Auxiliary Feedwater System during NSW upgrades. This footnote and the associated markings will be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that NSW upgrades were completed and thus the footnote is no longer applicable.

TS LCO 3.7.7 and associated footnote

LCO 3.7.7 on page 3.7.7-1 contains a footnote which allows extension of completion times for Required Action A.1 for one train of the Component Cooling Water System during NSW upgrades. This footnote and the associated markings will be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that NSW upgrades were completed and thus this footnote is no longer applicable under its own terms.

TS LCO 3.7.10 and associated footnote

LCO 3.7.10 on page 3.7.10-1 contains a footnote which allows extension of completion times for Required Action A.1 for one train of the Control Room Area Ventilation System during NSW upgrades. This footnote and the associated markings will be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that NSW upgrades were completed and thus the footnote is no longer applicable under its own terms.

TS LCO 3.7.12 and associated footnote

LCO 3.7.12 on page 3.7.12-1 contains a footnote which allows extension of completion times for Required Action B.1 for two trains of the Auxiliary Building Filtered Ventilation Exhaust System during 1B CCP repair and restoration. This footnote and the associated markings will be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that 1B CCP repair and restoration is complete and thus the footnote is no longer applicable under its own terms.

TS LCO 3.8.1 and associated footnotes

LCO 3.8.1 on pages 3.8.1-1 and 3.8.1-3 contains a footnote which allows extension, during NSW upgrades, of completion times for Required Action B.4 for one emergency diesel generator to be inoperable. This footnote and the associated markings will be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that NSW upgrades were completed and thus the footnote is no longer applicable under its own terms.

TS SRs 3.8.4.2, 3.8.4.3, and 3.8.4.6

On pages 3.8.4-2, SR 3.8.4.2 contains requirements for Nickel Cadmium Battery cells. The Nickel Cadmium cells were replaced with Lead Acid Batteries as documented in the LAR dated

July 19, 2004 (ADAMS Accession No. ML042160222), as approved by letter dated April 27, 2005 (ADAMS Accession No. ML050930033). On pages 3.8.4-2 and 3.8.4-3, SRs 3.8.4.3 and 3.8.4.6 contain descriptions of the batteries still in use.

The NRC staff examined these changes. The proposed new SR 3.8.4.2 deletes all words and is now "Not used. The proposed new SR 3.8.4.3 deletes the words "(For the DC channel and DG batteries utilizing lead acid cells only)." The proposed new SR 3.8.4.6 deletes the words "lead acid." As Catawba no longer used Nickel Cadmium batteries, and the description of the remaining batteries is not needed to meet 10 CFR 50.36(c)(3), these changes to the text of SRs 3.8.4.2, 3.8.4.3, and 3.8.4.6 are acceptable.

TS LCO 3.8.6

On pages 3.8.6-1 and 3.8.6-3, LCO 3.8.6 contains requirements for Nickel Cadmium Battery cells. The Nickel Cadmium cells were replaced with Lead Acid Batteries as documented in the LAR dated July 19, 2004 (ADAMS Accession No. ML042160222), as approved by letter dated April 27, 2005 (ADAMS Accession No. ML050930033). LCO 3.8.6 on pages 3.8.6-2 and 3.8.6-3 and SR 3.8.6.2 on page 3.8.6-4 contain descriptions of the batteries still in use.

The NRC staff examined these changes. The proposed changes delete condition C of LCO 3.8.6. The proposed new SR 3.8.6.2 deletes all words and is now "Not used. The words "utilizing lead acid cells" are deleted from condition A and B of LCO 3.8.6. The words "utilizing lead acid cells" are deleted from SR 3.8.6.1 and 3.8.6.3. As Catawba no longer uses Nickel Cadmium batteries, and the description of the remaining batteries is not needed to meet 10 CFR 50.36(c)(3), these changes to LCO 3.8.6 and SRs 3.8.6.1, 3.8.6.2, and 3.8.6.3 are acceptable.

TS 5.2, "Organization"

Section 5.2.1, item d, contains a reference to an organization title "Executive Vice President Nuclear Generation Department," which will be changed to "Chief Nuclear Officer."

The NRC staff has reviewed this proposed change and approves it based on the fact that the necessary duties are still performed by a new officer reflecting the updated organization.

TS 5.6, "Reporting Requirements"

TS 5.6.5, item a.15, incorrectly included TS 3.9.2. TS 3.9.2 and, therefore, reference to TS 3.9.2 would be deleted.

The NRC staff has reviewed this proposed change and approves it based on the fact that TS 3.9.2 is not related to Reactor Makeup Water Pumps Combined Flow Rates limits and, therefore, reference to TS 3.9.2 should be deleted.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact was published in the *Federal Register* on October 14, 2009 (74 FR 52823). Accordingly, based upon the environmental assessment, the Commission has determined that issuance of this amendment will not have a significant effect on the quality of the human environment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: J. Thompson

Date of issuance: October 30, 2009

October 30, 2009

Mr. J. R. Morris
Site Vice President
Catawba Nuclear Station
Duke Energy Carolinas, LLC
4800 Concord Road
York, SC 29745

SUBJECT: CATAWBA NUCLEAR STATION, UNITS 1 AND 2, ISSUANCE OF
AMENDMENTS REGARDING CHANGES TO THE LICENSES AND
TECHNICAL SPECIFICATIONS (TAC NOS. MD9871 AND MD9872)

Dear Mr. Morris:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 253 to Renewed Facility Operating License NPF-35 and Amendment No. 248 to Renewed Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2, respectively. The amendments consist of changes to the licenses and Technical Specifications (TSs) in response to your application dated October 8, 2008, as supplemented by letter dated May 5, 2009.

The amendments would revise the licenses and TSs by removing and updating portions of the licenses and TSs which are out of date or are obsolete including footnotes and references.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

If you have any questions, please call me at 301-415-1119.

Sincerely,

/RA/

Jon Thompson, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures:

1. Amendment No. 253 to NPF-35
2. Amendment No. 248 to NPF-52
3. Safety Evaluation

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NAME	JThompson	MO'Brien (SRohrer for)	RElliott	DRoth /w comments	GKulesa (DWright for)	JThompson
DATE	10/15/09	10/15/09	10/19/09	10/29/09	10/30/09	10/30/09

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