

U.S. NUCLEAR REGULATORY COMMISSION

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee 1. PharmaLogic Ltd. 2. 9 Krupp Drive P.O. Box 786 Williston, Vermont 05495	In accordance with the letter dated August 17, 2009, 3. License number 44-30124-01MD is amended in its entirety to read as follows: 4. Expiration date September 30, 2014 5. Docket No. 030-33449 Reference No.	
6. Byproduct, source, and/or special nuclear material A. Any byproduct material with atomic numbers 3 through 83, except fluorine 18, strontium 89, yttrium 90, molybdenum 99, technetium 99m, indium 111, iodine 123, iodine 131, xenon 133, samarium 153, and thallium 201 B. Fluorine 18 C. Strontium 89 D. Yttrium 90 E. Molybdenum 99 F. Technetium 99m G. Indium 111 H. Iodine 123 I. Iodine 131 J. Xenon 133 K. Samarium 153 L. Thallium 201 M. Any byproduct material permitted by 10 CFR 31.11	7. Chemical and/or physical form A. Any B. Any C. Any D. Any E. Any F. Any G. Any H. Any I. Any J. Any K. Any L. Any M. Prepackaged units for <u>in vitro</u> diagnostic tests	8. Maximum amount that licensee may possess at any one time under this license A. 200 millicuries per radionuclide and 2 curie total B. 1 curie C. 40 millicuries D. 500 millicuries E. 100 curies F. 100 curies G. 300 millicuries H. 50 millicuries I. 2.5 curies J. 1.5 curie K. 750 millicuries L. 1 curie M. 50 millicuries

MATERIALS LICENSE SUPPLEMENTARY SHEET

License Number
44-30124-01MD

Docket or Reference Number
030-33449

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| 6. Byproduct, source, and/or special nuclear material | 7. Chemical and/or physical form | 8. Maximum amount that licensee may possess at any one time under this license |
| N. Any byproduct material permitted by 10 CFR 35.65(a) | N. Sealed Sources (E.I. DuPont Model NES-356; International Isotopes Model BM03-57L and BM03-57A; International Isotopes Idaho, Inc., Model, BM06-37, BM06-90, BM08-37, BM08-371, BM06E series, and BM06S series; North American Scientific Model MED3550, MED3400, MED3402, MED3550, and MED3503; and Isotope Products Laboratories Model GF-152-R2, GF-152-R3, RV-XXX series, and EG-133-MX6-5U) | N. 100 millicuries |
| O. Any byproduct material permitted by 10 CFR 35.400 | O. Sealed Sources (Bard Brachytherapy Model STM 1251; IsoAid, LLC Model IAI-125A; Teragenics Theraseed Model 200; North American Scientific Model MED3631 and MED 3633) | O. 500 millicuries |
| P. Any byproduct material with atomic numbers 2 through 83 | P. Analytical samples | P. As needed (see item 9.P.) |
| Q. Depleted Uranium | Q. Solid | Q. 400 kilograms |

9. Authorized use:

- A. through L. Preparation and distribution of radioactive drugs and radiochemicals for medical use in accordance with 10 CFR 32.72 and for non-medical use to authorized recipients.
- D. and E. Redistribution of used and unused molybdenum 99/technetium 99m generators to authorized recipients for medical use in accordance with 10 CFR 32.72 and for non-medical use to authorized recipients.
- M. Redistribution to specific licensees or general licensees pursuant to 10 CFR 31.11 provided the packaging and labeling remain unchanged.

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- N. Calibration and checking of the licensee's instruments. Redistribution of sealed sources initially distributed by a manufacturer licensed pursuant to 10 CFR 32.74 to authorized recipients and to authorized recipients for non-medical use.
- O. Redistribution for medical use of sealed sources initially distributed by a manufacturer licensed pursuant to 10 CFR 32.74. Redistribution for non-medical use of sealed sources that have been registered either with the Nuclear Regulatory Commission under 10 CFR 32.210 or with an Agreement State and have been distributed in accordance with a Commission or Agreement State specific license authorizing distribution to persons specifically authorized by a Commission or Agreement State license to receive, possess and use the devices.
- P. For possession incident to the performance of leak testing of customers' sealed sources.
- Q. Shielding for molybdenum-99/technetium-99m generators.

CONDITIONS

10. Licensed material may be used or stored only at the licensee's facilities located at 9 Krupp Drive, Williston, Vermont.
11. Licensed material shall be used by, or under the supervision of:
- A. A pharmacist working or designated as an authorized nuclear pharmacist in accordance with 10 CFR 32.72(b)(2)(i) or (4).
 - B. Authorized nuclear pharmacists: William M. Chatoff, R.Ph., BCNP; Thomas DeFranco, R.Ph.; Jeffrey Letendre, R.Ph., BCNP; Howard Stoops R.Ph., BCNP; Sandra Geyser R.Ph., BCNP; Walter Adichie, R.Ph., BCNP; Shawn Lorraine, R.Ph.; Thomas W. Frank, R.Ph., Elliot Rosario, R.Ph., Rafael Rueda, R.Ph., Joel A. Cordero, R.Ph., Dory Seitz, R.Ph., Stephen G. Spochak, R.Ph. or Richard Sucece, R.Ph.
12. The Radiation Safety Officer for this license is Thomas DeFranco, R.Ph.
13. In addition to the possession limits in Item 8, the licensee shall further restrict the possession of licensed material to quantities below the minimum limit specified in 10 CFR 30.35(d) for establishing decommissioning financial assurance.
14. This license does not authorize commercial distribution of licensed material to persons generally licensed pursuant to 10 CFR Part 31 or to persons exempt from licensing pursuant to 10 CFR 30.14 through 30.21, inclusive, or equivalent regulations of any Agreement State.

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15. A. Sealed sources shall be tested for leakage and/or contamination at intervals not to exceed six months or at the intervals specified in the certificate of registration issued by the U.S. Nuclear Regulatory Commission under 10 CFR 32.210 or under equivalent regulations of an Agreement State.
- B. Notwithstanding Paragraph A of this Condition, sealed sources designed to primarily emit alpha particles shall be tested for leakage and/or contamination at intervals not to exceed 3 months.
- C. In the absence of a certificate from a transferor indicating that a leak test has been made within the intervals specified in the certificate of registration issued by the U.S. Nuclear Regulatory Commission under 10 CFR 32.210 or under equivalent regulations of an Agreement State, prior to the transfer, a sealed source received from another person shall not be put into use until tested and the test results received.
- D. Sealed sources need not be tested if they contain only hydrogen-3; or they contain only a radioactive gas; or the half-life of the isotope is 30 days or less; or they contain not more than 100 microcuries of beta- and/or gamma-emitting material or not more than 10 microcuries of alpha-emitting material.
- E. Sealed sources need not be tested if they are in storage and are not being used; however, when they are removed from storage for use or transferred to another person and have not been tested within the required leak test interval, they shall be tested before use or transfer. No sealed source shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.
- F. The leak test shall be capable of detecting the presence of 0.005 microcurie (185 becquerels) of radioactive material on the test sample. If the test reveals the presence of 0.005 microcurie (185 becquerels) or more of removable contamination, a report shall be filed with the U.S. Nuclear Regulatory Commission in accordance with 10 CFR 30.50(c)(2), and the source shall be removed immediately from service and decontaminated, repaired, or disposed of in accordance with Commission regulations.
- G. Tests for leakage and/or contamination, including leak test sample collection and analysis, shall be performed by the licensee or by other persons specifically licensed by the U.S. Nuclear Regulatory Commission or an Agreement State to perform such services.
- H. Records of leak test results shall be kept in units of microcuries and shall be maintained for 5 years.
16. Sealed sources or detector cells containing licensed material shall not be opened or sources removed from source holders by the licensee.

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17. The licensee shall conduct a physical inventory every six months, or at other intervals approved by the U.S. Nuclear Regulatory Commission, to account for all sources and/or devices received and possessed under the license. Records of inventories shall be maintained for 5 years from the date of each inventory and shall include the radionuclides, quantities, manufacturer's name and model numbers, and the date of the inventory.
18. The licensee is authorized to hold byproduct material with a physical half-life of less than or equal to 120 days for decay-in-storage before disposal without regard to its radioactivity if the licensee:
- A. Monitors byproduct material at the surface before disposal and determines that its radioactivity cannot be distinguished from the background radiation level with an appropriate radiation detection survey meter set on its most sensitive scale and with no interposed shielding; and
 - B. Removes or obliterates all radiation labels, except for radiation labels on materials that are within containers and that will be managed as biomedical waste after they have been released from the licensee; and
 - C. Maintains records of the disposal of licensed materials for 3 years. The record must include the date of disposal, the survey instrument used, the background radiation level, the radiation level measured at the surface of each waste container, and the name of the individual who performed the disposal.
19. The licensee is authorized to retrieve, receive and dispose of radioactive waste from its customers limited to radiopharmacy supplied syringes and vials and their contents.
20. Radioactive waste may be picked up from the licensee's customers and disposed of in accordance with the statements, representations and procedures in the letter dated March 14, 1994.
21. The licensee is authorized to transport licensed material in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."

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22. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The U.S. Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.

- A. Letter dated March 14, 1994 (ML090890396)
- B. Letter dated April 29, 1994 (ML090890393)
- C. Letter dated May 10, 1994 (ML090890378)
- D. Letter dated May 25, 1994 (ML090890214)
- E. Letter dated July 8, 1994 (ML090890361)
- F. Letter dated September 6, 1994 (ML090890245)
- G. Letter dated September 10, 1995 (ML090890146)
- H. Letter dated December 7, 1995 (ML090890122)
- I. Letter dated January 5, 1996 (ML090890076)
- J. Letter dated March 4, 2002 (ML020650454)
- K. Letter dated April 27, 2009 (ML091180613)
- L. Letter dated May 18, 2009 (ML091390699)



For the U.S. Nuclear Regulatory Commission

Date October 6, 2009

By

Original signed by Elizabeth Ullrich

Elizabeth Ullrich
Commercial and R&D Branch
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406