

FACILITY NAME: Robinson

Section 8

REPORT NUMBER: 2009-301

## FINAL RO WRITTEN EXAM

### CONTENTS:

- Final RO Written Exam (75 'as given' questions with changes made during administration annotated)
- Reference Handouts Provided To Applicants
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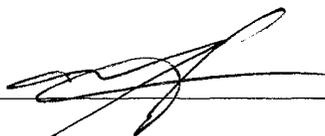
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Submitted By: 

Verified By: 

**U.S. Nuclear Regulatory Commission  
Site-Specific RO Written Examination**

**Applicant Information**

Name: \_\_\_\_\_

Date: \_\_\_\_\_

Facility/Unit: \_\_\_\_\_

Region:    I  II  III  IV

Reactor Type: W  CE  BW  GE

Start Time: \_\_\_\_\_

Finish Time: \_\_\_\_\_

**Instructions**

Use the answer sheets provided to document your answers. Staple this cover sheet on top of the answer sheets. To pass the examination, you must achieve a final grade of at least 80.00 percent. Examination papers will be collected 6 hours after the examination begins.

**Applicant Certification**

All work done on this examination is my own. I have neither given nor received aid.

\_\_\_\_\_   
Applicant's Signature

**Results**

Examination Value \_\_\_\_\_ Points

Applicant's Score \_\_\_\_\_ Points

Applicant's Grade \_\_\_\_\_ Percent

**ANSWER KEY REPORT**  
for ILC-09 NRC Exam Test Form: 0

Answers

ID	0	
1	007 EG2.4.49 1	B
2	009 EK2.03 1	D
3	011 EA1.15 1	A
4	022 AK1.02 1	A
5	025 AA1.12 1	B
6	026 AG2.4.4 1	B
7	027 AG2.1.19 1	B
8	029 EA1.08 1	D
9	038 EK1.04 1	B
10	040 AK1.02 1	D
11	057 AA2.19 1	A
12	058 AK3.01 1	B
13	062 AA2.02 1	D
14	065 AK3.04 1	D
15	077 AA2.07 1	C
16	WE04.EK2.2 1	D
17	WE05 EK2.1 1	B
18	WE11 EK3.2 1	C
19	001 AA1.01 1	C
20	005 AA1.01 1	C
21	024 AA2.01 1	C
22	036 AK3.02 1	B
23	059 AK3.04 1	A
24	067 AG2.4.49 1	C
25	069 AK2.03 1	D
26	WE08 EA1.3 1	C
27	WE09 EK1.1 1	C
28	003 A1.06 1	C
29	003 K6.04 1	D
30	004 A3.09 1	C
31	005 K6.03 1	A
32	006 A3.01 1	D
33	007 A2.05 1	C
34	008 A3.04 1	A
35	008 K1.04 1	B
36	010 A2.02 1	D
37	012 K2.01 1	C
38	012 K5.01 1	B
39	013 G2.1.7 1	B
40	022 K4.05 1	B
41	026 K2.01 1	B
42	026 K3.02 1	D
43	039 K5.01 1	C
44	059 K3.04 1	A
45	061 K1.07 1	B
46	061 K4.02 1	D

**ANSWER KEY REPORT**  
for ILC-09 NRC Exam Test Form: 0

Answers

ID	0
47	D
48	A
49	C
50	C
51	D
52	A
53	C
54	D
55	A
56	A
57	D
58	B
59	A
60	C
61	B
62	C
63	B
64	B
65	B
66	C
67	A
68	C
69	D
70	B
71	A
72	A
73	C
74	A
75	A
76	A
77	C
78	C
79	A
80	A
81	D
82	A
83	B
84	B
85	D
86	B
87	B
88	C
89	A
90	A
91	B
92	A

**ANSWER KEY REPORT**  
for ILC-09 NRC Exam Test Form: 0

Answers

ID	0
93	071 G2.1.28 1
94	G2.1.25 1
95	G2.2.13 1
96	G2.2.18 1
97	G2.3.11 2
98	G2.3.12 1
99	G2.4.13 1
100	G2.4.41 1

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1.

Given the following:

- Plant trip has just occurred from 100% RTP.
- PATH-1 immediate actions are being performed.
- The AUTOMATIC and MANUAL Turbine trips were NOT successful.

Which ONE (1) of the following describes the required operator actions IAW PATH-1?

- A. Place EH Pump A and B control switches in the Pull-To-Lock position.
- B. Manually runback the turbine to no load conditions.
- C. Close the MSIVs and MSIV Bypass valves.
- D. Manually trip the turbine locally at the Turbine Front Standard.

2.

Given the following:

- Reactor Trip and Safety Injection have occurred.
- RCS pressure is 1200 psig.
- Containment pressure is 4.5 psig.
- SI flow has just been verified per PATH-1.
- Total AFW flow is 245 gpm.
- S/G conditions are 10% NR level and 490 psig steam pressure.

Which ONE (1) of the following correctly describes the plant status and required procedure flowpath based on the given conditions?

- A. S/G level and AFW flow are adequate and the crew will remain in PATH-1.
- B. S/G level is adequate but AFW flow is NOT adequate and the crew will transition to FRP-H.1, Response to Loss of Secondary Heat Sink.
- C. AFW flow is NOT adequate but S/G level is adequate and the crew will remain in PATH-1.
- D. AFW flow and S/G level are NOT adequate and the crew will transition to FRP-H.1.

ILC-09 NRC Exam

3.

Given the following:

- A Large Break LOCA has occurred.
- CRSS has directed the RO to monitor RCS Pressure and Subcooling.

RTD subcooling is 33°F

CETC subcooling is 31°F

Which ONE (1) of the following describes the required instrumentation to be used to obtain these values IAW OMM-022, Emergency Operating Procedures User's Guide?

Obtain pressure using (1) and subcooling using (2).

- A. 1. Wide Range Pressure from ICCM  
2. CETC
- B. 1. Wide Range Pressure from ICCM  
2. RTD
- C. 1. PI-402, RCS Wide Range Pressure  
2. CETC
- D. 1. PI-402, RCS Wide Range Pressure  
2. RTD

ILC-09 NRC Exam

4.

Given the following:

- Plant in MODE 1 at 100% RTP.
- APP-001-B6, LP LTDN LN HI TEMP, goes into alarm.
- APP-003-F4, CHG PMP HI SPEED, goes into alarm.
- Charging pump discharge pressure has decreased to 2150 psig.

Which ONE (1) of the following describes the event in progress?

- A. A leak is occurring in the charging line.
- B. A leak in the Non-Regenerative Heat Exchanger is occurring.
- C. LCV-115A, VCT/Holdup Tank Diversion Valve, is diverting.
- D. Letdown Orifice Isolation valve leakage is occurring.

5.

Given the following:

- Plant is in Mode 5 with RHR Pump A in service.
- The RCS is water solid.
- RHR-751, RHR Loop Supply, has failed closed.
- CV penetrations are open.
- AOP-020, Loss of Residual Heat Removal (Shutdown Cooling), has been entered due to loss of RHR flow.

Which ONE (1) of the following describes the required parameter to be used for determining the RCS temperature IAW AOP-020?

- A. RHR Pump Discharge
- B. Core Exit Thermocouple
- C. Wide Range RCS Loop
- D. S/G Shell Side Metal

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6.

Given the following:

- Plant is in Mode 3 at 547°F.
- APP-001-A4, CCW SURGE TK HI/LO LVL alarm came in 2 minutes ago.
- Current CCW Surge Tank level is at 4% and decreasing.

Which ONE (1) of the following describes the required operator actions?

- A. Perform OP-306, CCW System. A lockout of all CCW pumps will be required.
- B. Enter AOP-014, Component Cooling Water System Malfunction. A lockout of all CCW Pumps will be required.
- C. Perform OP-306, CCW System. Makeup to the CCW Surge Tank with both Primary Water pumps will be required.
- D. Enter AOP-014, Component Cooling Water System Malfunction. Makeup to the CCW Surge Tank with both Primary Water pumps will be required.

7.

Given the following:

- A review of PZR Pressure ERFIS data shows the following:
  - PT-444, PZR Pressure, drifts upward and is indicating higher than actual RCS pressure.

Which ONE (1) of the following describes how PC-444J, PZR Pressure Controller and PT-445, PZR Pressure, will respond to this condition?

PC-444J output will .....

- A. increase. PT-445 indication will increase.
- B. increase. PT-445 indication will decrease.
- C. decrease. PT-445 indication will increase.
- D. decrease. PT-445 indication will decrease.

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8.

Given the following:

- MST-021, Reactor Protection Logic Train B at Power, is in progress with Reactor Trip Bypass Breaker B racked in and closed.
  
- During the testing, a plant transient occurred and resulted in an ATWS.

Which ONE (1) of the following describes the status of the Reactor Trip Breakers (RTBs) and Reactor Trip Bypass Breakers (RTBBs) upon depressing the Manual Reactor Trip Pushbutton(s) on the RTGB?

- A. RTB "A" shunt trip coil de-energized. RTBB "B" shunt trip coil energized.
- B. RTB "A" shunt trip coil energized. RTBB "B" shunt trip coil energized.
- C. RTB "A" shunt trip coil de-energized. RTBB "B" shunt trip coil de-energized.
- D. RTB "A" shunt trip coil energized. RTBB "B" shunt trip coil de-energized.

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9.

Given the following:

- The plant has experienced a S/G tube rupture with a Loss of Coolant Accident.
- Natural Circulation can NOT be verified IAW Supplement E, Natural Circulation Verification.
- Current RCS level is at the hot leg centerline.
- The break is at the loop flow transmitter section of the Cold Leg with RCS pressure at 1000 psig.
- No SI flow exists due to multiple malfunctions.

Which ONE (1) of the following is a current mode of heat removal from the core?

- A. Two phase flow
- B. Reflux Boiling
- C. Steam through RCS break
- D. Steam through ruptured S/G relief

10.

Given the following:

- A steam line break on S/G C inside the CV has occurred.
- Initial steam line pressure was 950 psig with a steaming rate of  $1 \times 10^6$  lbm/hr.
- Current steam line pressure is 475 psig.

Which ONE (1) of the following describes the approximate current steaming rate through the break?

- A.  $0.22 \times 10^6$  lbm/hr
- B.  $0.30 \times 10^6$  lbm/hr
- C.  $0.50 \times 10^6$  lbm/hr
- D.  $0.70 \times 10^6$  lbm/hr

11.

Given the following:

- The reactor is currently at 6% power with the turbine rolling at 1800 rpm IAW GP-005, Power Operation.
- Power is lost to Instrument Bus #1.

Which ONE (1) of the following describes the plant impact due to the loss of Instrument Bus #1?

Direct....

- A. Reactor Trip causing Turbine Trip.
- B. Turbine Trip causing Reactor Trip.
- C. Reactor Trip and direct Turbine Trip.
- D. Turbine Trip without Reactor Trip.

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12.

A complete loss of DC Bus A has occurred.

Which ONE (1) of the following describes the response of the "A" Emergency Diesel Generator?

The EDG will .....

- A. start due to the air start valves failing open and will be capable of energizing the E-bus.
- B. start due to the air start valves failing open and will NOT be capable of energizing the E-bus.
- C. NOT start due to the air start valves failing closed.
- D. NOT start due to the emergency bus remaining energized.

13.

Given the following:

- Unit operating in Mode 1 at 100% RTP.
- The NRC has notified the Control Room of a threat that an aircraft impact is expected at the RNP site within the next 5 minutes.
- AOP-034, Security Events, has been implemented.
- The reactor has been manually tripped.
- Offsite power remains available.
- Security personnel have reported that the tainter gates have been destroyed.
- Unit #1 personnel have reported that the low lake level alarm has been received.

Which ONE (1) of the following describes the impact that these conditions have on the potential loss of service water?

Service Water Pump(s).....

- A. are tripped immediately to prevent pump damage.
- B. can be maintained in operation for days due to the deep trench leading to the Unit #2 intake being lower than the dam tainter gates.
- C. operation is reduced to no more than ONE (1) pump operating to conserve water usage and prevent pump damage.
- D. are provided the necessary suction for the next 5 hours.

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14.

Given the following:

- Plant is operating in Mode 1 at 100% RTP.
- A loss of Instrument Air occurs due to a header rupture.

Which ONE (1) of the following describes the plant response to this loss of Instrument Air pressure?

- A. Steam Line PORVs will continue to operate as designed after manual backup alignment is made to air supply bottles.
- B. LCV-115B, Emergency MU to Chg Suct, will operate as designed after manual backup alignment is made to air supply bottles.
- C. Steam Line PORVs will continue to operate as designed due to the automatic backup air supply bottles.
- D. LCV-115B, Emergency MU to Chg Suct, will operate as designed due to the automatic backup air supply bottles.

15.

Given the following:

- Plant is in Mode 1 at 40% RTP.
- The Load Dispatcher reports that several major generating stations have tripped and that grid voltage is degrading.
- 2 minutes ago, 480V bus voltage reduced to 425V.

Which ONE (1) of the following describes the current status of 480V Busses E-1 and E-2?

Emergency busses are energized from.....

- A. offsite power and the EDGs will NOT start until the generator lockout separates the emergency busses from offsite power.
- B. offsite power and the EDGs will NOT start until the emergency busses undervoltage setpoint is reached.
- C. the EDGs with Blackout loads operating.
- D. the EDGs with Safeguards loads operating.

16.

Given the following:

- PATH-1 in progress due to LBLOCA.
- Transition to EPP-20, LOCA Outside Containment, was made.
- Crew is attempting to isolate the leak.

Which ONE (1) of the following describes the first pathway isolated in an attempt to isolate the leak IAW EPP-20?

- A. Cold Leg Injection Path
- B. Hot Leg Injection Path
- C. RWST to SI pump suction Path
- D. RHR Pump Suction Path

17.

Given the following:

- Plant is operating in Mode 1 at 100% RTP.
- A Reactor Trip and Safety Injection have occurred.
- Multiple failures have resulted in a loss of all AFW.
- PATH-1 has been implemented and the crew has transitioned to FRP-H.1, Response to Loss of Secondary Heat Sink.
- Both Condensate Pumps are running.

Which ONE (1) of the following describes the required actions to restore feed flow to the S/G(s) IAW FRP-H.1?

Override the Feedwater Isolation to.....

- A. ONLY ONE S/G and start 1 Main Feedwater Pump to feed ONE S/G with the Feedwater Regulating Valve.
- B. ALL S/Gs and start 1 Main Feedwater Pump to feed all S/Gs with the Feedwater Regulating Bypass Valves.
- C. ONLY ONE S/G and start 1 Main Feedwater Pump to feed ONE S/G with the Feedwater Regulating Bypass Valve.
- D. ALL S/Gs and start 1 Main Feedwater Pump to feed all S/Gs with the Feedwater Regulating Valves.

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18.

Given the following:

- The crew has implemented EPP-15, Loss of Emergency Coolant Recirculation.
- RWST level is 8%.
- The RHR, SI and CV Spray Pumps have been stopped and control power fuses have been removed from their breakers.
- The crew has been directed to perform the **INITIAL** depressurization of the intact S/G(s) by dumping steam to the condenser at **MAXIMUM** rate.

This step directs the crew to depressurize the Intact S/G(s) to (1) psig to (2).

- A. (1) 600  
(2) prevent SI Accumulator injection at this time
- B. (1) 600  
(2) ensure SI Accumulator injection at this time
- C. (1) 700  
(2) prevent SI Accumulator injection at this time
- D. (1) 700  
(2) ensure SI Accumulator injection at this time

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19.

Given the following:

- Power is being increased IAW GP-005, Power Operation.
- Reactor power is 17%.
- The RO withdraws Control Bank "D" 5 steps for temperature control, but the control rods continue to withdraw after the switch has been released to the mid position.

Which ONE (1) of the following describes the required procedural actions AND state the OPERABILITY of Control Bank "D" rods?

- A. Trip the Reactor AND Go to PATH-1. Rods are OPERABLE since they have performed their design function.
- B. Trip the Reactor AND Go to PATH-1. Rods are INOPERABLE due to the system failure.
- C. Place the ROD BANK SELECTOR Switch in AUTO and verify rod motion stops. Rods are OPERABLE since they are trippable.
- D. Place the ROD BANK SELECTOR Switch in AUTO and verify rod motion stops. Rods are INOPERABLE due to the system failure.

20.

Given the following:

- Unit power reduction is in progress.
- Control Bank D rods are being inserted when the Rod Deviation alarm on ERFIS is received.
- Control Bank D is at 188 steps.
- AOP-001, Malfunction of Reactor Control System, is entered and the investigation reveals that ONE (1) control rod in Control Bank D is misaligned from its bank average position by 13 steps.
- It is later determined that the control rod is STUCK.

Which ONE (1) of the following describes the required actions for the stuck rod IAW AOP-001?

Within ONE (1) hour of discovery of the misaligned rod.....

- A. reduce thermal power to less than or equal to 50%.
- B. reduce thermal power to less than or equal to 70%.
- C. verify that Shutdown Margin is within the limits of the COLR.
- D. initiate boration of the RCS to offset the rod worth of the stuck rod.

21.

Given the following:

- Plant in MODE 1 at 100% RTP.
- A reactor trip and turbine trip occurs due to the failure of Generator Lockout 86P.
- PATH-1 and EPP-4, Reactor Trip Response, are implemented.
- The RO notes that 2 control rods do not indicate inserted.
- The RO opens MOV-350, Boric Acid to Charging Pump Suction Header Valve, and starts the Boric Acid Pump aligned to BLEND as directed by EPP-4.

Which ONE (1) of the following describes the expected indications that boration from the Boric Acid Storage Tanks is occurring?

- A. FR-113 Pen 1, Boric Acid Flow indicates 0 GPM due to LCV-115B, Emerg MU to Chg Suct, being open and LCV-115C, VCT Outlet, being closed.
- B. FI-110, Boric Acid Bypass Flow, indicates 0 GPM due to LCV-115B, Emerg MU to Chg Suct, being open and LCV-115C, VCT Outlet, being closed.
- C. FI-110, Boric Acid Bypass Flow, indicates 65 GPM due to seeing all the boric acid flow.
- D. FR-113 Pen 1, Boric Acid Flow, indicates 65 GPM due to seeing all the boric acid flow.

22.

Given the following:

- The plant is in Mode 6 with the core offload to the SFP in progress.
- The CV Manipulator has just removed a fuel assembly from core location H-6 and is in transit to the CV Upender.
- The CV Manipulator Operator accidentally places the gripper control switch to the "DISENGAGE" position before the fuel assembly is lowered into the CV Upender.

Which ONE (1) of the following describes the results of his action?

The fuel assembly will NOT be released due to the .....

- A. HOIST-MAST POSITION interlock.
- B. GRIPPER-WEIGHT interlock.
- C. GRIPPER TUBE UP interlock.
- D. BRIDGE-TROLLEY interlock.

23.

Given the following:

- Plant in MODE 1 at 100% RTP.
- The crew has entered AOP-008, Accidental Release of Liquid Waste, due to a confirmed leak by the Inside AO performing rounds.
- Source of leakage has NOT been determined at this point.

Which ONE (1) of the following describes the order and reason of procedural steps of AOP-008?

Check leakage source from the .....

- A. RWST first. Leakage from this tank CAN result in Auxiliary Building flooding.
- B. RWST first. Leakage from this tank CAN cause the overflow of drains outside the SI Pump Room.
- C. Monitor Tanks first. Leakage from these tanks CAN result in Auxiliary Building flooding.
- D. Monitor Tanks first. Leakage from these tanks CAN cause the overflow of drains outside the SI Pump Room.

24.

Given the following:

- Plant is operating in Mode 1 at 100% RTP.
- APP-044-A02; ZN-1 Fire Alm. TRN-A B Diesel Gen. Rm. goes into alarm on the Fire Alarm Console with the alarm text in YELLOW.
- No other alarms exist.

Which ONE (1) of the following describes actions required by the crew with this condition IAW APP-044, Fire Alarm Console (FAC) ?

- A. Activate Fire Brigade IAW AOP-041, Response to Fire Event.
- B. Secure area ventilation fans IAW AOP-041, Response to Fire Event.
- C. Dispatch FPAO or closest Fire Brigade member to investigate alarm.
- D. Enter DSP-001, Alternate Shutdown Diagnostic.

25.

Given the following:

- Plant in MODE 1 at 100% RTP.
- A routine CV entry has been initiated for inspections.
- APP-036-L5, CV Personnel Hatch Door Open, is illuminated.

Which ONE (1) of the following describes the surveillance requirements to ensure Containment Integrity IAW APP-036-L5?

Perform....

- A. EST-010, Containment Personnel Airlock Leakage Test, within 24 hours.
- B. EST-010, Containment Personnel Airlock Leakage Test, within 72 hours.
- C. OST-014, LLRT of Personnel Air Lock Door Seals, within 24 hours.
- D. OST-014, LLRT of Personnel Air Lock Door Seals, within 72 hours.

26.

Given the following:

- Mode 1 at 100% RTP.
- Transient on the plant has resulted in 2 faulted S/Gs.
- Reactor Trip and Safety Injection have been actuated.
- AFW has been isolated to the faulted S/Gs IAW Foldout A.
- Cooldown rate has exceeded 100°F in the last 60 minutes.

Which ONE (1) of the following describes the actions that would meet the requirements of FRP-P.1, Response to Imminent Pressurized Thermal Shock?

Maintain RCS temperature stable for a minimum of (1) and then maintain maximum cooldown rate in RCS Cold Legs less than (2).

- A. (1) 2 hours  
(2) 50°F/hour
- B. (1) 2 hours  
(2) 60°F/hour
- C. (1) 1 hour  
(2) 50°F/hour
- D. (1) 1 hour  
(2) 60°F/hour

27.

Given the following:

- Plant tripped from 100% RTP due to Loss of Offsite Power.
- Decision has been made to place the unit in Mode 5 using EPP-5, Natural Circulation Cooldown.
- Cold shutdown boron concentration has been established.
- CRDM Cooling Fan HVH-5A is operating. HVH-5B is NOT available.
- Low Tave and Low PZR Pressure SI signals have been blocked.

Which ONE (1) of the following actions are required to prevent formation of a steam void in the reactor vessel upper head during the RCS cooldown IAW EPP-5, Natural Circulation Cooldown?

Maintain RCS subcooling greater than.....

- A. 35°F and cooldown rate less than 10°F / hour.
- B. 35°F and cooldown rate less than 25°F / hour.
- C. 100°F and cooldown rate less than 10°F / hour.
- D. 100°F and cooldown rate less than 25°F / hour.

28.

Given the following:

- Plant is in Mode 3 at 547°F.
- RCP B has been stopped as requested by RESS for balance shot placement.

Which ONE (1) of the following combinations will provide sufficient PZR spray flow?

- A. PCV-455A with ONLY RCP A operating.
- B. PCV-455A with ONLY RCP C operating.
- C. PCV-455B with ONLY RCP C operating.
- D. PCV-455B with ONLY RCP A operating.

29.

Given the following:

- Plant is in Mode 1 at 70% RTP.
- Valve CVC-381, RCP Seal Return Isolation, has closed and cannot be re-opened.

Which ONE (1) of the following describes the plant response and impact from the failure?

Seal return flow.....

- A. has been lost. RCP operation may continue provided that thermal barrier cooling flow verified.
- B. has been lost. RCP operation may NOT continue due to loss of seal cooling.
- C. will continue to the RCDDT through CVC-382, RCP Seal Return Line Relief.
- D. will continue to the PRT through CVC-382, RCP Seal Return Line Relief.

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30.

Which ONE (1) of the following describes the minimum instrumentation and VCT level setpoint to initiate automatic swapper of the charging pump suction to the RWST?

- A. LT-112 **OR** LT-115 - 12.4 inches and decreasing.
- B. LT-112 **OR** LT-115 - 14.4 inches and decreasing.
- C. LT-112 **AND** LT-115 - 12.4 inches and decreasing.
- D. LT-112 **AND** LT-115 - 14.4 inches and decreasing.

31.

Given the following:

- Plant is in Mode 4 at 210°F.
- RHR Pump B is in operation and aligned for core cooling IAW OP-201, RHR System.
- The OST team inadvertently closes valve RHR-759B, RHR HX "B" DISCH, while performing system alignments for testing.

Which ONE (1) of the following describes the system response to the given conditions?

- A. FCV-605 automatically adjusts OPEN
- B. HCV-758 automatically adjusts OPEN
- C. FCV-605 automatically adjusts CLOSE
- D. HCV-758 automatically adjusts CLOSE

32.

Given the following:

- Plant is in Mode 1 at 100% RTP.
- A Design Basis Large Break LOCA and Loss of Offsite Power has occurred.

Which ONE (1) of the following describes the component that will be the first to inject into the core?

- A. Safety Injection Pumps
- B. Residual Heat Removal Pumps
- C. Charging Pumps
- D. SI Accumulators

33.

Given the following:

- A PZR Safety valve has failed open.
- APP-003-B3, PRT HI TEMP has alarmed.
- APP-003-C3, PRT HI PRESS has alarmed.
- RCS pressure is 1900 psig and decreasing.
- CV pressure is approximately 3 psig.

Which ONE (1) of the following indications will exist based on the plant status and what APP-003 section will mitigate the situation?

PRT pressure will be approximately.....

- A. 3 psig. APP-003-B3, PRT HI TEMP, will direct operator to add primary water until high temperature alarm clears.
- B. 100 psig. APP-003-B3, PRT HI TEMP, will direct the crew to PATH-1.
- C. 3 psig. APP-003-C3, PRT HI PRESS will direct the crew to PATH-1.
- D. 100 psig. APP-003-C3, PRT HI PRESS, will direct venting the PRT.

34.

Given the following:

- Plant is in Mode 1 at 100% RTP.
- CCW Pump status: A - Standby  
B - Running  
C - Standby
- Crew has started CCW Pump "C" and secured CCW Pump "B" IAW OP-306, Component Cooling Water System.
- When CCW Pump "B" was stopped, APP-001-F5, CCW PMP LO PRESS, was received and cleared.

Which ONE (1) of the following correctly describes the status of CCW Pumps A and B?

- A. A - Locked Out  
B - Locked Out
- B. A - Available for autostart  
B - Locked Out
- C. A - Locked Out  
B - Available for autostart
- D. A - Available for autostart  
B - Available for Autostart

35.

Given the following:

- Plant is in Mode 1 at 30% RTP.
- APP-036-D8, PROCESS MONITOR HI RAD, has been received and R-17, CCW Radioactive Liquid, is in High alarm.
- FCV-626, Therm Barrier Outlet, has automatically closed.

Which ONE (1) of the following describes the method to determine the source of leakage into the CCW System IAW AOP-014, CCW System Malfunction?

AOP-014, Attachment 6 directs.....

- A. FCV-626 to remain closed.  
Verify seal injection flows at 8 to 13 gpm prior to manually throttling open FCV-626.
- B. isolation of all thermal barrier return lines.  
Re-open valve FCV-626 and unisolate each line one at a time.
- C. isolation of all thermal barrier return lines.  
Open CC-932, FCV-626 AND CC-735 BYPASS ISOLATION, and unisolate each line one at a time.
- D. FCV-626 to remain closed.  
Open CC-932, FCV-626 AND CC-735 BYPASS ISOLATION, provided that total thermal barrier flow remains < 100 GPM.

36.

Given the following:

- Plant is in Mode 2 at the Point of Adding Heat (POAH).
- PC-444J, PZR Pressure Controller, output is 48% and decreasing.
- PCV-455A, PZR Spray valve, indicates RED.

Which ONE (1) of the following procedures would be implemented to mitigate this condition and if not successful, what would be the first automatic RPS signal to trip the reactor?

- A. AOP-025, RTGB Instrument Failure  
PZR Pressure Low Reactor Trip
- B. AOP-025, RTGB Instrument Failure  
PZR Pressure Low Safety Injection
- C. AOP-019, Malfunction of RCS Pressure Control  
PZR Pressure Low Reactor Trip
- D. AOP-019, Malfunction of RCS Pressure Control  
PZR Pressure Low Safety Injection

37.

Which ONE (1) of the following provides the control power for Reactor Trip Breaker B and Reactor Trip Bypass Breaker B?

	<u>Reactor Trip Breaker B</u>	<u>Reactor Trip Bypass Breaker B</u>
A.	125V DC MCC-A	125V DC MCC-A
B.	125V DC MCC-A	125V DC MCC-B
C.	125V DC MCC-B	125V DC MCC-A
D.	125V DC MCC-B	125V DC MCC-B

38.

Given the following:

- Plant is in Mode 1 at 45% RTP.
- An electrical transient causes a momentary underfrequency and undervoltage condition on 4160V Bus 1.
- The RCP powered from 4160V Bus 1 trips.
- The other TWO (2) RCPs remain in operation, but the plant trips.

Which ONE (1) of the following identifies the input that DIRECTLY generates the FIRST Reactor Trip signal?

- A. RCS Low Flow
- B. RCP Breaker Trip
- C. 4160V Bus Undervoltage
- D. 4160V Bus Underfrequency

39.

Given the following:

- Plant is operating in Mode 1 at 100% RTP.
- An electrical fault has caused Inverter A to trip.
- During the electrical transient, a Safety Injection signal is generated.

Which ONE (1) of the following describes the impact on the operation of the Engineered Safety Features (ESF) Actuation System?

Train A Safeguards sequencer will (1). Train B Safeguards sequencer will (2).

- A. (1) actuate  
(2) actuate
- B. (1) NOT actuate  
(2) actuate
- C. (1) actuate  
(2) NOT actuate
- D. (1) NOT actuate  
(2) NOT actuate

40.

Given the following:

- Plant is operating in Mode 1 at 100% RTP.
- A Large Break LOCA occurs.
- The LOCA has collapsed the Containment ventilation ductwork resulting in loss of air flow from 2 HVH Units.

Which ONE (1) of the following describes the **minimum** design features required to maintain the Containment within pressure limits?

- A. ONLY ONE CV Spray Train with an actuation setpoint of 20 psig is required.
- B. ONLY ONE CV Spray Train with an actuation setpoint of 10 psig is required.
- C. BOTH CV Spray Trains with an actuation setpoint of 20 psig is required.
- D. BOTH CV Spray Trains with an actuation setpoint of 10 psig is required.

41.

Given the following:

- Plant is in Mode 1 at 50% RTP.
- Breaker 52/10, 4KV Bus 1 to 4KV Bus 2 TIE, trips.
- A Large Break LOCA occurs 15 seconds later.

Which ONE (1) of the following describes the configuration of the power supplies to CV Spray Pumps A and B?

CV Spray pump A powered from (1) and CV Spray pump B powered from (2).

- A. (1) EDG A  
(2) EDG B
- B. (1) EDG A  
(2) SUT
- C. (1) SUT  
(2) SUT
- D. (1) SUT  
(2) EDG B

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42.

Where does valve SI-871, Containment Spray Suction Relief Valve, relieve to?

- A. CVCS Holdup Tanks
- B. Waste Holdup Tank
- C. SI Pump discharge line
- D. CV Spray Pump discharge line

43.

Given the following:

- Plant in MODE 1 at 22% RTP and power ascension in progress IAW GP-005, Power Operation.
- The crew is making preparations for placing S/G blowdown heat recovery in service.

Which ONE (1) of the following describes the requirements for this alignment AND why IAW GP-005?

Turbine load shall be greater than.....

- A. 25%. This ensures sufficient condensate flow through the blowdown heat exchangers to minimize the possibility of water hammer.
- B. 25%. This ensures sufficient main steam flow to ensure adequate heating steam for the blowdown heat exchangers.
- C. 30%. This ensures sufficient condensate flow through the blowdown heat exchangers to minimize the possibility of water hammer.
- D. 30%. This ensures sufficient main steam flow to ensure adequate sealing steam for the blowdown heat exchangers.

44.

Given the following:

- Plant in Mode 1 at 100% RTP.
- A large feedwater line break just downstream of the Main Feedwater Pumps and upstream of the FW-8, S/G "A" INLET STOP CHECK, has resulted in a total loss of main feedwater.

Which ONE (1) of the following describes the effect of this event on the RCS without any operator action?

RCS temperature will (1) prior to the reactor trip and (2) after the reactor trip.

- A. (1) INCREASE  
(2) DECREASE to 547°F
- B. (1) INCREASE  
(2) DECREASE until the S/G blows dry
- C. (1) DECREASE  
(2) DECREASE to 547°F
- D. (1) DECREASE  
(2) DECREASE until the S/G blows dry

45.

Given the following:

- Plant in MODE 1 at 100% RTP.
- A reactor trip has occurred due to a loss of offsite power.
- PATH-1 is implemented and transition is made to EPP-4, Reactor Trip Response.
- During the transient, the Condensate Storage Tank (CST) level is currently at 14% and decreasing due to a leak.

Which ONE (1) of the following describes the actions necessary to provide AFW pump suction IAW EPP-4?

Once the CST level reaches 10%, align.....

- A. Fire Water from the Engine Driven Fire Pump to fill the CST.
- B. Service Water to provide suction to the AFW pumps.
- C. Fire Water to provide suction to the AFW pumps.
- D. Service Water to fill the CST.

46.

Given the following:

- The plant is in Mode 1 at 100% RTP.
- MDAFW Pump A is under clearance.
- A loss of offsite power and an SI signal occur at 10:00:00.
- The AFW Pumps are checked at 10:00:30 and ONLY the SDAFW Pump was running.

Which ONE (1) of the following choices completes the statement below IAW the Safeguards Sequencers?

MDAFW Pump B will start at time.....

- A. 10:00:39.5 by the Blackout Sequencer.
- B. 10:00:39.5 by the SI Sequencer.
- C. 10:00:49.5 by the Blackout Sequencer.
- D. 10:00:49.5 by the SI Sequencer.

47.

Given the following:

- Plant is in Mode 1 at 85% RTP.
- A spurious reactor trip has occurred.

Which ONE (1) of the following breakers will **automatically close** ONE (1) minute later?

Breaker nomenclature:

- 52/7 - UNIT AUX TO 4KV BUS 1
- 52/10 - 4KV BUS 1-2 TIE BKR
- 52/12 - START-UP TRANSFORMER TO 4KV BUS 2
- 52/17 - START-UP TRANSFORMER TO 4KV BUS 3
- 52/19 - 4KV BUS 3-4 TIE BKR
- 52/20 - UNIT AUX TO 4KV BUS 4

A. 52/10  
52/12

B. 52/10  
52/17

C. 52/17  
52/19

D. 52/12  
52/19

48.

Given the following:

- Plant operating in Mode 1 at 100% RTP.
- Charging Pump C is running.
- A plant trip occurs with the operators noting that Reactor Trip Breaker B has lost indication as well as equipment powered from 480V Bus E-2.

Which ONE (1) of the following is correct for the plant conditions THREE (3) minutes after the trip?

Charging Pump C will (1) and 4KV Busses 4 and 5 will (2).

- A. (1) remain in operation  
(2) be deenergized
- B. (1) remain in operation  
(2) remain energized
- C. (1) be tripped  
(2) be deenergized
- D. (1) be tripped  
(2) remain energized

49.

Given the following:

- Plant in MODE 1 at 100% RTP.
- A complete loss of 125V DC Distribution Panel A occurs.

Which ONE (1) of the following describes the effect on the Electrical Distribution System and a local action required IAW Foldout A?

Instrument Bus # (1) will be transferred to (2).

- A. (1) 3  
(2) MCC-8
- B. (1) 3  
(2) the DS Bus
- C. (1) 2  
(2) MCC-8
- D. (1) 2  
(2) the DS Bus

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50.

The minimum Emergency Diesel Generator fuel oil inventory maintained on site is \_\_\_\_\_ gallons to ensure that \_\_\_\_\_.

- A. 25,000; ONE (1) EDG can operate fully loaded for 7 days
- B. 25,000; BOTH EDGs can operate fully loaded for 7 days
- C. 34,000; ONE (1) EDG can operate fully loaded for 7 days
- D. 34,000; BOTH EDGs can operate fully loaded for 7 days

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51.

Which ONE (1) of the following describes Service Water (SW) flow to each of the Emergency Diesel Generators?

The North SW header supplies (1) while the South SW header supplies (2).  
These SW headers (3) be cross-connected.

- A. (1) A EDG  
(2) B EDG  
(3) can NOT
- B. (1) B EDG  
(2) A EDG  
(3) can NOT
- C. (1) A EDG  
(2) B EDG  
(3) can
- D. (1) B EDG  
(2) A EDG  
(3) can

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52.

Given the following:

- AOP-009, Accidental Gas Release from a WGDT, has been entered.
- While progressing through AOP-009, it is determined that R-14C, PLANT EFFLUENT NG LO, has failed LOW.

Which ONE (1) of the following describes the actions necessary to ensure monitoring via the plant vent IAW AOP-009?

- A. Shift R-11 and R-12 to the plant vent by selecting VENT on the RMS Control Panel in the Control Room.
- B. Notify E&C personnel to manually shift R-14D, PLANT EFFLUENT NG MID to in service.
- C. Shift R-11 and R-12 to the plant vent by selecting PURGE on the RMS Control Panel in the Control Room.
- D. Notify E&C personnel to manually shift R-14E, PLANT EFFLUENT NG HI to in service.

53.

Given the following:

- The plant is in Mode 3 at 547°F.
- Service Water Pump B LOCAL/REMOTE switch has been placed in LOCAL while I&C personnel are troubleshooting the RTGB control switch for the pump.
- Service Water Pumps A and C are operating.
- A spurious Train A Safeguards signal is received during the troubleshooting.

Which ONE (1) of the following describes the automatic response of the Service Water Pumps?

- A. ONLY Service Water Pumps A, B and C are running.
- B. ONLY Service Water Pumps A, C, and D are running.
- C. ONLY Service Water Pumps A and C are running.
- D. ALL Service Water Pumps are running.

54.

Given the following:

- The unit has tripped from 100% RTP followed by a loss of offsite power.
- EDGs A and B are powering 480V Busses E-1 and E-2.

Which ONE (1) of the following describes the air compressors that are available under these conditions?

- A. Instrument Air Compressor D and Primary Air Compressor.
- B. Instrument Air Compressor B and Primary Air Compressor.
- C. Instrument Air Compressors A and D.
- D. Instrument Air Compressors A and B.

55.

Given the following:

- Mode 1 at 100% RTP.
- CV temperature is 110°F and increasing.

Which ONE (1) of the following describes the maximum CV temperature and the actions to be taken IAW PLP-118, Hot Weather Operations, prior to reaching the ITS limit?

- A. 120°F limit; CV pressure is maintained at  $\leq 0.1$  psig.
- B. 115°F limit; CV pressure is maintained at  $\leq 0.1$  psig.
- C. 120°F limit; CV Purge is initiated continuously.
- D. 115°F limit; CV Purge is initiated continuously.

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56.

Which ONE (1) of the following describes the reactor trip signal that is disabled by maneuvering the plant below Permissive P-7 during a plant shutdown?

- A. PZR High Level
- B. PZR High Pressure
- C. S/G Low Low Level
- D. Source Range High Flux

57.

Given the following:

- Plant is in Mode 1 at 90% RTP.
- Control Bank D step counters indicate 196 steps.
- RPI indication for Control Bank D indicate as follows:

D8 - 124"  
M8 - 116"  
H4 - 120"  
H8 - 121"  
H12 - 129"

*represents*  


Which ONE (1) of the following ~~represents~~ represents the rod alignment status for Control Bank D?

- A. ONLY Control Rod M8 is misaligned.
- B. ONLY Control Rod H12 is misaligned.
- C. Control Rods M8 and H12 are misaligned.
- D. ALL control rods are within the allowable alignment limits.

58.

Given the following:

- Plant is in Mode 1 at 100% RTP.

Which ONE (1) of the following conditions could cause ONE (1) of the Overtemperature Delta T Reactor Protection trip bistables to actuate?

- A. PZR Pressure channel PT-455 fails HIGH.
- B. Power Range channel N-43 lower detector fails LOW.
- C. Power Range channel N-44 upper detector fails LOW.
- D. RCS Loop 2 cold leg wide range temperature channel fails HIGH.

59.

Given the following:

- Plant is in Mode 1 at 100% RTP with Tave = Tref.
- Tave inputs to the Rod Control System are as follows:
  - Loop A - 575.8°F
  - Loop B - 575.7°F
  - Loop C - 575.6°F
- Subsequently, Loop B Tave fails HIGH.

Which ONE (1) of the following describes the impact on the Rod Control System?  
**(Assume no operator action)**

Rods will.....

- A. NOT move because an insufficient demand signal is present.
- B. NOT move because an Urgent Failure alarm will be generated.
- C. insert because the controlling temperature input has failed HIGH.
- D. insert because the NI / Turbine rate circuit demand signal is present.

60.

Given the following:

- Crew is in GP-008, Draining the Reactor Coolant System.
- Both ICCM Panels are OOS.
- RO has been directed to obtain Incore Thermocouple readings locally.

Which ONE (1) of the following describes how incore temperatures are determined IAW GP-008, Attachment 10.4, Core Exit Thermocouples?

Resistance and (1) readings are taken locally in (2), then are plotted on a chart to obtain temperature.

- A. (1) millivolt  
(2) Containment
- B. (1) amperage  
(2) Containment
- C. (1) millivolt  
(2) the Rod Control Room
- D. (1) amperage  
(2) the Rod Control Room

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61.

Which ONE (1) of the following is the power supplies for HVE-3 and HVE-4, CV Air Iodine Removal Exhaust Fans respectively?

- A. MCC-1 and MCC-2
- B. MCC-5 and MCC-6
- C. MCC- 9 and MCC-10
- D. 480V Bus 2B and 480V Bus 3

62.

Given the following:

- Plant in Mode 1 at 65% RTP and increasing plant to full load.
- S/G A conditions: FF - 2.2E6 pph, SF - 2.2E6 pph, Steam Pressure - 890 psig.
- S/G B conditions: FF - 2.2E6 pph, SF - 2.2E6 pph, Steam Pressure - 880 psig.
- S/G C conditions: FF - 2.8E6 pph, SF - 3.7E6 pph, Steam Pressure - 740 psig.
- CV pressure is 0.15 psig and stable.

Which ONE (1) of the following contains the expected plant response to these conditions?

A Reactor Trip with a Safety Injection from High Steam Line.....

- A. Flow with Low Steam Line Pressure and NO Main Steam Line Isolation.
- B. Flow with Low Steam Line Pressure and Main Steam Line Isolation.
- C. Delta P and NO Main Steam Line Isolation.
- D. Delta P and Main Steam Line Isolation.

63.

Given the following:

- The unit trips due to a loss of condenser vacuum.
- All THREE (3) Circulating Water Pumps are operating.
- Actual condenser vacuum stabilizes at 12 inches Hg.
- The demand signal on the Steam Dump Pressure Controller (PC-464B) is at 50%.
- The BOP operator transfers the Steam Dump Mode switch from TAVG mode to STEAM PRESS mode.

Which ONE (1) of the following describes the response of the Steam Dump valves?

- A. ALL valves trip full open.
- B. ALL valves remain closed.
- C. THREE (3) valves trip full open.
- D. THREE (3) valves modulate open.

64.

Given the following:

- Mode 1 at 100% RTP.
- Voltage instabilities have resulted in a Generator Lockout actuation.
- APP-009-D6, OCB 52-9 FAILED TO OPEN, has been received.

Which ONE (1) of the following describes the consequences of this breaker failure?

The Main Generator will (1) and a (2) 230KV Bus Lockout will be received.

- A. (1) be disconnected from the grid  
(2) SOUTH
- B. (1) be disconnected from the grid  
(2) NORTH
- C. (1) motorize  
(2) SOUTH
- D. (1) motorize  
(2) NORTH

65.

Which ONE (1) of the following describes the source of the seal water for the Waste Gas Compressors AND what indication would be provided if the water source is lost?

A. Seal water is supplied by Service Water.

Separator tank low level is ONLY annunciated on the Waste Disposal Boron Recycle Panel.

B. Seal water is supplied by Component Cooling Water.

Separator tank low level is annunciated on the Waste Disposal Boron Recycle Panel and resultant alarm on APP-036, Auxiliary Annunciator.

C. Seal water is supplied by Service Water.

Separator tank low level is annunciated on the Waste Disposal Boron Recycle Panel and resultant alarm on APP-036, Auxiliary Annunciator.

D. Seal water is supplied by Component Cooling Water.

Separator tank low level is ONLY annunciated on the Waste Disposal Boron Recycle Panel.

66.

Given the following:

- Loss of offsite power has occurred.
- BOTH EDGs have failed to start and all attempts to start them have failed.
- The DSDG has been started and the DS Bus is energized.
- The Outside AO has been directed to implement EPP-22, Energizing Plant Equipment Using Dedicated Shutdown Diesel Generator.

Which ONE (1) of the following describes the actions and location for locally starting Service Water Pump D IAW EPP-22?

- A. Place the LOCAL / REMOTE control switch in the Rod Control Room to LOCAL and start the pump.
- B. Place the LOCAL / REMOTE control switch on the Charging Pump Room Panel in LOCAL and start the pump.
- C. Transfer the kirk-key switch in the CCW Pump Room for SW Pump D to the DS Bus and start the pump from the Charging Pump Room Panel.
- D. Transfer the kirk-key switch in the CCW Pump Room for SW Pump D to the DS Bus, and start the pump from the Rod Control Room.

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67.

Which ONE (1) of the following describes the NORMAL standby alignment for the specified components of the Control Room Ventilation System?

The Outside Air Dampers are .....

- A. OPEN with HVE-19A, CONT RM AIR CLEANING-STOPPED.
- B. CLOSED with HVE-19A, CONT RM AIR CLEANING - OPERATING.
- C. OPEN with HVE-19A, CONT RM AIR CLEANING - OPERATING.
- D. CLOSED with HVE-19A, CONT RM AIR CLEANING - STOPPED.

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68.

Which ONE (1) of the following describes the required duties of the RO in the Control Room during the core reload?

- A. Plot the 1/M when the fuel assemblies are loaded in the core.
- B. Notifies WCC Supervisor of changes in the Shutdown Safety Function Status Sheets.
- C. Monitor Source Range indication for proper reactivity management.
- D. Concurrence for bypassing refueling interlocks.

69.

Given the following:

- The unit is in Mode 3 at 547°F.
- A pressure transient causes RCS pressure to increase to 2755 psig.

Which ONE (1) of the following describes which ITS requirements have been exceeded and the required action duration?

- A. The Reactor Core Safety Limits have been exceeded. Restore compliance within 1 hour.
- B. The Reactor Core Safety Limits have been exceeded. Restore compliance within 5 minutes.
- C. The RCS Pressure Safety Limit has been exceeded. Restore compliance within 1 hour.
- D. The RCS Pressure Safety Limit has been exceeded. Restore compliance within 5 minutes.

70.

Given the following:

- RCS heatup is in progress IAW GP-002, Cold Shutdown to Hot Subcritical at No Load Tavg.
- RCS is at 312°F and 350 psig.
- RO notes that the SI Accumulators are de-pressurized.

Which ONE (1) of the following states the ITS LCO applicability for the SI Accumulators?

- A. Mode 3 at 350°F RCS temperature.
- B. Mode 3 at > 1000 psig PZR pressure.
- C. Mode 3 at 375°F RCS temperature.
- D. Mode 3 at > 2000 psig PZR pressure.

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71.

Which ONE (1) of the following describes the actions for controlling radiological release from a ruptured Steam Generator? (Assume no other complications other than ruptured Steam Generator)

Verify the proper setpoint on.....

- A. the ruptured S/G PORV IAW PATH-2 once the S/G has been identified.
- B. all S/G PORVs IAW PATH-2 once the S/G has been identified.
- C. the ruptured S/G PORV IAW PATH-2 ONLY after RCS temperature is below 547°F.
- D. all S/G PORVs IAW PATH-2 ONLY after RCS temperature is below 547°F.

72.

Given the following:

- RCS cooldown is in progress IAW GP-007, Plant Cooldown from Hot Shutdown to Cold Shutdown.
- The RCS Filter general area radiation level is 1200 mR/hr.
- You have been assigned by the WCC SRO to hang a clearance on the filter for replacement.

Which ONE (1) of the following describes the radiation area classification and the minimum approval authority for room entry?

	<u>Classification</u>	<u>Approval for Entry</u>
A.	Locked High Radiation Area	Radiation Control Supervision
B.	High Radiation Area	Radiation Control Supervision
C.	Locked High Radiation Area	CRSS
D.	High Radiation Area	CRSS

73.

Given the following:

- The reactor has tripped.
- The crew is performing actions of EPP-4, Reactor Trip Response.
- The CRSS is at Step 12, Determine Feedwater Status, which has an asterisk next to the step number.

Which ONE (1) of the following describes the reason for the asterisk next to the step?

- A. The substeps contained in the step must be performed in the exact order written.
- B. The substeps contained in the step may be performed in any order.
- C. The actions contained in the step are continuously applicable.
- D. The actions contained in the step must be completed prior to moving on to the next step.

74.

Given the following:

- The plant was operating in Mode 1 at 100% RTP when a loss of offsite and onsite power occurred.
- Critical Safety Function Status Trees indicate as follows:
  - Subcriticality - Green
  - Core Cooling - Red
  - Heat Sink - Red
  - Integrity - Green
  - Containment - Green
  - Inventory - White

Which ONE (1) of the following describes the proper procedure to be used and the basis for the procedure use?

- A. EPP-1, Loss of All AC Power, is used due to power must be available to at least ONE (1) Emergency Bus for minimum safeguards equipment to function.
- B. FRP-C.1, Response to Degraded Core Cooling, is used to reduce RCS pressure and minimize subcooling.
- C. EPP-1, Loss of All AC Power, is used due to power must be available to the DS Bus for minimum safeguards equipment to function.
- D. FRP-C.1, Response to Degraded Core Cooling, is used to reduce RCS temperature and ensure that RCS inventory is maintained.

75.

Given the following:

- An ALERT Emergency Event has been declared.
- The Shift Manager has assumed the duties of the Site Emergency Coordinator (SEC).
- Dialogic has been activated with the TSC and EOF to be staffed.

Which ONE (1) of the following describes the facility and position that is going to relieve the Control Room SEC of his duties?

- A. Technical Support Center - Site Emergency Coordinator
- B. Emergency Operations Facility - Emergency Response Manager
- C. Technical Support Center - Emergency Response Manager
- D. Emergency Operations Facility - Site Emergency Coordinator