

Indian Point Nuclear Generating Units 2 and 3
Docket Nos. 50-247/ 50-286-LR

**NRC Staff's Response in Opposition to State of New York's Motion for Partial Summary
Disposition of NYS Contention 16/16A**

Exhibit B

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Education

M.S., Mechanical Engineering, University of Maryland, 1981
B.S., Mechanical Engineering, University of Maryland, 1975

Employment

U.S. Nuclear Regulatory Commission, 1981 – present

Performs technical evaluations of license applications and policy issues in the areas of severe accident progression and phenomena, containment performance, offsite consequences, and probabilistic risk assessment.

NRC lead on the following:

- review of Severe Accident Mitigation Alternatives (SAMAs) for all license renewal applications to date, including Indian Point, all certified advanced reactor designs (ABWR, CE System 80+, AP600 and AP1000); and initial plant licenses (Limerick, Comanche Peak, Watts Bar Unit 1)
- development of sections of Generic Environmental Impact Statement for License Renewal (NUREG-1437, Revision 1, Draft for Comment) related to severe accidents
- NRC headquarters support to NRC regions for Significance Determination Process issues/findings related to containment performance and large early release frequency
- development of NRC staff guidance on use of risk information in review of non-risk-informed license amendment requests, and all key documents, including SECY-99-246, Regulatory Issue Summaries 2000-7 and 2001-02, and Appendix D to SRP Chapter 19
- review of industry severe accident management guidelines (SAMG) and severe accident management program implementation (including issuance of Generic Letter 88-20, Supp. 2)
- review of Level 2 (containment performance) and Level 3 (offsite consequences) portions of PRA and severe accident evaluations for all certified advanced reactor designs
- risk evaluation for Generic Safety Issue 189-Susceptibility of Ice Condenser and Mark III Containments to Early Failure from Hydrogen Combustion During a Severe Accident

Task force member and contributing author on the following:

- Review of petitions for rulemaking related to treatment of Spent Fuel Pool (SFP) fires and terrorism in environmental assessments
- NRC response to 9/11 in areas related to strategies for responding to loss of large areas of the plant (Item B.5.b), including development of Safeguards Advisories and Security Orders
- Technical Study of Spent Fuel Pool Accident Risk at Decommissioning Nuclear Power Plants (NUREG-1738)

- NRC staff affidavit regarding the risk implications of a proposed expansion of spent fuel storage at Shearon Harris (Docket 50-400-LA; ASLBP No. 99-762-02-LA)
- Risk Assessment of Severe Accident-Induced Steam Generator Tube Rupture (NUREG-1570)
- Significance Determination Process for assessing risk significance of containment-related inspection findings as part of the Reactor Oversight Process (Manual Chapter 0609, Appendix H).

Member of Reactor Safety Team, USNRC Emergency Operations Center

Professional Societies

American Society of Mechanical Engineers