

2009 POINT BEACH NUCLEAR PLANT

INITIAL EXAMINATION

ADMINISTRATIVE FILES

Facility: <u>Point Beach Nuclear Plant, Units 1 and 2</u> Date of Examination: <u>Weeks of 5/11 and 18/ 2009</u>		
Developed by: Written - Facility <input checked="" type="checkbox"/> NRC <input type="checkbox"/> // Operating - Facility <input checked="" type="checkbox"/> NRC <input type="checkbox"/>		
Target Date*	Task Description (Reference)	Chief Examiner's Initials
-180	1. Examination administration date confirmed (C.1.a; C.2.a and b)	meb
-120	2. NRC examiners and facility contact assigned (C.1.d; C.2.e)	meb
-120	3. Facility contact briefed on security and other requirements (C.2.c)	meb
-120	4. Corporate notification letter sent (C.2.d)	meb
[-90]	[5. Reference material due (C.1.e; C.3.c; Attachment 3)]	NA
{-75}	6. Integrated examination outline(s) due, including Forms ES-201-2, ES-201-3, ES-301-1, ES-301-2, ES-301-5, ES-D-1's, ES-401-1/2, ES-401-3, and ES-401-4, as applicable (C.1.e and f; C.3.d)	meb
{-70}	{7. Examination outline(s) reviewed by NRC and feedback provided to facility licensee (C.2.h; C.3.e)}	meb
{-45}	8. Proposed examinations (including written, walk-through JPMs, and scenarios, as applicable), supporting documentation (including Forms ES-301-3, ES-301-4, ES-301-5, ES-301-6, and ES-401-6, and any Form ES-201-3 updates), and reference materials due (C.1.e, f, g and h; C.3.d)	meb
-30	9. Preliminary license applications (NRC Form 398's) due (C.1.i; C.2.g; ES-202)	meb
-14	10. Final license applications due and Form ES-201-4 prepared (C.1.i; C.2.i; ES-202)	meb
-14	11. Examination approved by NRC supervisor for facility licensee review (C.2.h; C.3.f)	meb
-14	12. Examinations reviewed with facility licensee (C.1.j; C.2.f and h; C.3.g)	meb
-7	13. Written examinations and operating tests approved by NRC supervisor (C.2.i; C.3.h)	meb
-7	14. Final applications reviewed; 1 or 2 (if >10) applications audited to confirm qualifications / eligibility; and examination approval and waiver letters sent (C.2.i; Attachment 5; ES-202, C.2.e; ES-204)	meb
-7	15. Proctoring/written exam administration guidelines reviewed with facility licensee (C.3.k)	meb
-7	16. Approved scenarios, job performance measures, and questions distributed to NRC examiners (C.3.i)	meb
<p>* Target dates are generally based on facility-prepared examinations and are keyed to the examination date identified in the corporate notification letter. They are for planning purposes and may be adjusted on a case-by-case basis in coordination with the facility licensee. [Applies only] {Does not apply} to examinations prepared by the NRC.</p>		

Master Security Agreement

The Master Security Agreement is used for those personnel having both knowledge of Exam Sensitive Material and unrestricted access to Primary and Secondary Containment.

1. Pre-Examination (Review FP-T-SAT-71, Attachment 1 for pre-job briefing requirements)

I acknowledge that I have acquired specialized knowledge about the ILT 2009 NRC examination(s) scheduled for the date(s) of May 4-15, 2009 as of the date of my signature. I agree that I will not knowingly divulge any information about these examinations to any persons who have not been authorized by an Operations Training Supervisor. I understand that I am not to instruct or provide performance feedback to those individuals scheduled to be administered these examinations from this date until completion of examination administration. I also understand that I am not to evaluate individuals scheduled to be administered these examinations from this date until the date of administration. Acting as a simulator booth operator or communicator is acceptable if I do not select the training content or provide direct or indirect feedback to an examinee. Furthermore, I am aware of the physical security measures and requirements (as documented in procedures) and understand that violation of the conditions of this agreement may result in cancellation of the examinations and/or an enforcement action against my facility or me. I will immediately report to the Exam Project Manager any indications or suggestions that examination security may have been compromised.

2. Post-Examination

To the best of my knowledge, I did not divulge to any unauthorized persons any information concerning the examinations administered during the date(s) of May 11-18, 2009. From the date that I entered into this security agreement until the completion of examination administration, I did not instruct, evaluate, or provide performance feedback to those individuals who were administered these examinations.

	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
1.	Randall C. Amundson	GSOT	R.C. Amundson	9/19/08	R.C. Amundson	5/18/09	
2.	Sam Hall	Ops Instructor	SH	9/19/08	SH	5/18/09	
3.	Andrew Zimmers	OPS INSTRUCTOR/ERM	AZ	9/19/08	AZ	5/18/09	
4.	RICK PARUATO	SIMULATOR	R. Paruato	12/17/08	R. Paruato	5/28/09	

Retention: Life of Plant
Retain in: Exam File

Point Beach Nuclear Plant
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	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
5.	Robert J. McLaughlin	Simulator Hardware Eng.	<i>Robert J. McLaughlin</i>	12/17/08	<i>Robert J. McLaughlin</i>	5/19/09	
6.	Ken Draska	Simulator Tester	<i>Ken Draska</i>	12/17/08	<i>Ken Draska</i>	5/19/09	
7.	Mike Angle	Ops Instructor	<i>Mike Angle</i>	12/19/08	<i>Mike Angle</i>	5/27/09	
8.	THOMAS G. LARSON	Exam Prep/OPS SUPERVISOR	<i>Thomas G. Larson</i>	1/13/09	<i>Thomas G. Larson</i>	5/19/09	
9.	Robert Shae Ferguson	Ops Instructor	<i>Robert Shae Ferguson</i>	4-8-09	<i>Robert Shae Ferguson</i>	5-19-09	
10.	ART MARRIS	OPS INSTRUCTOR	<i>Art Maris</i>	5/8/09	<i>Art Maris</i>		①
11.	Rich Baird	OPS TRAINING SUP	<i>Rich Baird</i>	5/8/9	<i>Rich Baird</i>	5-19-9	
12.	Joe Smith	SROC OPS INST.	<i>Joe Smith</i>	5/8/9	<i>Joe Smith</i>	5/20/9	
13.	Troy Wick	OPS INST	<i>Troy Wick</i>	5/8/9	<i>Troy Wick</i>	5/19/09	
14.							
15.							

NOTES: ① SIGNATURES ATTACHED, THEY WERE OBTAINED VIA FAX DUE TO THIS PERSON BEING OUT OF THE AREA. *OKJ* 5/29/09

Retention: Life of Plant
Retain in: Exam File

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	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
5.	Robert J. McEnachie	Simulator Hardware Eng.	<i>Robert J. McEnachie</i>	12/17/08	<i>Robert J. McEnachie</i>	5/19/09	
6.	Ken Daska	Simulator Tester	<i>K Daska</i>	12/17/08	<i>Ken Daska</i>	5/19/09	
7.	Mike Angle	Ops Instructor	<i>Mike Angle</i>	12/19/08	<i>Mike Angle</i>	5/27/09	
8.	TITANS G. LARSON	Exam PROBEOR OPS SUPERVISOR	<i>T. Larson</i>	1/13/09	<i>T. Larson</i>	5/19/09	
9.	Robert Shae Ferguson	Ops Instructor	<i>Robert Shae Ferguson</i>	4-9-09	<i>Robert Shae Ferguson</i>	5-19-09	
10.	ART MORRIS	OP S INSTRUCTOR	<i>Art Morris</i>	5/8/09	<i>Art Morris</i>	5/28/09	
11.	REN BARR	OPS TRAINING SUP	<i>Ren Barr</i>	5/8/9	<i>Ren Barr</i>	5-19-9	
12.	Joe Smith	SROC OPS INST.	<i>Joe Smith</i>	5/8/09	<i>Joe Smith</i>	5/20/09	
13.	Troy Wick	OPS INST	<i>Troy Wick</i>	5/8/9	<i>Troy Wick</i>	5/19/09	
14.							
15.							

NOTES:

Point Beach ~~Plant~~
Limited Security Agreement

The Limited Security Agreement is used for those personnel having limited knowledge of Exam Sensitive Material but **do not** have unrestricted access to Primary and Secondary Containment.

1. Pre-Examination (Review FP-T-SAT-71, Attachment A for pre-job briefing requirements)

I acknowledge that I have acquired specialized knowledge about the PBNP ILT NRC examination scheduled for the date(s) of May 4-15, 2009 as of the date of my signature. I agree that I will not knowingly divulge any information about this examination to any persons who have not been authorized by an Operations Training Supervisor. I understand that I am not to instruct or provide performance feedback to those individuals scheduled to be administered these examinations from this date until completion of examination administration. I also understand that I am not to evaluate individuals scheduled to be administered these examinations from this date until the date of administration. Acting as a simulator booth operator or communicator is acceptable if I do not select the training content or provide direct or indirect feedback to an examinee. Furthermore, I am aware of the physical security measures and requirements (as documented in procedures) and understand that violation of the conditions of this agreement may result in cancellation of the examination and/or an enforcement action against my facility or me. I will immediately report to the Exam Project Manager any indications or suggestions that examination security may have been compromised.

2. Post-Examination

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Furthermore, I agree to **NOT** discuss any aspects associated with the contents of this examination with **ANY** examinee until the completion of their examination administration. I further understand that violation of the conditions of this agreement may result in cancellation of the examination and/or enforcement action against the facility licensee or me.

	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
1.	<u>Charles Storm</u>	<u>SRO</u>	<u>[Signature]</u>	<u>4/14/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	
2.	<u>Michael Vance</u>	<u>NCO instructor</u>	<u>Michael Vance</u>	<u>4.15.09</u>	<u>Michael Vance</u>	<u>5.19.09</u>	
3.	<u>Joe Smith</u>	<u>SROC INSTRUCTOR</u>	<u>[Signature]</u>	<u>5/8/09</u>	<u>[Signature]</u>	<u>5/20/09</u>	
4.	<u>Troy Wick</u>	<u>OPS Instructor</u>	<u>[Signature]</u>	<u>5/8/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	
5.	<u>Art Morris</u>	<u>OPS INSTRUCTOR</u>	<u>[Signature]</u>	<u>5/8/09</u>	<u>[Signature]</u> (1)		(1)
6.	<u>RICH BAIRD</u>	<u>ILT SUPERVISOR</u>	<u>[Signature]</u>	<u>5/8/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	

Retention: Life of plant

Form retained in accordance with record retention schedule identified in NP 1.3.1.

Retain in: Exam file

(1) Signatures Attached, they were obtained via fax due to this person being out of the area. OK [Signature] 5/29/09

Point Beach ~~Plant~~ Plant
Limited Security Agreement

No. 0664

P. 3

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1. Pre-Examination (Review FP-T-SAT-71, Attachment A for pre-job briefing requirements)

I acknowledge that I have acquired specialized knowledge about the TBNP IOT NRC examination scheduled for the date(s) of May 4-15, 2009 as of the date of my signature. I agree that I will not knowingly divulge any information about this examination to any persons who have not been authorized by an Operations Training Supervisor. I understand that I am not to instruct or provide performance feedback to those individuals scheduled to be administered these examinations from this date until completion of examination administration. I also understand that I am not to evaluate individuals scheduled to be administered these examinations from this date until the date of administration. Acting as a simulator booth operator or communicator is acceptable if I do not select the training content or provide direct or indirect feedback to an examinee. Furthermore, I am aware of the physical security measures and requirements (as documented in procedures) and understand that violation of the conditions of this agreement may result in cancellation of the examination and/or an enforcement action against my facility or me. I will immediately report to the Exam Project Manager any indications or suggestions that examination security may have been compromised.

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1.	<u>Charles Storm</u>	<u>SRO</u>	<u>[Signature]</u>	<u>4/14/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	_____
2.	<u>Michael Vance</u>	<u>NCO instructor</u>	<u>Michael Vance</u>	<u>4.15.09</u>	<u>Michael Vance</u>	<u>5.19.09</u>	_____
3.	<u>Joe Smith</u>	<u>SROC INSTRUCTOR</u>	<u>[Signature]</u>	<u>5/8/09</u>	<u>[Signature]</u>	<u>5/20/09</u>	_____
4.	<u>Toy Wick</u>	<u>OPS Instructor</u>	<u>[Signature]</u>	<u>5/8/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	_____
5.	<u>ART MORRIS</u>	<u>OPS INSTRUCTOR</u>	<u>[Signature]</u>	<u>5/8/09</u>	<u>[Signature]</u>	<u>5/28/09</u>	_____
6.	<u>RICH BAIRD</u>	<u>ILT SUPERVISOR</u>	<u>[Signature]</u>	<u>5/8/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	_____

Retention: Life of plant

Retain in: Exam file

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MAY. 28. 2009 7:46AM

Point Beach Nuclear Plant
Limited Security Agreement

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1. Pre-Examination (Review FP-T-SAT-71, Attachment A for pre-job briefing requirements)

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	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
1.	<u>Thomas L. Larson</u>	<u>EPDS</u>	<u>[Signature]</u>	<u>9/19/08</u>	<u>[Signature]</u>	<u>5/13/09</u>	<u>①</u>
* 2.	<u>Jenny Heltmann</u>	<u>Office Assistant</u>	<u>[Signature]</u>	<u>11/5/08</u>	<u>[Signature]</u>	<u>5/19/09</u>	
3.	<u>Harley M'Daniel</u>	<u>Operations Instructor</u>	<u>[Signature]</u>	<u>12/22/08</u>	<u>[Signature]</u>	<u>5/19/09</u>	
* 4.	<u>Andy Fehrenkrug</u>	<u>Knowledge Exam Writer</u>	<u>[Signature]</u>	<u>01/12/09</u>	<u>[Signature]</u>	<u>5/20/09</u>	
* 5.	<u>Stephen Johnson</u>	<u>Knowledge / Exam Writer</u>	<u>[Signature]</u>	<u>01/12/09</u>	<u>[Signature]</u>	<u>5/20/09</u>	
* 6.	<u>Cristalle Walsh</u>	<u>Knowledge / Exam Writer</u>	<u>[Signature]</u>	<u>01/12/09</u>	<u>[Signature]</u>	<u>5/20/09</u>	

Retention: Life of plant

Retain in: Exam file

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	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
7.	<u>Mike Bressler</u>	<u>CO</u>	<u>[Signature]</u>	<u>1/15/9</u>	<u>[Signature]</u>	<u>5/19/09</u>	
8.	<u>SCOTT PHILLIPS</u>	<u>C.O.</u>	<u>[Signature]</u>	<u>1-15-9</u>	<u>[Signature]</u>	<u>5-19-9</u>	
9.	<u>Ricky L. Robbins</u>	<u>SRO</u>	<u>[Signature]</u>	<u>1-16-9</u>	<u>[Signature]</u>	<u>5-19-9</u>	
10.	<u>Doug Watry</u>	<u>CO</u>	<u>[Signature]</u>	<u>1-16-9</u>	<u>[Signature]</u>	<u>5-19-09</u>	
11.	<u>Todd Hansen</u>	<u>CO</u>	<u>[Signature]</u>	<u>1/14/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	
12.	<u>RICK MERKES</u>	<u>SRO</u>	<u>[Signature]</u>	<u>1/16/09</u>	<u>[Signature]</u>	<u>5/22/09</u>	
13.	<u>Daniel C Pond</u>	<u>CO</u>	<u>[Signature]</u>	<u>1/19/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	
14.	<u>Chuck Sizemore</u>	<u>MGR Fleet Ops TRN/oversight</u>	<u>[Signature]</u>	<u>1/23/09</u>	<u>[Signature]</u>	<u>5/26/0</u>	
15.	<u>PAUL SMITH</u>	<u>SRO</u>	<u>[Signature]</u>	<u>1/27/9</u>	<u>[Signature]</u>	<u>5/19/9</u>	

NOTES:

- * NOT ISSUED AN EXAM SECURITY badge.
- ① SIGNED ON TO MASTER SECURITY AGREEMENT EFFECTIVE 1/13/09.

Point Beach Nuclear Plant
Limited Security Agreement

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	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
1.	<u>MIKE MILAN</u>	<u>Ops Projects</u>	<u>[Signature]</u>	<u>11/28/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	
2.	<u>★ Scott Kohlmann</u>	<u>IT Analyst</u>	<u>[Signature]</u>	<u>2/16/09</u>	<u>[Signature]</u>	<u>5/21/09</u>	
3.	<u>Alan Reiff</u>	<u>RP Field Ops General Supervisor</u>	<u>[Signature]</u>	<u>2/17/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	
4.	<u>RICK WOOD</u>	<u>Rx Engineer</u>	<u>[Signature]</u>	<u>2/26/2009</u>	<u>[Signature]</u>	<u>5/19/2009</u>	
5.	<u>Douglas K Meaney</u>	<u>Control OP</u>	<u>[Signature]</u>	<u>3/24/09</u>	<u>[Signature]</u>	<u>5/20/09</u>	
6.	<u>STEVEN M. THILMONEY</u>	<u>CONTROL OP</u>	<u>[Signature]</u>	<u>3/04/09</u>	<u>[Signature]</u>	<u>5/20/09</u>	

Retention: Life of plant

Retain in: Exam file

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Point Beach Nuclear Plant
Limited Security Agreement

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	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
7.	<u>John D. Welch</u>	<u>Control Operator</u>	<u>[Signature]</u>	<u>3/4/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	_____
* 8.	<u>Jim Gading</u>	<u>Sys. Eng</u>	<u>[Signature]</u>	<u>3/6/09</u>	<u>[Signature]</u>	<u>5/19/09</u>	_____
* 9.	<u>Andy Bussiere</u>	<u>IT Analyst</u>	<u>[Signature]</u>	<u>3/11/09</u>	<u>[Signature]</u>	<u>5/17/09</u>	_____
10.	<u>Doug Lauterbur</u>	<u>Training Manager</u>	<u>[Signature]</u>	<u>3-18-09</u>	<u>[Signature]</u>	<u>5-19-09</u>	_____
11.	<u>Ken Lafountain</u>	<u>CO</u>	<u>[Signature]</u>	<u>4/2/9</u>	<u>[Signature]</u>	<u>5/19/9</u>	_____
12.	<u>Darrell Sheppman</u>	<u>CO</u>	<u>[Signature]</u>	<u>4/2/9</u>	<u>[Signature]</u>	<u>5-19-9</u>	_____
13.	<u>Michael LeGreve</u>	<u>PPCS Engineer</u>	<u>[Signature]</u>	<u>4/13/09</u>	<u>[Signature]</u>	<u>5-19-09</u>	_____
14.	<u>Robert Post</u>	<u>CO</u>	<u>[Signature]</u>	<u>4/17/09</u>	<u>[Signature]</u>	<u>5/20/09</u>	_____
15.	<u>Jimmy L. Barnard</u>	<u>ILT Instructor</u>	<u>[Signature]</u>	<u>5/4/18/09</u>	<u>[Signature]</u>	<u>5/18/09</u>	_____

NOTES:

* NOT ISSUED AN EXAM SECURITY red badge due TO LIMITED scope of EXAM knowledge.

Tier/Group	Randomly Selected K/A	Reason for Rejection
1/1 (RO)	065.K1	9/23/08: No KA >2.5 listed for K.1 or K.2. Swapped 065.K1 to 065.K3 and W/E11.K3 to W/E11.K1.
1/2 (RO)	001.K1	9/23/08: Selected AK1.14 which applies to B&W plant. Randomly re-selected
2/1 (RO)	007.K2	9/23/08: No KA listed. Swapped 007.K2 to 007.K1 and 006.K1 to 006.K2.
2/1 (RO)	076.K6	9/23/08: No KA >2.5 listed. Randomly selected new category for 076 Service Water
2/1 (RO)	078.A1	9/23/08: No KA listed for A.1. Randomly selected new category for areas with KA >2.5
2/2 (RO)	017.K2	9/23/08: No KA >2.5. Selected A.3 because no systems were selected in that category
2/2 (RO)	056.K5	9/23/08: No KA >2.5. Randomly selected new category for areas with KA >2.5
1/1 (SRO)	027.A2.09	12/1/08: Unable to write a valid question for selected system and topic. With the plant design, in order to make reactor power change enough for an SRO question, the plant conditions wouldn't be operationally valid. Reselected Station Blackout 055.EA2.06.
2/1 (RO)	103.A2.04	12/1/08: Ability 04 under A2 does not apply to PBNP. We have no containment evacuation alarms or automatic functions which would allow us to write a question to the K/A. Randomly selected K3.01 within the system.
2/2 (RO)	028.K4.01	1/14/2009: Per FSAR section 5.3.2.4 Post Accident Containment Venting, the NRC dropped hydrogen release associated with a LOCA from 10CFR50.44. These systems are no longer required for design basis, but are still referenced for situations beyond design basis. Randomly reselected K/A 055.K3.01
2/1 (SRO)	006.A2.10	1/15/2009: Dropped 10 and randomly selected area 12 under A2 to maintain sample plan consistency. Rejected 10 due to lack of procedural guidance regarding restoring low boron concentration in SIS to match second half of K/A.
1/2 (RO)	074.K3.06	1/15/2009: Dropped K/A due to no procedural guidance in core cooling regarding the automatic opening of PORV's. There was only guidance on manual operation of the PORV's. Randomly reselected a new system, but kept K3 for sample plan consistency.
1/1 (RO)	026.02.04.01	1/21/2009: PBNP does not have immediate actions associated with a loss of component cooling water. To maintain sample plan consistency, re-selected generic K/A 2.1.31.
1/2 (RO)	001.K1.21	4/1/2009: Could not write a difficult enough question with plausible distracters for this K/A. Reselected Emergency and Abnormal Plant Evolution 003 Dropped and kept K1 to maintain sample plan. Selected item AK1.22.
2/1 (RO)	022.A1.03	4/1/2009: Dropped K/A due to LOD 1 question and cannot write an appropriate level of difficulty. To maintain sample plan we randomly selected K2.01.

Facility:		Date of Exam:	5/18/09	Exam Level:	RO X	SRO X
Item Description	Initials					
	a	b	c			
1. Clean answer sheets copied before grading		RCA	MFB			
2. Answer key changes and question deletions justified and documented ***	N/A 	N/A RCA	N/A MFB			
3. Applicants' scores checked for addition errors (reviewers spot check > 25% of examinations)		RCA	MFB			
4. Grading for all borderline cases (80 ±2% overall and 70 or 80, as applicable, ±4% on the SRO-only) reviewed in detail		RCA	MFB			
5. All other failing examinations checked to ensure that grades are justified	N/A 	N/A RCA	N/A MFB			
6. Performance on missed questions checked for training deficiencies and wording problems; evaluate validity of questions missed by half or more of the applicants		RCA	MFB			
Printed Name/Signature		Date				
a. Grader	Andrew Zimmers	5/21/09				
b. Facility Reviewer(*)	R.C. Amundson	5/21/09				
c. NRC Chief Examiner (*)	Michael E. Bielby /	5/27/09				
d. NRC Supervisor (*)	Hironori Peterson	6/8/09				
(*) The facility reviewer's signature is not applicable for examinations graded by the NRC; two independent NRC reviews are required.						

** NO EXAMS CHECKED DUE TO ALL EXAMINEES PASSING

*** No answer key changes needed

Point Beach May 2009 Exam

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6. B/M/N	7. U/E/S	8. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/units	Backward	Q=K/A	SRO Only			
1	H	2												B	S	
2	H	3												N	E	In the question stem, add the word "INITIALLY" before the word "entering." Reason: As the PORV opens the PRT pressure will increase, and the temperature of the saturated steam-water mixture will increase as the PRT pressure increases. <u>RESOLUTION</u> : Comment incorporated. <u>NOTE</u> : Question requires a reference.
3+	H	2												B	S	
4	F	2												N	S	
5	H	3												B	S	

Instructions

[Refer to Section D of ES-401 and Appendix B for additional information regarding each of the following concepts.]

- Enter the level of knowledge (LOK) of each question as either (F)undamental or (H)igher cognitive level.
- Enter the level of difficulty (LOD) of each question using a 1 – 5 (easy – difficult) rating scale (questions in the 2 – 4 range are acceptable).
- Check the appropriate box if a psychometric flaw is identified:
 - The stem lacks sufficient focus to elicit the correct answer (e.g., unclear intent, more information is needed, or too much needless information).
 - The stem or distractors contain cues (i.e., clues, specific determiners, phrasing, length, etc).
 - The answer choices are a collection of unrelated true/false statements.
 - The distractors are not credible; single implausible distractors should be repaired, more than one is unacceptable.
 - One or more distractors is (are) partially correct (e.g., if the applicant can make unstated assumptions that are not contradicted by stem).
- Check the appropriate box if a job content error is identified:
 - The question is not linked to the job requirements (i.e., the question has a valid K/A but, as written, is not operational in content).
 - The question requires the recall of knowledge that is too specific for the closed reference test mode (i.e., it is not required to be known from memory).
 - The question contains data with an unrealistic level of accuracy or inconsistent units (e.g., panel meter in percent with question in gallons).
 - The question requires reverse logic or application compared to the job requirements.
- Check questions that are sampled for conformance with the approved K/A and those that are *designated SRO-only* (K/A and license level mismatches are unacceptable).
- Enter question source: (B)ank, (M)odified, or (N)ew. Check that (M)odified questions meet criteria of ES-401 Section D.2.f.
- Based on the reviewer's judgment, is the question as written (U)nsatisfactory (requiring repair or replacement), in need of (E)ditorial enhancement, or (S)atisfactory?
- At a minimum, explain any "U" ratings (e.g., how the Appendix B psychometric attributes are not being met).

A "+" in the "Q#" column indicates that question was reviewed as part of the representative sample of 30 questions.

Point Beach May 2009 Exam

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6. B/M/N	7. U/E/S	8. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/units	Backward	Q=K/A	SRO Only			
6+	H	3				X								N	E	To make distractor B more plausible (i.e., to add a condition inside containment), replace distractor B with; "Letdown Relief Valve lifting." <u>RESOLUTION:</u> Comment incorporated.
7	H	1												B	U	LOD = 1, that both CCW pumps are running if both 4 KV safeguards buses are energized with breaker indicating lights indicating closed. Suggest that you re-phrase question such that one EDG does not start, and ask what the breaker indications are (i.e., RED or GREEN) and ask if each CCW pump is running or not.. <u>RESOLUTION:</u> Question was replaced. The replaced question is Bank, Higher cognitive level, and has same K/A. In the replaced question, need to verify that the 4 th bulleted item in the question stem says that 4 KV Bus 1A05 and 1B03 480 VAC Safeguards bus are subsequently reenergized.
8	H	3												B	S	
9+	F	3												B	E	In the question stem, change "the word "should" to "is required to." <u>RESOLUTION:</u> Comment incorporated.
10	F	3												N	E	Editorial: In distractor C, change the word "an" to "a." <u>RESOLUTION:</u> Comment incorporated.
11	F	3												N	E	1) In the question stem, change the question to: "Which is the FIRST action the crew is required to take?" <u>Reason:</u> After the TD AFW pump is secured (distractor D), the pump casing could be vented (distractor A) per the ARP. 2) In the first sentence in the question stem, add that the event is "on Unit 1." <u>RESOLUTION:</u> 1) Need to fully capitalize the word "FIRST." 2) Comment incorporated.
12	H	3												N	E	In the last sentence in the question stem, add "and why?" at the end of the sentence. <u>RESOLUTION:</u> Comment incorporated.
13+	H	3												N	E	1) Need to add to question stem that Unit 2 is at RTP. 2) Need to add to question stem that no LCOs are initially in effect for Unit 2. <u>RESOLUTION:</u> Comments incorporated.

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6. B/M/N	7. U/E/S	8. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/units	Back-ward	Q=K/A	SRO Only			
14	H	2												N	E	In distractor C, change to "PZR PORVs and safeties." <u>RESOLUTION:</u> Comment incorporated.
15	F	2												B	S	
16+	H	3						X						B	E	1) Question is at the SRO level as a selection of appropriate procedure question (per 10CFR55.43(b)(5)), unless there is an RO Learning objective related to this issue. 2) In question stem, change the word "should" to "is required to." <u>RESOLUTION:</u> 1) There is an RO Learning Objective 031.02.LP0465.001 related to this issue. 2) Comment incorporated.
17	H	3												B	E	In distractor C, do not include another plant condition (i.e., no SI pumps running) in a distractor, since it becomes confusing whether this condition applies to all the distractors. If no SI pumps are running, distractor B would be the correct answer. <u>RESOLUTION:</u> Comment incorporated.
18	F	3												N	S	
19+	F						X							N	U	Distractor A and C are not plausible that the greatest change in reactivity for a continuous rod withdrawal event would be an event with the control bank near the top of the core instead of the middle of the core. <u>RESOLUTION:</u> Question was replaced. The replaced question is New, Fundamental, and has a different K/A. In the replacement question, need to verify that in the question stem, the word "calculated" was changed to "determined." Also, need to verify that the skyscraper is still satisfied with the different K/A.
20	F	3												B	S	
21	F	3												B	S	
22	F	3						X						B	S	
23+	H	3												N	S	
24	H	3												B	E	Change the first part of the distractors such that there are only two possible answers to the first part of the question (e.g., 10 steps BELOW or 10 steps above). Otherwise, one only needs to answer the first part of the question to get the correct answer. <u>RESOLUTION:</u> Comment incorporated.
25	H	3												B	S	

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6. B/M/N	7. U/E/S	8. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/units	Backward	Q=K/A	SRO Only			
26+	H	2												N	E	Question is at the Higher cognitive level instead of at the Fundamental level. <u>RESOLUTION:</u> Comment incorporated.
27	F	2				X								N	E	1) Question is at the SRO level as a Tech Spec Bases question (per 10CFR55.43(b)(2)), unless there is an RO Learning objective related to this issue. 2) Distractor A (remove accumulator relief valve) is not plausible. <u>RESOLUTION:</u> 1) There is an RO Learning Objective 055.01.LP0272.001 related to this issue. 2) Distractor was replaced.
28	H	3												N	S	
29+	H	3												N	S	<u>NOTE:</u> Question requires a reference.
30	F	2												N	S	
31	H	3				X								B	E	Distractor A is not plausible that both SI Accumulator outlet valves would be fed from the same 480V safeguards bus. <u>RESOLUTION:</u> Replaced distractor A.
32	F	3												N	S	
33+	H	3												B	S	
34	F	2				X								B	E	Distractor C is not plausible that the PZR spray bypass flow is there to minimize the peak pressure during a design basis pressure transient. Change to: "Prevents thermal binding of spray valves." <u>RESOLUTION:</u> Comment incorporated.
35	F	2												B	S	
36+	H	3												M	S	
37	F	3												B	S	
38	H	1												N	U	LOD = 1 that an RCS leak will result in an increase in containment humidity and an increase in containment radiation levels. <u>RESOLUTION:</u> Question was replaced. The replaced question is New, Higher cognitive level, and has a different K/A. Need to verify that the skyscraper is still satisfied with the different K/A.
39+	H	3												N	E	In distractors B, C, and D, re-word similar to wording in distractor A (i.e., Isolate the ...). <u>RESOLUTION:</u> Comment incorporated.

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6. B/M/N	7. U/E/S	8. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/units	Back-ward	Q=K/A	SRO Only			
40	H	3												N	S	Question is at the SRO level as a selection of appropriate procedure question (per 10CFR55.43(b)(5)), unless there is an RO Learning objective related to this issue. RESOLUTION: There is an RO Learning Objective 031.02.LP0465.001 related to this issue.
41	F	3												B	S	
42	H	3												N	S	
43+	H	3												B	S	
44	H	1				X								B	U	1) LOD = 1 that with FW reg valve controller in Manual at 0%, that the valve will stay closed when the Reactor Trip breakers are closed. 2) Distractor B is not plausible (controller in Manual at 40%), because if this distractor were correct, then distractor D (controller in Manual at 0%) would also logically be correct. RESOLUTION: : Question was replaced. The replaced question is New, Higher cognitive level, and has the same K/A. In the replacement question, need to verify that the question was changed to ask "Which of the following would allow manually opening the Main Feed Regulating Valves?" (instead of "... must be completed to manually open ...").
45	H	3												N	S	
46+	H	2				X								B	E	Distractor D is not plausible to stop AFW flow to each SG. Suggest changing distractor to raise AFW to whatever AFW flow is specified in the CSP-S.1 ATWS procedure. RESOLUTION: Comment incorporated.
47	F	3												N	S	1) Question is at the SRO level as a Tech Spec Bases question (per 10CFR55.43(b)(2)), unless there is an RO Learning objective related to this issue. 2) The Tech Spec Bases for the AFW system only discusses a backup pneumatic supply for the AFW recirculation AOV, not for the AFW pump flow control valves. Based on this, why is distractor C the correct answer? RESOLUTION: 1) There are RO Learning Objectives 052.05.LP0159.005, 052.05.LP0159.006, and 052.05.LP0159.011 related to this issue. 2) The same nitrogen backup bottle supplies both the AFW recirculation AOV and the AFW pump FCV and the question stem says that the leak is unisolable.
48	F	3												B	S	

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws					5. Other		6. B/M/N	7. U/E/S	8. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/units	Back-ward	Q=K/A	SRO Only				
49+	F	2													B	E	To increase the LOD, change question such that there is a loss of Bus 1A01, and pick 1P-27C Heater Drain Pump as the correct answer (would need to also change distractor D). RESOLUTION: Comment incorporated.
50	F	3													B	S	
51	F	2													B	E	Editorial: Re-format distractors A and C into two sentences instead of one. RESOLUTION: Comment incorporated.
52	H	3													B	E	Editorial: In distractor B, delete the word "now." RESOLUTION: Comment incorporated.
53+	F	2													B	S	
54	H	3													B	S	
55	H	3	X												B	E	In the question stem, need to add that the RCS is vented to containment (i.e., so that the RCS is not intact). RESOLUTION: Comment incorporated. Need to verify that comment was incorporated.
56+	F	3													N	S	
57	H	2				X									B	U	Distractors A and B are not plausible that a single power range channel failure will cause a reactor trip at power. RESOLUTION: Replaced distractors A and B, and changed distractor D to make a new correct answer.
58	H	2													B	S	
59+	H	2													B	S	
60	H	3													N	U	There is no correct answer. The correct answer should: "Aux Governor will actuate and the reactor will not trip," since the Aux Governor overspeed sensor will actuate at 103% and momentarily close the turbine governor valves, and thus prevent the IOPS overspeed trip at 104%. RESOLUTION: Changed distractors such that the correct answer is "Aux Governor will actuate and the reactor will not trip." Need to verify that distractor B was changed to say that "... the reactor will trip."

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6. B/M/N	7. U/E/S	8. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/units	Back-ward	Q=K/A	SRO Only			
61	F	3	X											B	U	In the question stem, add the word "momentarily" before the word "LIT" in the phrase "... Low Suction Pressure Timer is momentarily LIT." <u>RESOLUTION:</u> Comment incorporated. Need to verify that comment was incorporated.
62	F	2												B	E	Change question stem to ask what condition would isolate the discharge to the CW system. <u>RESOLUTION:</u> Comment incorporated.
63+	F	3				X								B	S	
64	H	2				X								B	U	1) Distractors A and B are not plausible that the ice melt valve would be closed to get rid of ice in the forebay. 2) Add "U2" to the titles of the two columns in the question stem. 3) Is distractor D the correct answer? The ice melt valve is a 10 inch diameter valve, while the valve to the lake is a 14 inch diameter valve. Thus, the flow area for the ice melt valve is only approximately one-half that of the flow area for the valve going to the lake. With the valve to the lake fully closed and the ice melt valve to the forebay fully open, water make backup from the circulating water return line and overflow out into the plant area outside. <u>RESOLUTION:</u> 1) Distractors replaced. 2) Comment incorporated. 3) Need explanation why distractor D is the correct answer.
65	F	3												N	S	
66+	H	3				X								B	E	1) To make distractor C more plausible, change the second sentence of distractor C to say: You must reactivate your license by standing two (2) additional 8-hour shifts in any CO position." 2) In the first sentence of the question stem, add that you were assigned to the WCC on 9/1/08. 3) In the sentence in the question stem before the list of 8-hours shifts worked, add that all the hours were on a Unit that was at power. <u>RESOLUTION:</u> 1) Comment incorporated. Need to verify that the word "additional" was added to distractor C. 2) and 3) Comments incorporated.
67	F	3												B	S	
68	F	3												N	S	

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws					5. Other		6. B/M/N	7. U/E/S	8. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/units	Backward	Q=K/A	SRO Only				
69+	F	3				X									N	E	Does the RCS heatup procedure allow realigning the RHR pumps to the LHSI mode with the PZR solid as described in the question stem? <u>RESOLUTION:</u> In the question stem, deleted the words "and being aligned for LHSI."
70	F	3													B	S	
71	F	3													M	U	Only need Department Manager permission (with Rad Protection Manager concurrence) to go up to 4500 mrem. Suggest changing question to ask what the maximum dose that a First Manager can approve. (4000 mrem). <u>RESOLUTION:</u> Comment incorporated.
72	F	3													B	S	
73+	F	3													N	S	
74	H	3													B	S	
75	H	3													N	E	1) Can get the correct answer for the wrong reason by using the 1 ft. below flange curve and not using the 1.3 multiplication factor. 2) Editorial: In the 1 st sentence of the question stem, add the word "the" before the word "following." <u>RESOLUTION:</u> 1) In the question stem, changed the Reactor vessel level and the time after shutdown so that this potential problem would not occur. 2) Comment incorporated. <u>NOTE:</u> Question requires a reference.
76+	H	3											X	N	S		
77	F	3											X	M	S		
78	H	3											X	B	S		
79+	H	3											X	N	S		
80	H	3											X	N	E		In the question stem, change the word "should" to "would." <u>RESOLUTION:</u> Comment incorporated.
81	H	3											X	N	E		In the question stem, change the word "should" to "would." <u>RESOLUTION:</u> Comment incorporated.
82	H	3											X	M	S		
83+	F	3											X	N	S		

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws					5. Other		6. B/M/N	7. U/E/S	8. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/units	Backward	Q=K/A	SRO Only				
84	H	2												X	B	S	
85	H	3												X	N	S	
86+	H	3												X	N	S	<u>NOTE</u> : Question requires a reference.
87	F	3					X							X	B	E	1) In the question stem, change the word "should" to "is required to." 2) In the question stem, change "135 gpm" to "140 gpm." <u>RESOLUTION</u> : 1) Comment incorporated. 2) Changed "135 gpm" to "139 gpm." Need to check the division increments on the charging flowmeter. If the divisions are 5 gpm or greater, then change the "139 gpm" back to "135 gpm."
88	H	3												X	B	S	
89+	H	2												X	N	S	
90	H	2												X	B	S	<u>NOTE</u> : Question requires a reference.
91	F	3												X	N	E	In the question stem, change the IRPI position of Rod C-7 from 218 steps to 217 steps (to have it within 24 steps of the Bank Demand position). <u>RESOLUTION</u> : Comment incorporated.
92	F	3												X	N	S	
93+	H	3				X								X	B	E	To make distractor B more plausible, change to go to EOP-2 and then to EOP-1, since this would be the normal transition if one is not kicked of E-2 before the end of the procedure. <u>RESOLUTION</u> : Comment incorporated.
94	F	3												X	B	S	
95	H	2				X								X	B	E	1) Distractor D is not plausible that one would directed to ECA-3.1 from EOP-0. Suggest changing this distractor to: "EOP-0.0, Rediagnosis." 2) Change distractor C from EOP-1.4 to EOP-1.3. <u>RESOLUTION</u> : Comments incorporated.

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6. B/M/N	7. U/E/S	8. Explanation	
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/units	Back-ward	Q=K/A	SRO Only				
96+	F	2											X	X	B	U	<p>1) Question ≠ K/A. The K/A associated with this question is related to K/A 2.1.29 on how to conduct system lineups, instead of the ability to determine the expected plant configuration using design and configuration control documentation.</p> <p>2) In the question stem, change the word "should" to "is required to."</p> <p>RESOLUTION: 1) and 2) Question was replaced. The replaced question is New, Higher cognitive level, and has same K/A. The new question requires a reference. Need to review replaced question once Point Beach approves the question.</p>
97	H	2												X	B	E	<p>Need to clarify the first sentence of question stem, since the question stem states the time when Chemistry has approved the discharge permit, whereas step 2.3 of RAM 3.1 discusses the time when the sample is taken and analyzed.</p> <p>RESOLUTION: Clarified the first sentence of the question stem to state it was the time that Chemistry took the sample.</p>
98	F	3												X	N	S	
99+	H	2												X	B	E	<p>To increase the LOD, change distractor A to have the STA report a RED path exists for CSP-P.1 Response to Pressurized Thermal Shock.</p> <p>RESOLUTION: Comment incorporated.</p>
100	F	3												X	B	E	<p>In the question stem, instead of saying that total AFW flow is 250 gpm, say that have 125 gpm to each SG.</p> <p>RESOLUTION: Comment incorporated.</p>

Operating Test Comments Rev.1

Admin JPMs (ES-301-1):

Comment	Resolution
General Comments:	

System JPMs (ES-301-2):

Comments	Resolution
1) Verify KAs 2) Make sure Safety Function Area agrees with JPM performed. 3) Identify start of alternate path in JPM.	
JPM e. SIM RCP Malfunction (A,D,S): 1) ES-301-2 Outline errata: KA should be "APE 015/017 AK3.03?"	
JPM f. SIM Purge the PRT (L,N,S): 1) JPM Step 4: Explain simulator modeling and valve being used.	

Scenarios:

Comment	Resolution
Overall Comments:	
1. Expedite scenarios by having the operator(s) do their reactivity briefs and reviews / calculations prior to taking the shift. 2. Provide a formalized turnover sheet to crew. 3. Capture/collect data for each scenario (all operator logs / paper, chart data) at end of each scenario. 4. Clarify termination criteria at end of each scenario. 5. Include the applicable Tech Specs in the event summary on cover page (ES-D-1).	Done
SCENARIO 1:	
1) Add evolutions for placing FWRV in service and continuing power ascension to the Initial Conditions for turnover.	1) Comment incorporated.
Event 6 1) Why is this a "C" for both RO and BOP?	1) Credit for BOP removed.

Operating Test Comments Rev.1

Event 7 1) BOP actions?	1) Credit for BOP removed. BOP identifies malfunctions but no actions taken to recover equipment based on procedural flowpath.
SCENARIO 3:	
1) Swap Events 2 and 3 to enhance flow of scenario.	1) Done.

Admin JPMS
 System JPMS
 Scenario 1 Events
 Scenario 2 Events
 Scenario 3 Events
Scenario 4 Events
 TOTAL

X 100 = %



May 21, 2009

10 CFR 55.40
NRC 2009-0060

Mr. Hironori Peterson, Chief
Operator Licensing Branch
U. S. Nuclear Regulatory Commission
Region III
2443 Warrenville Road, Suite 210
Lisle, IL 60532-4352

Point Beach Nuclear Plant, Units 1 and 2
Dockets 50-266 and 50-301
Renewed License Nos. DPR-24 and DPR-27

Initial Operator and Senior Operator License Examinations
Written Examination Results

Operator and Senior Operator License written examinations were conducted at Point Beach Nuclear Plant on May 18, 2009.

Enclosed please find the following examination-related materials:

- One (1) form ES-403-1 Written Examination Grading Quality Checklist
- Student seating chart
- Seven (7) forms ES-401-7 site-specific Reactor Operator (RO) written examination cover sheets
- Seven (7) RO exam original answer forms, clean photocopies and graded photocopies
- Five (5) forms ES-401-8 site-specific Senior Reactor Operator (SRO) written examination cover sheets
- Five (5) SRO exam original answer forms, clean photocopies and graded photocopies
- Candidate questions during written exam administration and proctor responses
- Exam score statistics
- Question analysis for questions with $\leq 50\%$ pass rate
- Question enhancements based on post-exam review with candidates

The Licensee is not submitting significant post-exam comments and agrees with the exam as given. The completed Examination Security Agreement will be sent separately once all personnel have signed-off the post-examination statement.

Very truly yours,

NextEra Energy Point Beach, LLC


James Costedio
Licensing Manager

Enclosures

Point Beach Nuclear Plant
Written Exam Item Review

Evaluation ID#: 2009 Point Beach Initial License Exams (RO and SRO)

Date Evaluation Instrument Administered: 5/18/09 Number of Trainees Evaluated: 7

LIST EACH QUESTION THAT HAS AN AVERAGE SCORE LESS THAN 50%.

Question Number	Average Score (%)	Review Conclusion ¹	Corrective Action
RO 12	43%	Answer verified and distractors confirmed incorrect, question is correct as written. Candidates who answered incorrectly were not familiar enough with the power supply to the rod indication. Material was covered during initial license training.	
RO 16	29%	Answer verified and distractors confirmed incorrect, question is correct as written. Candidates who answered incorrectly were not familiar enough with the entry conditions for the contingency procedure for loss of both trains of sump recirculation. Material was covered during initial license training.	
RO 66	29%	Answer verified and distractors confirmed incorrect, question is correct as written. Candidates who answered incorrectly were not familiar enough with requirements for maintaining licensing proficiency. Material was covered during initial license training.	
SRO 2	40%	Answer verified and distractors confirmed incorrect, question is correct as written. Candidates who answered incorrectly did not identify that high stator temperature needs to be backed up with additional indications prior to tripping RCP. Material was covered during initial license training.	
SRO 19	0%	Answer verified and distractors confirmed incorrect, question is correct as written. Candidates who answered incorrectly were not familiar enough with the LCO allowances for securing RHR flow. Material was covered during initial license training.	

¹Review Conclusions:

- A. Insufficient training for the learning objective tested
- B. Learning objective not adequately covered in the lesson plan
- C. Poorly worded or invalid learning objective
- D. Poorly worded or invalid test item or answer
- E. Incorrect answer in the exam key
- F. More than one correct answer
- G. Question acceptable
- H. Other (state reason in table or on additional sheet)

Review performed by: Sam Au SPH

Date: 5/21/09

Approved by: R. C. Anderson
 Training Supervisor

Date: 5/21/09

Retention: Life of the Plant
 Retain in: Exam Key

Form retained in accordance with record retention schedule identified in NP 1.3.1.

Question enhancements based on post-exam review with candidates

- RO 16 Candidates asked that a trend for RCS pressure be added to the question stem. A trend may aid in determining correct answer.
- RO 57 Candidates asked if distractor 'B' could also be correct depending on the amount of time allowed for plant transient after the failure. 'B' verified to be incorrect regardless of time frame. However, the stem should be enhanced to specify the immediate affect of the failure.
- There was also a question if the rod stop setpoint of 105% would be reached if initial power was 75% as stated in the stem. The question was reviewed with an Instrument and Controls technician who confirmed that the amplifiers associated with NIs could produce an output high enough to trigger the 105% bistable. Because the initial power level is not necessary for the intent of the question and may cause similar confusion if the question is used again, the initial condition in the stem should be changed to 100%.
- SRO 19 Candidate asked if more than one answer might be correct because the LCO statement associated with the question doesn't preclude securing RHR for any reason as long as boron is not diluted. The candidate was correct that the LCO does allow for other circumstance other than the examples given in the basis; however, none of the three distractors listed in the question is an acceptable reason to use the technical specification allowance.
- SRO 25 Candidates asked that the stem be changed from "125 gpm to each S/G" to "125 gpm per S/G" to clarify that that the total amount of flow would be 250 gpm.

Final question enhancements will be evaluated and documented in the exam report process.

Candidate Questions asked during Point Beach Initial License exam on 5/18/09:

RO13: Candidate stated that Inverters will transfer but the buses will not. They stay on the inverter. No question was asked. (Jason Wells 55-33295)

RO25: "Is the AFW flow 125 gpm to 'A' and 125 to 'B' or 125 gpm total?" (Jim Buckley 55-33289)

Proctor answered, "To each steam generator means 125 gpm to 'A' and 125 to 'B' S/G."

RO68: "Am I assuming that none of these automatic actions (referring to actions in choices A-D) occurred?" (Jason Wells 55-33295)

Candidate was directed to answer the question based on the information in the question. Proctor also reminded candidate of the Appendix E (pg 2 section 7) guidance regarding assumptions about conditions not specified in the question.

RO75: "Does 'recovering from a refueling outage' mean fuel is in the core or are we just removing nozzle dams?" (Jeff Baugniet 55-33049)

Clarifying information provided to all candidates that the core reload was completed.

SRO12: "Can assumption be made that automatic actions occurred i.e. w/pressurizer level @ 11% letdown isolated?" (Jamie Weigandt 55-33294)

Candidate was directed to answer the question based on the information in the question. Proctor also reminded candidate of the Appendix E (pg 2 section 7) guidance regarding assumptions about conditions not specified in the question.

REVISED FORMS FOR THE 2009 POINT BEACH NUCLEAR PLANT INITIAL EXAMINATION

Facility: <u>Point Beach</u>		Date of Examination: <u>5/11/2009</u>	
Examination Level: <u>RO</u>		Operating Test Number: <u>2009-01</u>	
Control Room Systems [®] (8 for RO); (7 for SRO-I); (2 or 3 for SRO-U, including 1 ESF)			
System / JPM Title		Type Code*	Safety Function
a. Control Rod Drive System / Perform Rod Exercise Test (2005 ILT Exam) 001.K4.02 (3.8/3.8), 001.A3.05 (3.5/3.5), 001.A4.03 (4.0/3.7)		A, D, P, S	1
b. Engineered Safety Features Actuation System / Respond to loss of sump recirc EPE E11 EA2.2 (3.4/4.2)		A, D, EN, S	2
c. Emergency Core Cooling System / Drain SI Accumulator 006 K2.04 (3.6/3.8)		D, S	3
d. Main Turbine Generator System / Synchronize generator to grid 062 A4.07 (3.1/3.1)		D, L, S	4S
e. Reactor Coolant Pump / RCP Malfunction (2007 ILT Exam) APE 015/017.K3 (3.7/4.0)		A, D, P, S	4P
f. Pressurizer Relief tank / Purge the PRT 007 A1.01 (2.9/3.1)		L, N, S	5
g. A.C. Electrical Distribution System / Secure from Main Transformer Backfeed 062 A4.01 (3.3/3.1)		L, N, S	6
h. Nuclear Instrumentation System / Adjust Nuclear Instruments. (2005 ILT Exam) 015 A1.01 (3.5/3.8)		D, P, S	7
In-Plant Systems [®] (3 for RO); (3 for SRO-I); (3 or 2 for SRO-U)			
i. A. C. Electrical Distribution System / Locally Operate Gas Turbine APE 068 AA1.10 (3.7/3.9)		A, N, E	6
j. Steam Generator System / Locally isolate S/G per EOP 2 APE 040 AA1.03 (4.3/4.3)		D, E, R	4P
k. Auxiliary Feedwater System / Line up alternate bearing cooling 061 K4.13 (2.7/2.9), 2.4.34 (4.2/4.1)		A, D, E	4S
@ All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.			
* Type Codes		Criteria for RO / SRO-I / SRO-U	
(A)lternate path		4-6 / 4-6 / 2-3	
(C)ontrol room		≤ 9 / ≤ 8 / ≤ 4	
(D)irect from bank		≥ 1 / ≥ 1 / ≥ 1	
(E)mergency or abnormal in-plant		- / - / ≥ 1 (control room system)	
(EN)gineered safety feature		≥ 1 / ≥ 1 / ≥ 1	
(L)ow-Power / Shutdown		≥ 2 / ≥ 2 / ≥ 1	
(N)ew or (M)odified from bank including 1(A)		≤ 3 / ≤ 3 / ≤ 2 (randomly selected)	
(P)revious 2 exams		≥ 1 / ≥ 1 / ≥ 1	
(R)CA			
(S)imulator			

Facility: <u>Point Beach</u>		Date of Examination: <u>5/11/2009</u>	
Examination Level: SRO-I		Operating Test Number: <u>2009-01</u>	
Control Room Systems [®] (8 for RO); (7 for SRO-I); (2 or 3 for SRO-U, including 1 ESF)			
System / JPM Title		Type Code*	Safety Function
a. Control Rod Drive System / Perform Rod Exercise Test (2005 ILT Exam) 001.K4.02 (3.8/3.8), 001.A3.05 (3.5/3.5), 001.A4.03 (4.0/3.7)		A, D, P, S	1
b. Engineered Safety Features Actuation System / Respond to loss of sump recirc EPE E11 EA2.2 (3.4/4.2)		A, D, EN, S	2
c. Emergency Core Cooling System / Drain SI Accumulator 006 K2.04 (3.6/3.8)		D, S	3
d. Main Turbine Generator System / Synchronize generator to grid 062 A4.07 (3.1/3.1)		D, L, S	4S
e. Reactor Coolant Pump / RCP Malfunction (2007 ILT Exam) APE 015/017.K3 (3.7/4.0)		A, D, P, S	4P
g. A.C. Electrical Distribution System / Secure from Main Transformer Backfeed 062 A4.01 (3.3/3.1)		L, N, S	6
h. Nuclear Instrumentation System / Adjust Nuclear Instruments. (2005 ILT Exam) 015 A1.01 (3.5/3.8)		D, P, S	7
In-Plant Systems [®] (3 for RO); (3 for SRO-I); (3 or 2 for SRO-U)			
i. A. C. Electrical Distribution System / Locally Operate Gas Turbine APE 068 AA1.10 (3.7/3.9)		A, N, E	6
j. Steam Generator System / Locally isolate S/G per EOP 2 APE 040 AA1.03 (4.3/4.3)		D, E, R	4P
k. Auxiliary Feedwater System / Line up alternate bearing cooling 061 K4.13 (2.7/2.9), 2.4.34 (4.2/4.1)		A, D, E	4S
@ All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.			
* Type Codes		Criteria for RO / SRO-I / SRO-U	
(A)lternate path		4-6 / 4-6 / 2-3	
(C)ontrol room			
(D)irect from bank		≤ 9 / ≤ 8 / ≤ 4	
(E)mergency or abnormal in-plant		≥ 1 / ≥ 1 / ≥ 1	
(EN)gineered safety feature		- / - / ≥1 (control room system)	
(L)ow-Power / Shutdown		≥ 1 / ≥ 1 / ≥ 1	
(N)ew or (M)odified from bank including 1(A)		≥ 2 / ≥ 2 / ≥ 1	
(P)revious 2 exams		≤ 3 / ≤ 3 / ≤ 2 (randomly selected)	
(R)CA		≥ 1 / ≥ 1 / ≥ 1	
(S)imulator			

Facility: <u>Point Beach</u>		Date of Examination: <u>5/11/2009</u>
Examination Level: SRO-U		Operating Test Number: <u>2009-01</u>
Control Room Systems [®] (8 for RO); (7 for SRO-I); (2 or 3 for SRO-U, including 1 ESF)		
System / JPM Title	Type Code*	Safety Function
b. Engineered Safety Features Actuation System / Respond to loss of sump recirc EPE E11 EA2.2 (3.4/4.2)	A, D, EN, S	2
d. Main Turbine Generator System / Synchronize generator to grid 062 A4.07 (3.1/3.1)	D, L, S	4S
h. Nuclear Instrumentation System / Adjust Nuclear Instruments. (2005 ILT Exam) 015 A1.01 (3.5/3.8)	D, P, S	7
In-Plant Systems [®] (3 for RO); (3 for SRO-I); (3 or 2 for SRO-U)		
i. A. C. Electrical Distribution System / Locally Operate Gas Turbine APE 068 AA1.10 (3.7/3.9)	A, N, E	6
j. Steam Generator System / Locally isolate S/G per EOP 2 APE 040 AA1.03 (4.3/4.3)	D, E, R	4P
<p>@ All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.</p>		
* Type Codes	Criteria for RO / SRO-I / SRO-U	
(A)lternate path	4-6 / 4-6 / 2-3	
(C)ontrol room	≤ 9 / ≤ 8 / ≤ 4	
(D)irect from bank	≥ 1 / ≥ 1 / ≥ 1	
(E)mergency or abnormal in-plant	- / - / ≥1 (control room system)	
(EN)gineered safety feature	≥ 1 / ≥ 1 / ≥ 1	
(L)ow-Power / Shutdown	≥ 2 / ≥ 2 / ≥ 1	
(N)ew or (M)odified from bank including 1(A)	≤ 3 / ≤ 3 / ≤ 2 (randomly selected)	
(P)revious 2 exams	≥ 1 / ≥ 1 / ≥ 1	
(R)CA		
(S)imulator		

CREW A

Facility: Point Beach Nuclear Plant		Date of Exam: 5/11/2009		Operating Test No.: 2009-01													
A P P L I C A N T	E V E N T T Y P E	Scenarios												T O T A L	M I N I M U M(*)		
		1			2			3			4						
		C R E W P O S I T I O N			C R E W P O S I T I O N			C R E W P O S I T I O N			C R E W P O S I T I O N						
		S R O	A T C	B O P	S R O	A T C	B O P	S R O	A T C	B O P	S R O	A T C	B O P		R	I	U
RO <input type="checkbox"/>	RX					4								1	1	1	0
SRO-I <input checked="" type="checkbox"/>	NOR	4							1					2	1	1	1
X	I/C	1,2				1,3			3					5	4	4	2
SRO-U <input type="checkbox"/>	MAJ	5				5			4					3	2	2	1
	TS	1,3							2,3					4	0	2	2
RO <input type="checkbox"/>	RX								1					1	1	1	0
SRO-I <input checked="" type="checkbox"/>	NOR		4	4										2	1	1	1
X	I/C		1,3	1,3,6					3					6	4	4	2
SRO-U <input type="checkbox"/>	MAJ		5	5					4					3	2	2	1
	TS			2,3										2	0	2	2
RO <input checked="" type="checkbox"/>	RX		4											1	1	1	0
X	NOR					4			1					2	1	1	1
SRO-I <input type="checkbox"/>	I/C		2,6			6,7			5,6					6	4	4	2
SRO-U <input type="checkbox"/>	MAJ		5			5			4					3	2	2	1
	TS														0	2	2
RO <input type="checkbox"/>	RX														1	1	0
SRO-I <input type="checkbox"/>	NOR														1	1	1
SRO-U <input type="checkbox"/>	I/C														4	4	2
	MAJ														2	2	1
	TS														0	2	2

Instructions:

1. Check the applicant level and enter the operating test number and Form ES-D-1 event numbers for each event type; TS are not applicable for RO applicants. ROs must serve in both the "at-the-controls (ATC)" and "balance-of-plant (BOP)" positions; Instant SROs must serve in both the SRO and the ATC positions, including at least two instrument or component (I/C) malfunctions and one major transient, in the ATC position. If an Instant SRO *additionally* serves in the BOP position, one I/C malfunction can be credited toward the two I/C malfunctions required for the ATC position.
2. Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.5.d) but must be significant per Section C.2.a of Appendix D. (*) Reactivity and normal evolutions may be replaced with additional instrument or component malfunctions on a 1-for-1 basis.
3. Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements specified for the applicant's license level in the right-hand columns.

CREW B

Facility: Point Beach Nuclear Plant			Date of Exam: 5/11/2009			Operating Test No.: 2009-01											
A P P L I C A N T	E V E N T T Y P E	Scenarios												T O T A L	M I N I M U M(*)		
		1			2			3			4						
		CREW POSITION			CREW POSITION			CREW POSITION			CREW POSITION						
		S R O	A T C	B O P	S R O	A T C	B O P	S R O	A T C	B O P	S R O	A T C	B O P		R	I	U
RO <input type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U <input checked="" type="checkbox"/>	RX		4										1	1	1	0	
	NOR				4								1	1	1	1	
	I/C		2,6		1,3,6								5	4	4	2	
	MAJ		5		5								2	2	2	1	
	TS				2,3								2	0	2	2	
RO <input type="checkbox"/> SRO-I <input checked="" type="checkbox"/> SRO-U <input type="checkbox"/>	RX				4								1	1	1	0	
	NOR	4											1	1	1	1	
	I/C	1,2			1,3								4	4	4	2	
	MAJ	5			5								2	2	2	1	
	TS	1,3											2	0	2	2	
RO <input type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U <input type="checkbox"/>	RX													1	1	0	
	NOR													1	1	1	
	I/C													4	4	2	
	MAJ													2	2	1	
	TS													0	2	2	
RO <input type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U <input type="checkbox"/>	RX													1	1	0	
	NOR													1	1	1	
	I/C													4	4	2	
	MAJ													2	2	1	
	TS													0	2	2	

Instructions:

1. Check the applicant level and enter the operating test number and Form ES-D-1 event numbers for each event type; TS are not applicable for RO applicants. ROs must serve in both the "at-the-controls (ATC)" and "balance-of-plant (BOP)" positions; Instant SROs must serve in both the SRO and the ATC positions, including at least two instrument or component (I/C) malfunctions and one major transient, in the ATC position. If an Instant SRO *additionally* serves in the BOP position, one I/C malfunction can be credited toward the two I/C malfunctions required for the ATC position.
2. Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.5.d) but must be significant per Section C.2.a of Appendix D. (*) Reactivity and normal evolutions may be replaced with additional instrument or component malfunctions on a 1-for-1 basis.
3. Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements specified for the applicant's license level in the right-hand columns.

Facility Name: Point Beach Nuclear Plant														Date of Exam: 5/11/2009			
Tier	Group	RO K/A Category Points											SRO-Only Points				
		K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	A 2	G *	Total	
1. Emergency & Abnormal Plant Evolutions	1	3	3	3	N/A			3	3	N/A			3	18	3	3	6
	2	1	2	2	N/A			2	1	N/A			1	9	2	2	4
	Tier Totals	4	5	5	N/A			5	4	N/A			4	27	5	5	10
2. Plant Systems	1	3	3	3	2	2	1	1	2	3	4	4	28	2	3	5	
	2	1	0	1	1	0	1	1	2	1	1	1	10	1	1	3	
	Tier Totals	4	3	4	3	2	2	2	4	4	5	5	38	4	4	8	
3. Generic Knowledge and Abilities Categories					1	2	3	4				10	1	2	3	4	7
					3	2	3	2					2	1	2	2	

Note: 1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two).

2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.

3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems that are not included on the outline should be added. Refer to Section D.1.b of ES-401 for guidance regarding the elimination of inappropriate K/A statements.

4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.

5. Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.

6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.

7.* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. Refer to Section D.1.b of ES-401 for the applicable K/As.

8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IRs) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above; if fuel handling equipment is sampled in other than Category A2 or G* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note #1 does not apply). Use duplicate pages for RO and SRO-only exams.

9. For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43.

ES-401 PWR Examination Outline Form ES-401-2										
Emergency and Abnormal Plant Evolutions - Tier 1/Group 1 (RO)										
Q#	E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
RO1	000007 Reactor Trip - Stabilization - Recovery / 1						04. 02	Knowledge of system set points, interlocks and automatic actions associated with EOP entry conditions.	4.5	1
RO2	000008 Pressurizer Vapor Space Accident / 3	0 1						Thermodynamics and flow characteristics of open or leaking valves	3.2	1
RO3	000009 Small Break LOCA / 3		0 3					S/Gs	3	1
RO4	000011 Large Break LOCA / 3			1 0				PTS limits on RCS pressure and temperature	3.7	1
	000015 RCP Malfunctions / 4									0
	000017 RCP Malfunctions (Loss of RC Flow) / 4									
RO5	000022 Loss of Rx Coolant Makeup / 2				0 4			Speed demand controller and running indicators (positive displacement pump)	3.3	1
RO6	000025 Loss of RHR System / 4					0 3		Increasing reactor building sump level	3.6	1
RO7	000026 Loss of Component Cooling Water / 8						01. 31	Ability to locate control room switches, controls, and indications, and to determine that they correctly reflect the desired plant lineup.	4.6	1
RO8	000027 Pressurizer Pressure Control System Malfunction / 3	0 1						Definition of saturation temperature	3.1	1
RO9	000029 ATWS / 1		0 6					Breakers, relays, and disconnects	2.9	1
RO10	000038 Steam Gen. Tube Rupture / 3			0 8				Criteria for securing RCP	4.1	1
	000040 Steam Line Rupture - Excessive Heat Transfer / 4									0
	WE12 Uncontrolled Depressurization of all Steam Generators / 4									
RO11	000054 (CE/E06) Loss of Main Feedwater / 4				0 3			AFW auxiliaries, including oil cooling water supply	3.5	1
	000055 Station Blackout / 6									0
RO12	000056 Loss of Off-site Power / 6					3 4		Rod bottom lights	4.1	1
RO13	000057 Loss of Vital AC Inst. Bus / 6						02. 42	Ability to recognize system parameters that are entry-level conditions for Technical Specifications	3.9	1
	000058 Loss of DC Power / 6									0
	000062 Loss of Nuclear Svc Water / 4									0
RO14	000065 Loss of Instrument Air / 8			0 8				Actions contained in EOP for loss of instrument air	3.7	1
RO15	WE04 LOCA Outside Containment / 3		0 2					Facility's heat removal systems, including primary coolant, emergency coolant, the decay heat removal systems, and relations between the proper operation of these systems to the operation of the facility	3.8	1
RO16	WE11 Loss of Emergency Coolant Recirc. / 4	0 2						Normal, abnormal and emergency operating procedures associated with Loss of Emergency Coolant Recirculation	3.6	1
RO17	BWE04; WE05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4				0 2			Operating behavior characteristics of the facility	3.7	1
RO18	000077 Generator Voltage and Electric Grid Disturbances / 6					0 4		VARs outside the capability curve	3.6	1
K/A Category Totals:		3	3	3	3	3	3	Group Point Total:		18

ES-401		PWR Examination Outline							Form ES-401-2	
Emergency and Abnormal Plant Evolutions - Tier 1/Group 2 (RO)										
Q#	E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
	000001 Continuous Rod Withdrawal / 1									0
RO19	000003 Dropped Control Rod / 1	22						Calculation of power defect: algebraic sum of moderator temperature and fuel temperature defects	2.5	1
RO24	000005 Inoperable/Stuck Control Rod / 1			02				Rod insertion limits	3.6	1
RO20	000024 Emergency Boration / 1		01					Valves	2.7	1
	000028 Pressurizer Level Malfunction / 2									0
	000032 Loss of Source Range NI / 7									0
RO21	000033 Loss of Intermediate Range NI / 7			01				Termination of startup following loss of intermediate-range instrumentation	3.2	1
	000036 Fuel Handling Accident / 8									0
	000037 Steam Generator Tube Leak / 3									0
	000051 Loss of Condenser Vacuum / 4									0
	000059 Accidental Liquid RadWaste Rel. / 9									0
	000060 Accidental Gaseous Radwaste Rel. / 9									0
RO22	000061 ARM System Alarms / 7				01			Automatic actuation	3.6	1
	000067 Plant Fire On-site / 8									0
RO23	000068 Control Room Evac. / 8		07					ED/G	3.3	1
	000069 Loss of CTMT Integrity / 5									0
	WE14 High Containment Pressure / 5									0
	000074 Inad. Core Cooling / 4									
	WE06 Degraded Core Cooling / 4									
	WE07 Saturated Core Cooling / 4									
RO25	000076 High Reactor Coolant Activity / 9				04			Failed fuel-monitoring equipment	3.2	1
	WE01 Rediagnosis / 3									0
	WE02 SI Termination / 3									0
	WE13 Steam Generator Over-pressure / 4									0
	WE15 Containment Flooding / 5									0
	WE16 High Containment Radiation / 9									0
	WE03 LOCA Cooldown - Depress. / 4									0
RO26	WE09 Natural Circulation Operations / 4					02		Adherence to appropriate procedures and operation within the limitations in the facility's license and amendments	3.4	1
	WE10 Natural Circulation with Steam Void in Vessel with/without RVLIS. / 4									
RO27	WE08 RCS Overcooling - PTS / 4						02. 25	Knowledge of the bases in Technical Specifications for limiting conditions for operations and safety limits.	3.2	1
K/A Category Totals:		1	2	2	2	1	1	Group Point Total:	9	

ES-401		PWR Examination Outline											Form ES-401-2		
Plant Systems - Tier 2/Group 1 (RO)															
Q#	System # / Name	K1	K2	K3	K4	K5	K6	A1	A2	A3	A4	G	K/A Topic(s)	IR	#
RO28	003 Reactor Coolant Pump									01			Seal injection flow	3.3	1
RO29	004 Chemical and Volume Control										04		Calculation of boron concentration changes	3.2	1
RO30	005 Residual Heat Removal											04.18	Knowledge of the specific bases for EOPs.	3.3	1
RO31	006 Emergency Core Cooling		02										Valve operators for accumulators	2.5	1
RO32	007 Pressurizer Relief/Quench Tank	03											RCS	3	1
RO33	008 Component Cooling Water			01									Loads cooled by CCWS	3.4	1
RO34	010 Pressurizer Pressure Control				01								Spray valve warm-up	2.7	1
RO35	012 Reactor Protection					01							DNB	3.3	1
RO36 RO37	013 Engineered Safety Features Actuation						01	06					Sensors and detectors; RWST level	2.7; 3.6	2
RO38 RO39	022 Containment Cooling		01						05				Containment cooling fans; Major leak in CCS	3; 3.1	2
	025 Ice Condenser														0
RO40 RO41	026 Containment Spray								07	01			Loss of containment spray pump suction when in recirculation mode, possibly caused by clogged sump screen, pump inlet high temperature exceeded cavitation, voiding), or sump level below cutoff (interlock) limit; Pump starts and correct MOV positioning	3.6; 4.3	2
RO42 RO43	039 Main and Reheat Steam									02	07		Isolation of the MRSS; Steam dump valves	3.1; 2.8	2
RO44 RO45	059 Main Feedwater										12	04.45	Initiation of automatic feedwater isolation; Ability to prioritize and interpret the significance of each annunciator or alarm	3.4; 4.1	2
RO46 RO47	061 Auxiliary/Emergency Feedwater	04										02.36	RCS; Ability to analyze the effect of maintenance activities, such as degraded power sources, on the status of limiting conditions for operations.	3.9; 3.1	2
RO48 RO49	062 AC Electrical Distribution	02	01										ED/G; Major system loads	4.1; 3.3	2
RO50	063 DC Electrical Distribution			02									Components using DC control power	3.5	1
RO51	064 Emergency Diesel Generator				05								Incomplete-start relay	2.8	1
RO52	073 Process Radiation Monitoring					01							Radiation theory, including sources, types, units, and effects	2.5	1
RO53	076 Service Water										04		Emergency heat loads	3.5	1
RO54	078 Instrument Air											04.11	Knowledge of abnormal condition procedures.	4	1
RO55	103 Containment			01									Loss of containment integrity under shutdown conditions	3.3	1
															0
K/A Category Totals:		3	3	3	2	2	1	1	2	3	4	4	Group Point Total:	28	

ES-401		PWR Examination Outline											Form ES-401-2		
		Plant Systems - Tier 2/Group 2 (RO)													
Q#	System # / Name	K1	K2	K3	K4	K5	K6	A1	A2	A3	A4	G	K/A Topic(s)	IR	#
	001 Control Rod Drive														0
	002 Reactor Coolant														0
RO56	011 Pressurizer Level Control											01, 20	Ability to interpret and execute procedure steps	4.6	1
	014 Rod Position Indication														0
RO57	015 Nuclear Instrumentation	01											RPS	4.1	1
	016 Non-nuclear Instrumentation														0
RO58	017 In-core Temperature Monitor									01			Indications of normal, natural, and interrupted circulation of RCS	3.6	1
	027 Containment Iodine Removal														0
	028 Hydrogen Recombiner and Purge Control														0
	029 Containment Purge														0
	033 Spent Fuel Pool Cooling														0
	034 Fuel Handling Equipment														0
	035 Steam Generator														0
	041 Steam Dump/Turbine Bypass Control														0
RO60	045 Main Turbine Generator				46								Defeat of reactor trip by overspeed trip test lever	2.5	1
RO59	055 Condenser Air Removal			01									Main condenser	2.5	1
RO61	056 Condensate								04				Loss of condensate pumps	2.6	1
RO62	068 Liquid Radwaste							10					Radiation monitors	2.5	1
RO63	071 Waste Gas Disposal							06					Ventilation system	2.5	1
	072 Area Radiation Monitoring														0
RO64	075 Circulating Water								01				Loss of intake structure	3	1
	079 Station Air														0
RO65	086 Fire Protection											02	Fire detection panels	3.5	1
K/A Category Totals:		1	0	1	1	0	1	1	2	1	1	1	Group Point Total:		10

ES-401		PWR Examination Outline							Form ES-401-2	
Emergency and Abnormal Plant Evolutions - Tier 1/Group 1 (SRO)										
Q#	E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
	000007 Reactor Trip - Stabilization - Recovery / 1									0
	000008 Pressurizer Vapor Space Accident / 3									0
	000009 Small Break LOCA / 3									0
SRO1	000011 Large Break LOCA / 3						04. 20	Knowledge of the operational implications of EOP warnings, cautions, and notes.	4.3	1
SRO2	000015 RCP Malfunctions / 4						0 9	When to secure RCPs on high stator temperatures	3.5	1
	000017 RCP Malfunctions (Loss of RC Flow) / 4									
	000022 Loss of Rx Coolant Makeup / 2									0
	000025 Loss of RHR System / 4									0
SRO3	000026 Loss of Component Cooling Water / 8						01. 07	Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation.	4.7	1
	000027 Pressurizer Pressure Control System Malfunction / 3									0
	000029 ATWS / 1									0
	000038 Steam Gen. Tube Rupture / 3									0
SRO5	000040 Steam Line Rupture - Excessive Heat Transfer / 4						04. 50	Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.	4	1
	WE12 Uncontrolled Depressurization of all Steam Generators / 4									
	000054 (CE/E06) Loss of Main Feedwater / 4									0
SRO4	000055 Station Blackout / 6						0 6	Faults and lockouts that must be cleared prior to re-energizing buses	4.1	1
	000056 Loss of Off-site Power / 6									0
SRO6	000057 Loss of Vital AC Inst. Bus / 6						0 5	S/G pressure and level meters	3.8	1
	000058 Loss of DC Power / 6									0
	000062 Loss of Nuclear Svc Water / 4									0
	000065 Loss of Instrument Air / 8									0
	WE04 LOCA Outside Containment / 3									0
	WE11 Loss of Emergency Coolant Recirc. / 4									0
	BW/E04; WE05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4									0
	000077 Generator Voltage and Electric Grid Disturbances / 6									0
K/A Category Totals:		0	0	0	0	3	3	Group Point Total:		6

ES-401		PWR Examination Outline							Form ES-401-2	
Emergency and Abnormal Plant Evolutions - Tier 1/Group 2 (SRO)										
Q#	E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
	000001 Continuous Rod Withdrawal / 1									0
	000003 Dropped Control Rod / 1									0
	000005 Inoperable/Stuck Control Rod / 1									0
	000024 Emergency Boration / 1									0
	000028 Pressurizer Level Malfunction / 2									0
	000032 Loss of Source Range NI / 7									0
	000033 Loss of Intermediate Range NI / 7									0
	000036 Fuel Handling Accident / 8									0
	000037 Steam Generator Tube Leak / 3									0
	000051 Loss of Condenser Vacuum / 4									0
SRO7	000059 Accidental Liquid RadWaste Rel. / 9					05		The occurrence of automatic safety actions as a result of a high PRM system signal	3.9	1
	000060 Accidental Gaseous Radwaste Rel. / 9									0
	000061 ARM System Alarms / 7									0
	000067 Plant Fire On-site / 8									0
	000068 Control Room Evac. / 8									0
	000069 Loss of CTMT Integrity / 5									0
	WE14 High Containment Pressure / 5									0
	000074 Inad. Core Cooling / 4									0
	WE06 Degraded Core Cooling / 4									0
	WE07 Saturated Core Cooling / 4									0
	000076 High Reactor Coolant Activity / 9									0
	WE01 Rediagnosis / 3									0
	WE02 SI Termination / 3									0
	WE13 Steam Generator Over-pressure / 4									0
	WE15 Containment Flooding / 5									0
	WE16 High Containment Radiation / 9									0
SRO8	WE03 LOCA Cooldown - Depress. / 4						02.22	Knowledge of limiting conditions for operations and safety limits.	4.7	1
SRO9	WE09 Natural Circulation Operations / 4					01		Facility conditions and selection of appropriate procedures during abnormal and emergency operations	3.8	1
	WE10 Natural Circulation with Steam Voide in Vessel with/without RVLIS. / 4									1
SRO10	WE08 RCS Overcooling - PTS / 4						04.45	Ability to prioritize and interpret the significance of each annunciator or alarm.	4.3	1
K/A Category Totals:		0	0	0	0	2	2	Group Point Total:		4

ES-401		PWR Examination Outline											Form ES-401-2			
Plant Systems - Tier 2/Group 1 (SRO)																
Q#	System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#	
	003 Reactor Coolant Pump														0	
SRO11	004 Chemical and Volume Control												02. 39	Knowledge of less than or equal to one hour Technical Specification action statements for systems.	4.5	1
	005 Residual Heat Removal														0	
SRO12	006 Emergency Core Cooling								1 2					Conditions requiring actuation of ECCS	4.8	1
	007 Pressurizer Relief/Quench Tank														0	
	008 Component Cooling Water														0	
	010 Pressurizer Pressure Control														0	
	012 Reactor Protection														0	
	013 Engineered Safety Features Actuation														0	
	022 Containment Cooling														0	
	025 Ice Condenser														0	
SRO13	026 Containment Spray												04. 06	Knowledge of EOP mitigation strategies.	4.7	1
	039 Main and Reheat Steam														0	
	059 Main Feedwater														0	
	061 Auxiliary/Emergency Feedwater														0	
	062 AC Electrical Distribution														0	
	063 DC Electrical Distribution														0	
	064 Emergency Diesel Generator														0	
	073 Process Radiation Monitoring														0	
SRO14	076 Service Water								0 1					Loss of SWS	3.7	1
	078 Instrument Air														0	
SRO15	103 Containment												02. 38	Knowledge of conditions and limitations in the facility license.	4.5	1
															0	
K/A Category Totals:		0	0	0	0	0	0	0	0	2	0	0	3	Group Point Total:		5

ES-401		PWR Examination Outline											Form ES-401-2		
		Plant Systems - Tier 2/Group 2 (SRO)													
Q#	System # / Name	K1	K2	K3	K4	K5	K6	A1	A2	A3	A4	G	K/A Topic(s)	IR	#
	001 Control Rod Drive														0
	002 Reactor Coolant														0
	011 Pressurizer Level Control														0
SRO16	014 Rod Position Indication												01. Ability to perform specific system and integrated plant procedures during all modes of plant operation.	4.4	1
	015 Nuclear Instrumentation														0
	016 Non-nuclear Instrumentation														0
	017 In-core Temperature Monitor														0
	027 Containment Iodine Removal														0
	028 Hydrogen Recombiner and Purge Control														0
	029 Containment Purge														0
	033 Spent Fuel Pool Cooling														0
SRO17	034 Fuel Handling Equipment					0	2						Limiting of load	2.6	1
SRO18	035 Steam Generator								0	1			Faulted or ruptured S/Gs	4.6	1
	041 Steam Dump/Turbine Bypass Control														0
	045 Main Turbine Generator														0
	055 Condenser Air Removal														0
	056 Condensate														0
	068 Liquid Radwaste														0
	071 Waste Gas Disposal														0
	072 Area Radiation Monitoring														0
	075 Circulating Water														0
	079 Station Air														0
	086 Fire Protection														0
K/A Category Totals:		0	0	0	0	1	0	0	1	0	0	1	Group Point Total:		3

Facility Name: Point Beach Nuclear Plant Date of Exam: 5/11/2009							
Q#	Category	K/A #	Topic	RO		SRO-Only	
				IR	#	IR	#
RO66	1. Conduct of Operations	2.1. 01	Knowledge of conduct of operations requirements.	3.8	1		
RO67		2.1. 09	Ability to direct personnel activities inside the control room.	2.9	1		
RO68		2.1. 39	Knowledge of conservative decision making practices.	3.6	1		
SRO19		2.1. 40	Knowledge of refueling administrative requirements.			3.9	1
SRO20		2.1. 45	Ability to identify and interpret diverse indications to validate the response of another indicator.			4.3	1
		2.1.					
		Subtotal			3		2
RO69	2. Equipment Control	2.2. 40	Ability to apply Technical Specifications for a system.	3.4	1		
RO70		2.2. 06	Knowledge of the process for making changes to procedures.	3	1		
SRO21		2.2. 15	Ability to determine the expected plant configuration using design and configuration control documentation, such as drawings, line-ups, tag-outs, etc.			4.3	1
		2.2.					
		2.2.					
		2.2.					
		Subtotal			2		1
RO71	3. Radiation Control	2.3. 04	Knowledge of radiation exposure limits under normal or emergency conditions.	3.2	1		
RO72		2.3. 05	Ability to use radiation monitoring systems, such as fixed radiation monitors and alarms, portable survey instruments, personnel monitoring equipment, etc.	2.9	1		
RO73		2.3. 12	Knowledge of radiological safety principles pertaining to licensed operator duties, such as containment entry requirements, fuel handling responsibilities, access to locked high-radiation areas, aligning filters, etc.	3.2	1		
SRO22		2.3. 06	Ability to approve release permits.			3.8	1
SRO23		2.3. 13	Knowledge of radiological safety procedures pertaining to licensed operator duties, such as response to radiation monitor alarms, containment entry requirements, fuel handling responsibilities, access to locked high-radiation areas, aligning filters, etc.			3.8	1
		2.3.					
		Subtotal			3		2
RO74	4. Emergency Procedures / Plan	2.4. 09	Knowledge of low power/shutdown implications in accident (e.g., loss of coolant accident or loss of residual heat removal) mitigation strategies.	3.8	1		
RO75		2.4. 47	Ability to diagnose and recognize trends in an accurate and timely manner utilizing the appropriate control room reference material.	4.2	1		
SRO24		2.4. 05	Knowledge of the organization of the operating procedures network for normal, abnormal, and emergency evolutions.			4.3	1
SRO25		2.4. 21	Knowledge of the parameters and logic used to assess the status of safety functions, such as reactivity control, core cooling and heat removal, reactor coolant system integrity, containment conditions, radioactivity release control, etc.			4.6	1
		2.4.					
		2.4.					
		Subtotal			2		2
Tier 3 Point Total					10		7

Facility: Point Beach Scenario No.: 1 Op-Test No.: 2009-01

Examiners: _____ Operators: _____

Initial Conditions: Unit 1 EOL 75% power following a load reduction for load control. 1P-29 TDAFW OOS 4 hours ago for turbine bearing replacement. 1P-2B Charging Pump is OOS due to repairs on the variable frequency drive system. Fan 1W-45 B Bus Duct Cooling fan is OOS due to motor failure.

Turnover: OP 1C, "Startup to Power Operation Unit 1" is in effect at step 5.133. Pressurizer Backup Heater group B is energized and an additional letdown orifice is in service. Protected equipment includes 1P-2A Charging Pump, 1P-2C Charging Pump, P-38A MDAFW Pump and P-38B MDAFW Pump. TSAC 3.7.5.B to restore 1P-29 to OPERABLE in 72 hours was entered 4 hours ago. The goal of the shift is to maintain current plant conditions.

Event No.	Malf. No.	Event Type*	Event Description
1		C-BOP C-SRO TS-SRO	1P-11A CCW pump trip with failure of 1P-11B auto start
2		I-RO I-SRO	VCT level transmitter 112 fails low
3		C-BOP TS-SRO	Trip of Accident Fan W1B1
4		R-RO N-BOP N-SRO	B SGFP high bearing temp and associated power reduction
5		M-ALL	B S/G feed line rupture inside containment requiring plant trip and subsequent entry into CSP-H.1. (P-38A MDAFW Pump fails to auto start and trips after start. P-38B MDAFW Pump fails to start)
6		C-RO	Rx Trip buttons fail on panel 1C04

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

REVISED

Facility: Point Beach Scenario No.: 3 Op-Test No.: 2009-01

Examiners: _____ Operators: _____

Initial Conditions: Unit 1 MOL 28% power hold for Chemistry and Reactor Engineering testing from a trip 6 days ago. Control Rods are in manual for the start up. 1P-29 TDAFW OOS 4 hours ago for turbine bearing replacement. 1P-2B Charging Pump is OOS due to repairs on the variable frequency drive system. Fan 1W-45 B Bus Duct Cooling fan is OOS due to motor failure.

Turnover: OP 1C, "Startup to Power Operation Unit 1" is in effect at step 5.120. Pressurizer Backup Heater group B is energized and an addition letdown orifice is in service. Protected equipment includes 1P-2A Charging Pump, 1P-2C Charging Pump, P-38A MDAFW Pump and P-38B MDAFW Pump. TSAC 3.7.5.B to restore 1P-29 to OPERABLE in 72 hours was entered 4 hours ago. The 28% power hold is cleared and you are directed to raise power to 47% per step 5.120 at a rate of 15% per hour

Event No.	Malf. No.	Event Type*	Event Description
1		R-RO N-BOP N-SRO	Raise power per OP-1C
2		TS-SRO	G01 EDG alarm, low starting air pressure
3		I-RO TS-SRO I-SRO	PT-431 failing high
4		M-ALL	B SGTR requiring plant trip
5		C-BOP	SW-2907/2908 fail to auto open
6		C-BOP	P32D SW Pump failure to auto start

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor