

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 15, 2009

Mr. Mark A. Schimmel Site Vice President – Acting Prairie Island Nuclear Generating Plant Northern States Power - Minnesota 1717 Wakonade Drive East Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 -

CORRECTION TO TECHNICAL SPECIFICATIONS (TAC NOS. MD9142 AND

MD9143)

Dear Mr. Schimmel:

On July 1, 2009 (Agencywide Documents Access and Management System Accession No. ML091460809), the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 192 to Facility Operating License No. DPR-42 and Amendment No. 181 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2 (PINGP), respectively. The amendments consisted of changes to the facility operating licenses and the Technical Specifications (TSs) in response to your application dated June 26, 2008, as supplemented by letters dated August 4, August 26, and November 14, 2008, and January 30, February 9, February 20, March 12, and May 4 (2 letters) 2009.

The amendments revised the PINGP TSs to allow the use of Westinghouse 422 VANTAGE+ nuclear fuel and made changes to certain references identified in the Design Features section of the TSs.

TS page 2.0-1, issued with the amendments, contained two editorial errors. In TS section 2.1.1.1, the phrase "... for OFA fuel" was inadvertently retained in the issued TS page. The marked-up TS page submitted with the June 26, 2008, application, correctly showed the change to remove these words from the sentence. Also, in TS Section 2.1.1.2 (b), the abbreviation for weight percent (w/o) was erroneously notated in subscript font.

TS page 2.0-1 has been revised to correct the editorial errors that were inadvertently introduced when the NRC processed Amendment Nos. 192 and 181. Please replace the corresponding page from the TS with the corrected page 2.0-1 enclosed with this letter.

This correction does not change any of the conclusions in the safety evaluation associated with these amendments. Please contact me at (301) 415-4037 if you have any questions on this issue.

Sincerely,

Thomas J. Wengert, Senior Project Manager

Plant Licensing Branch III-1

Chamland for

Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosure:

Corrected TS Page 2.0-1

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2.0 SAFETY LIMITS (SLs)

2.1 SLs

2.1.1 Reactor Core SLs

In MODES 1 and 2, the combination of THERMAL POWER, Reactor Coolant System (RCS) highest loop average temperature, and pressurizer pressure shall not exceed the limits specified in the COLR and the following SLs shall not be exceeded:

- 2.1.1.1 The departure from nucleate boiling ratio (DNBR) shall be maintained ≥ 1.17 for WRB-1 DNB correlation.
- 2.1.1.2 The peak fuel centerline temperature shall be maintained as follows:
 - a) <5080 °F, decreasing by 58 °F per 10,000 MWD/MTU burnup, for fuel containing UO2;
 - b) <(5080 °F minus 6.75 °F per w/o Gd₂O₃), decreasing by 58 °F per 10,000 MWD/MTU burnup, for fuel containing gadolinia;

2.1.2 RCS Pressure SL

In MODES 1, 2, 3, 4, and 5, the RCS pressure shall be maintained \leq 2735 psig.

2.2 SL Violations

- 2.2.1 If SL 2.1.1 is violated, restore compliance and be in MODE 3 within 1 hour.
- 2.2.2 If SL 2.1.2 is violated:
 - 2.2.2.1 In MODE 1 or 2, restore compliance and be in MODE 3 within 1 hour.
 - 2.2.2.2 In MODE 3, 4, or 5, restore compliance within 5 minutes.

This correction does not change any of the conclusions in the safety evaluation associated with these amendments. Please contact me at (301) 415-4037 if you have any questions on this issue.

Sincerely,

/RA by M. Chawla for/

Thomas J. Wengert, Senior Project Manager Plant Licensing Branch III-1 **Division of Operating Reactor Licensing** Office of Nuclear Reactor Regulation

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