



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

October 2, 2009

Mr. Jon A. Franke, Vice President  
Crystal River Nuclear Plant (NA1B)  
ATTN: Supervisor, Licensing & Regulatory Programs  
15760 W. Power Line Street  
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT – PARTIAL CLOSE  
OUT AND REQUEST FOR ADDITIONAL INFORMATION RELATED TO  
GENERIC LETTER 2004-02 (TAC NO. MC4678)

Dear Mr. Franke:

By letters dated February 29, 2008, and February 26, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML080640544 and ML090620392, respectively), the Florida Power Corporation (the licensee) submitted supplemental responses to Generic Letter (GL) 2004-02, "Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized-Water Reactors," for Crystal River Nuclear Plant, Unit 3 (CR-3). The Nuclear Regulatory Commission (NRC) staff has reviewed the supplemental submittals with a process that involved a detailed review by a team of 10 subject matter experts and a focus on the review areas described in the NRC's "Content Guide for Generic Letter 2004-02 Supplemental Responses," (ADAMS Accession No. ML073110389). The CR-3 review process also included an additional review of the submittals by an independent team of the subject matter experts that focused on whether the licensee has demonstrated that CR-3's corrective actions for GL 2004-02 are adequate.

The NRC staff has no further questions at this time regarding the performance of the emergency core cooling system strainer at CR-3. This conclusion is directly attributable to the detailed and comprehensive information provided to the NRC staff for review, and to the licensee's use of a testing and evaluation process that allowed the NRC staff to ascertain compliance with the applicable regulations.

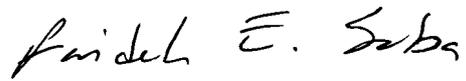
However, an additional set of information is needed to close the GL 2004-02 review for CR-3. The requirement for this information, regarding in-vessel downstream effects and the use of Topical Report WCAP-16793, "Evaluation of Long-Term Cooling Considering Particulate, Fibrous and Chemical Debris in the Recirculating Fluid," is adequately captured by the commitment made in the licensee's above-mentioned letter dated February 26, 2009. Since the under development WCAP-16793 Revision 1 is the best available information to the licensee regarding plant susceptibility to this issue, the NRC recommends: 1) the licensee review in-vessel downstream effects at CR-3 against the criteria developed in the latest available revision to WCAP-16793 and, 2) inform the NRC of the planned actions at CR-3, if the results of the evaluation show that CR-3 is not bounded by the these criteria. While the NRC has not yet issued a final safety evaluation for WCAP-16793, it is prudent for licensees who plan to rely on this topical report to evaluate their plant-specific conditions against the latest available report.

J. Franke

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If you have any questions regarding this matter, please contact me at (301) 415-1447.

Sincerely,

A handwritten signature in black ink that reads "Farideh E. Saba". The signature is written in a cursive style with a large, stylized initial 'F'.

Farideh E. Saba, Senior Project Manager  
Plant Licensing Branch  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-302

cc: Distribution via Listserv

J. Franke

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Sincerely,

*/ra/*

Farideh E. Saba, Senior Project Manager  
Plant Licensing Branch  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

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NRR-088

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