ArevaEPRDCPEm Resource

From: Pederson Ronda M (AREVA NP INC) [Ronda.Pederson@areva.com]
Sent: Thursday, August 27, 2009 6:41 PM
To: Tesfaye, Getachew
Cc: BENNETT Kathy A (OFR) (AREVA NP INC); SLIVA Dana (AREVA NP INC); DELANO Karen V (AREVA NP INC); BEELMAN Ronald J (AREVA NP INC)
Subject: Response to U.S. EPR Design Certification Application RAI No. 1, FSAR Ch 6, Supplement 7
Attachments: RAI 1 Supplement 7 Response US EPR DC.pdf

Getachew,

AREVA NP, Inc. provided responses to 6 of the 10 questions of RAI No. 1 on May 9, 2008. Supplement 1 response to RAI No. 1 was sent on September 05, 2008 to address Question 06.02.01-10. Supplement 2 response to RAI No. 1 was sent on December 19, 2008 to provide additional information concerning containment foils and dampers. Supplement 3 to RAI No.1, containing AREVA NP, Inc. Containment Technical Report, ANP-10299P, was sent on January 28, 2009, and provided a technically correct and complete response to 1 of the remaining 4 questions. Supplement 4 response to RAI No. 1 was sent on June 10, 2009 to address 1 of the 3 remaining questions. Supplement 5 response to RAI No. 1 was sent on June 22, 2009 to change the response schedule. Supplement 6 response to RAI No. 1 was sent on July 31, 2009 to also change the response schedule. The attached file, "RAI 1 Supplement 7 Response US EPR DC.pdf," provides technically correct and complete responses to 1 of the remaining 2 questions.

The following table indicates the respective pages in the response document, "RAI 1 Supplement 7 Response US EPR DC.pdf," that contain AREVA NP's response to the subject questions.

Question #	Start Page	End Page
RAI 1 — 06.02.01-07d	2	202

The schedule for a technically correct and complete response to the remaining question remains unchanged and is provided below.

Question #	Response Date
RAI 01 — 06.02.01-01	December 2, 2009

Sincerely,

Ronda Pederson
ronda.pederson@areva.com
Licensing Manager, U.S. EPR Design Certification
AREVA NP Inc.
An AREVA and Siemens company
3315 Old Forest Road
Lynchburg, VA 24506-0935
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Cell: 434-841-8788

From: Pederson Ronda M (AREVA NP INC)
Sent: Friday, July 31, 2009 1:56 PM
To: 'Tesfaye, Getachew'
Cc: BENNETT Kathy A (OFR) (AREVA NP INC); DELANO Karen V (AREVA NP INC); BEELMAN Ronald J (AREVA NP INC)
Subject: Response to U.S. EPR Design Certification Application RAI No. 1, FSAR Ch 6, Supplement 6

Getachew,

AREVA NP, Inc. provided responses to 6 of the 10 questions of RAI No. 1 on May 9, 2008. Supplement 1 response to RAI No. 1 was sent on September 05, 2008 to address Question 06.02.01-10. Supplement 2 response to RAI No. 1 was sent on December 19, 2008 to provide additional information concerning containment foils and dampers. Supplement 3 to RAI No.1, provided AREVA NP, Inc.'s Containment Technical Report, ANP-10299P, and a technically correct and complete response to 1 of the remaining questions on January 28, 2009. Supplement 4 response to RAI No. 1 was sent on June 10, 2009 to address 1 of the remaining questions. Supplement 5 response to RAI No. 1 was sent on June 22, 2009 to change the response schedule.

AREVA NP is unable to provide a response to the remaining questions at this time. The U.S. EPR scaling analysis and distortion analysis are in process. Revision 1 of ANP-10299P includes the scaling analysis of the U.S. EPR for the initial (blowdown) pressure peak. The remaining work scope prevents a complete response according to the original RAI schedule.

The schedule for technically correct and complete responses to the remaining questions has been changed and is provided below.

Question #	Response Date
RAI 01 — 06.02.01-01	December 2, 2009
RAI 01 — 06.02.01-07	August 27, 2009

Sincerely,

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From: Pederson Ronda M (AREVA NP INC)
Sent: Monday, June 22, 2009 3:54 PM
To: 'Getachew Tesfaye'
Cc: BEELMAN Ronald J (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC); DELANO Karen V (AREVA NP INC)
Subject: Response to U.S. EPR Design Certification Application RAI No. 1, FSAR Ch 6, Supplement 5

Getachew,

AREVA NP, Inc. provided responses to 6 of the 10 questions of RAI No. 1 on May 9, 2008. Supplement 1 response to RAI No. 1 was sent on September 05, 2008 to address Question 06.02.01-10. Supplement 2 response to RAI No. 1 was sent on December 19, 2008 to provide additional information concerning containment foils and dampers. Supplement 3 to RAI No.1, containing AREVA NP, Inc. Containment

Technical Report, ANP-10299P, was sent on January 28, 2009, and provided a technically correct and complete response to 1 of the remaining questions. Supplement 4 response to RAI No. 1 was sent on June 10, 2009 to address 1 of the remaining questions.

Pursuant to the NRC audit conducted in April, the U.S. EPR GOTHIC multi-node containment model has undergone substantial nodal changes that prevent a response according to the original RAI schedule.The schedule for technically correct and complete responses to the remaining questions has been changed and is provided below.

Question #	Response Date
RAI 01 — 06.02.01-01	July 31, 2009
RAI 01 — 06.02.01-07	August 27, 2009

Sincerely,

Ronda Pederson

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Licensing Manager, U.S. EPR Design Certification
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From: WELLS Russell D (AREVA NP INC)
Sent: Friday, May 22, 2009 1:22 PM
To: 'Getachew Tesfaye'
Cc: Pederson Ronda M (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC); DELANO Karen V (AREVA NP INC)
Subject: Response to U.S. EPR Design Certification Application RAI No. 1, FSAR Ch 6, Supplement 4

Getachew,

AREVA NP, Inc. (AREVA NP) provided responses to 6 of the 10 questions of RAI No. 1 on May 9, 2008. On January 27, 2009, replacement pages 28 and 29 were provided for the original response. Supplement 1 response to RAI No. 1 was sent on September 05, 2008 to address Question 06.02.01-10. Supplement 2 response to RAI No. 1 was sent on December 19, 2008 to provide additional information concerning containment foils and dampers. AREVA NP Technical Report ANP-10299P, "Applicability of AREVA NP Containment Response Evaluation Methodology to the U.S. EPR™ for Large Break LOCA Analysis," submitted via AREVA NP letter NRC:09:006, dated January 28, 2009, provided technically correct and complete responses to 1 of the 4 remaining questions in RAI No. 01.

Attached please find AREVA NP Inc.'s supplemental response to the subject request for additional information (RAI). The attached file, "RAI 1 Supplement 4 Response US EPR DC.pdf" provides technically correct and complete responses to 1 of the 3 remaining questions.

The following table indicates the respective pages in the response document, "RAI 1 Supplement 4 Response US EPR DC.pdf," that contain AREVA NP's response to the subject questions.

Question #	Start Page	End Page

RAI 01 — 06.02.01-01.a	2	4
RAI 01 — 06.02.01-01.c.4	2	4
RAI 01 — 06.02.01-03.b	5	5

The schedule for technically correct and complete responses to the remaining two questions has been changed and is provided below:

Question #	Response Date
RAI 01 — 06.02.01-01	July 31, 2009
RAI 01 — 06.02.01-07	June 23, 2009

Sincerely,

(Russ Wells on behalf of)

Ronda Pederson
ronda.pederson@areva.com
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New Plants Deployment
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From: Pederson Ronda M (AREVA NP INC)
Sent: Wednesday, January 28, 2009 7:17 PM
To: Getachew Tesfaye
Cc: BENNETT Kathy A (OFR) (AREVA NP INC); DELANO Karen V (AREVA NP INC); BEELMAN Ronald J (AREVA NP INC)
Subject: Response to U.S. EPR Design Certification Application RAI No. 1, Supplement 3

Getachew,

AREVA NP, Inc. provided responses to 6 of the 10 questions of RAI No. 1 on May 9, 2008. On January 27, 2009, replacement pages 28 and 29 were provided for the original response. RAI No. 1, Supplement 1 response was sent on September 05, 2008 to address Question 06.02.01-10. RAI No. 1, Supplement 2 response was sent on December 19, 2008 to provide additional information concerning containment foils and dampers.

The proprietary and non-proprietary versions of the Containment Technical Report, ANP-10299, are submitted via AREVA NP, Inc. letter, "Submittal of ANP-10299P, 'Applicability of AREVA NP Containment Response Evaluation Methodology to the U.S. EPR™ for Large Break LOCA Analysis Technical Report'," NRC:09:006, dated January 28, 2009.

The following table indicates the RAI No. 1 questions that are answered in the subject technical report.

Question #
RAI 01 — 06.02.01-01
RAI 01 — 06.02.01-01.b
RAI 01 — 06.02.01-01.c.1
RAI 01 — 06.02.01-01.c.2
RAI 01 — 06.02.01-01.c.3
RAI 01 — 06.02.01-01.c.5

RAI 01 — 06.02.01-01.c.6
RAI 01 — 06.02.01-02
RAI 01 — 06.02.01-03.a

The schedule for technically correct and complete responses to the remaining questions has been revised and is provided below:

Question #	Response Date
RAI 01 — 06.02.01-01.a	May 22, 2009
RAI 01 — 06.02.01-01.c.4	May 22, 2009
RAI 01 — 06.02.01-03.b	May 22, 2009
RAI 01 — 06.02.01-07.d	June 23, 2009

Sincerely,

Ronda Pederson
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From: Pederson Ronda M (AREVA NP INC)
Sent: Tuesday, January 27, 2009 3:54 PM
To: 'Getachew Tesfaye'
Cc: BEELMAN Ronald J (AREVA NP INC); DELANO Karen V (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC)
Subject: Response to U.S. EPR Design Certification Application RAI No. 1, FSAR Ch 6, Revised pages

Getachew,

AREVA NP Inc. (AREVA NP) provided responses to 6 of the 10 questions of RAI No. 1 on May 9, 2008. Attached are AREVA NP's replacement pages (28 and 29) for the Response to U.S. EPR Design Certification Application RAI No. 1.

The attached file, "RAI 1 Response (revised pages 28 and 29) US EPR DC.pdf" provides two corrected pages of Table 6.2.1-07-3, which replace pages 28 and 29 of the 55 page response document. AREVA NP has determined that the information struck out in the attached document is obsolete and the reviewer should refer to the AREVA NP's Response to U.S. EPR Design Certification Application RAI No. 1, Supplement 2 for the relevant information. The submittal of incorrect information has been entered into AREVA NP's corrective action program.

The following table indicates the respective pages in the attached response document, "RAI 1 Response (revised pages 28 and 29) US EPR DC.pdf," that contain AREVA NP's revised pages to the subject question.

Question #	Start Page	End Page
RAI 1 — 6.2.1-07c	1	2

The schedule for technically correct and complete responses to the remaining RAI No. 1 questions is provided below and remains unchanged.

Question #	Response Date
RAI 1—6.2.1-01	January 28, 2009
RAI 1—6.2.1-02	January 28, 2009
RAI 1—6.2.1-03	January 28, 2009
RAI 1—6.2.1-07d	January 28, 2009

Sincerely,

Ronda Pederson
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From: WELLS Russell D (AREVA NP INC)
Sent: Friday, December 19, 2008 5:12 PM
To: 'Getachew Tesfaye'
Cc: 'John Rycyna'; Pederson Ronda M (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC); DELANO Karen V (AREVA NP INC); SLIVA Dana (EXT)
Subject: Response to U.S. EPR Design Certification Application RAI No. 1, FSAR Ch 6, Supplement 2

Getachew,

During the October 31, 2008 meeting between NRC and AREVA NP at NRC Headquarters to discuss U.S. EPR containment analysis, AREVA NP committed to providing additional information concerning containment foils and dampers. AREVA NP's original response, dated May 9, 2008, to RAI-1 Questions 6.2.1-07a and 6.2.1.-07b are hereby superseded by this supplemental response.

Appended to this file are affected pages of the U.S. EPR Final Safety Analysis Report in redline-strikeout format which support the revised response to RAI 1 Questions 6.2.1-07a and 6.2.1-07b.

The following table indicates the respective pages in the attached response document, "RAI 1 Response Supplement 2 US EPR DC.pdf," that contain AREVA NP's revised response to the subject questions.

Question #	Start Page	End Page
RAI 1 — 6.2.1-07	2	4

The schedule for technically correct and complete responses to the remaining RAI No. 1 questions is provided below and remains unchanged.

Question #	Response Date

RAI 1—6.2.1-01	January 28, 2009
RAI 1—6.2.1-02	January 28, 2009
RAI 1—6.2.1-03	January 28, 2009
RAI 1—6.2.1-07c and -07d	January 28, 2009

Sincerely,

(Russ Wells on behalf of)
Ronda Pederson

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From: Pederson Ronda M (AREVA NP INC)
Sent: Friday, September 05, 2008 1:42 PM
To: 'Getachew Tesfaye'
Cc: 'John Rycyna'; WELLS Russell D (AREVA NP INC); DUNCAN Leslie E (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC); DELANO Karen V (AREVA NP INC)
Subject: FW: Response to U.S. EPR Design Certification Application RAI No. 1, Question 06.02.01-10, Supplement 1

Getachew,

Attached is the AREVA NP Inc. supplemental response to the RAI No. 1, Question 06.02.01-10. AREVA NP Inc. provided responses to 6 of the 10 "RAI No. 1" questions on May 9, 2008. The attached file, "RAI 1, Question 06.02.01-10 Supplement 1 Response US EPR DC.pdf" provides a correct Figure 6.2.1-10-3, which replaces the previously submitted Figure 6.2.1-10-3. The submittal of an incorrect figure has been entered into AREVA NP Inc.'s corrective action program.

The following table provides the page in the response document, "RAI 1, Question 06.02.01-10 Supplement 1 Response US EPR DC.pdf" containing the supplemental information.

Question #	Start Page	End Page
RAI 1 — 06.02.01-10	2	2

Sincerely,

Ronda Pederson

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From: Pederson Ronda M (AREVA NP INC)
Sent: Friday, May 09, 2008 2:49 PM
To: 'Getachew Tesfaye'
Cc: 'John Rycyna'; SHAHROKHI F (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC)
Subject: FW: U.S. EPR Design Certification Application RAI No. 1

Getachew,

Attached please find AREVA NP Inc.'s response to the subject request for additional information (RAI). The attached file, "RAI 1 Response-US EPR DC.pdf" provides technically correct and complete responses to 6 of the 10 questions. Since the response file contains security-related sensitive information that should be withheld from public disclosure in accordance with 10 CFR 2.390, a public version is provided with the security-related sensitive information redacted. This email does not contain any security-related information. The unredacted SUNSI version is provided under separate email.

The attached file, "RAI 1 US EPR FSAR Changes.pdf" provides affected pages of the U.S. EPR Final Safety Analysis Report in redline-strikeout format.

The following table provides the page(s) in the response document, "RAI 1 Response-US EPR DC.pdf" containing the response to each question.

Question #	Start Page	End Page
RAI 1—6.2.1-01	2	4
RAI 1—6.2.1-02	5	5
RAI 1—6.2.1-03	6	6
RAI 1—6.2.1-04	7	7
RAI 1—6.2.1-05	8	10
RAI 1—6.2.1-06	11	11
RAI 1—6.2.1-07	12	33
RAI 1—6.2.1-08	34	37
RAI 1—6.2.1-09	38	50
RAI 1—6.2.1-10	51	55

Complete answers are not provided for 4 of the questions. The schedule for technically correct and complete response to for these questions is provided below.

Question #	Response Date
RAI 1—6.2.1-01	January 28, 2009
RAI 1—6.2.1-02	January 28, 2009
RAI 1—6.2.1-03	January 28, 2009
RAI 1—6.2.1-07c and -07d	January 28, 2009

The provided response and schedule meet NRC's stated expectation and support the established review schedule for the U.S. EPR Design Certification application.
Sincerely,

Ronda Pederson

ronda.pederson@areva.com
Licensing Manager, U.S. EPR Design Certification
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From: Getachew Tesfaye [mailto:Getachew.Tesfaye@nrc.gov]
Sent: Wednesday, April 09, 2008 4:16 PM
To: ZZ-DL-A-USEPR-DL
Cc: Pederson Ronda M (AREVA NP INC); Jack Donohue; Walton Jensen; Christopher Jackson; Joseph Colaccino
Subject: U.S. EPR Design Certification Application RAI No. 1

Attached please find the subject requests for additional information (RAIs). This RAI was discussed with your staff on April 2, 2008. The schedule we have established for review of your application assumes technically correct and complete responses within 30 days of receipt of RAIs. For any RAIs that cannot be answered within 30 days, it is expected that a date for receipt of this information will be provided to the staff within the 30 day period so that the staff can assess how this information will impact the published schedule.

Getachew Tesfaye
Office of New Reactors
U.S. Nuclear Regulatory Commission
(301) 415-3361

Hearing Identifier: AREVA_EPR_DC_RAIs
Email Number: 767

Mail Envelope Properties (5CEC4184E98FFE49A383961FAD402D31012E9B5B)

Subject: Response to U.S. EPR Design Certification Application RAI No. 1, FSAR Ch 6, Supplement 7
Sent Date: 8/27/2009 6:41:18 PM
Received Date: 8/27/2009 6:41:38 PM
From: Pederson Ronda M (AREVA NP INC)

Created By: Ronda.Pederson@areva.com

Recipients:
"BENNETT Kathy A (OFR) (AREVA NP INC)" <Kathy.Bennett@areva.com>
Tracking Status: None
"SLIVA Dana (AREVA NP INC)" <Dana.Sliva@areva.com>
Tracking Status: None
"DELANO Karen V (AREVA NP INC)" <Karen.Delano@areva.com>
Tracking Status: None
"BEELMAN Ronald J (AREVA NP INC)" <Ronald.Beelman@areva.com>
Tracking Status: None
"Tesfaye, Getachew" <Getachew.Tesfaye@nrc.gov>
Tracking Status: None

Post Office: AUSLYNCMX02.adom.ad.corp

Files	**Size**	**Date & Time**
MESSAGE	18967	8/27/2009 6:41:38 PM
RAI 1 Supplement 7 Response US EPR DC.pdf	3764798	

Options
Priority: Standard
Return Notification: No
Reply Requested: No
Sensitivity: Normal
Expiration Date:
Recipients Received:

Response to

Request for Additional Information No. 1, Supplement 7

4/9/2008

U. S. EPR Standard Design Certification

AREVA NP Inc.

Docket No. 52-020

SRP Section: 6.2.1 – Containment Functional Design

Application Section: 6.2

SPCV Branch

Question 6.2.1-07:

(FSAR Section 6.2.1) In order to facilitate the review, the NRC staff needs certain design information as soon as possible. These include: (a) The design of rupture foils and convection foils; (b) details on the modeling of the containment in the multi-node GOTHIC calculations; and (c) detailed results for one of the multi-node GOTHIC calculations. (See detail below.)

a. Foils are installed in a steel framework. Does the framework separate the foil into many foils each of which has to rupture individually? What is the size of the individual foils? What is the total surface area of the foils? What is the available flow area once the foils rupture? Justify this flow area. What materials are the foils made of? What is the thickness of the foils? What is the weight of the foils per square foot? The above questions apply to both rupture foils and convection foils. In addition, how many fusible links are on the frame of a convection foil? Where are the links located?
b. Please provide a simplified sketch of the containment. Show internal walls, major components (steam generators, tanks, and so on) and the location and size of all mixing dampers, rupture foils and convection foils.
c. Provide the noding diagram that was used in the multi-node GOTHIC calculations (pages 47-50 of the U.S.-EPR Design Certification Acceptance Review presentation by AREVA of January 29, 2008). Provide the input data used in these calculations: volumes, elevations, cross sections, flow path dimensions, heat transfer surfaces. What was the break location and break size selected for the above referenced multi-node GOTHIC calculations?
d. Provide for one of the multi-node calculations (LB LOCA cold leg break if available) sufficient details of the results to permit visualization of flow patterns in the containment as well as heat transfer to the various heat sinks. Results should be given as a function of time for the duration of the accident. Please include flow in each flow path; content, temperature and pressure of each node; surface temperatures of significant heat sinks and heat transfer to each significant heat sink.

Response to Question 6.2.1-07:

Response to Question 6.2.1-07d:

The following plots are from a cold leg pump suction analysis that allowed the non-safety grade doors to open if the doors experienced sufficient pressure differential. This case shows the magnitude of the flow through the various doors in the containment and the impact of the doors on the pressure response. Figures 6.2.1-07d-1 through 6.2.1-07d-41 show the development of the circulation profiles in the containment. The magnitude of flow through the doors can be seen in Figures 6.2.1-07d-19 through 6.2.1-07d-23 as the doors near the break burst open; in this case the break is in Loop 3 (Node 10 on Figure 9-7 of ANP-12099P). The blowdown phase dominates the response as the later portions of the event evolve to a steady-state condition.

Figures 6.2.1-07d-42 through 6.2.1-07d-53 show the temperature response in various lumped parameter cells in the model, while Figures 6.2.1-07d-54 through 6.2.1-07d-63 provide the temperature distribution in the subdivided dome at various elevations.

Figures 6.2.1-07d-64 through 6.2.1-07d-75 provide the containment pressure response in both the equipment area and the accessible space. Figures 6.2.1-07d-76 through 6.2.1-07d-87 present the liquid volume fractions in the lumped parameter nodes. These figures show that the

in-containment refueling water storage tank (IRWST) level remains relatively constant during the event and locations where liquid may be held during the event.

Figures 6.2.1-07d-88 through 6.2.1-07d-123 provide the air and steam volume fractions. In the legend for each plot “1Rx” refers to the air volume fraction in node x and “SRx” refers to the steam volume fraction in node x. The plots show the concentration of steam decreasing following hot leg injection as the heat conductors in each node continue to condense the steam.

Figures 6.2.1-07d-124 through 6.2.1-07d-161 provide heat conductor surface temperatures and Figures 6.2.1-07d-162 through 6.2.1-07d-199 provide the heat conductor heat rates for a variety of conductors located throughout the containment.

FSAR Impact:

The U.S. EPR FSAR will not be changed as a result of this question.

Containment Internal Flow Rates

Communication between Equipment Area and Accessible Area

Figure 6.2.1-07d-1—Vapor Flow from the Equipment Area to the Containment Dome via the Rupture and Convection Foils

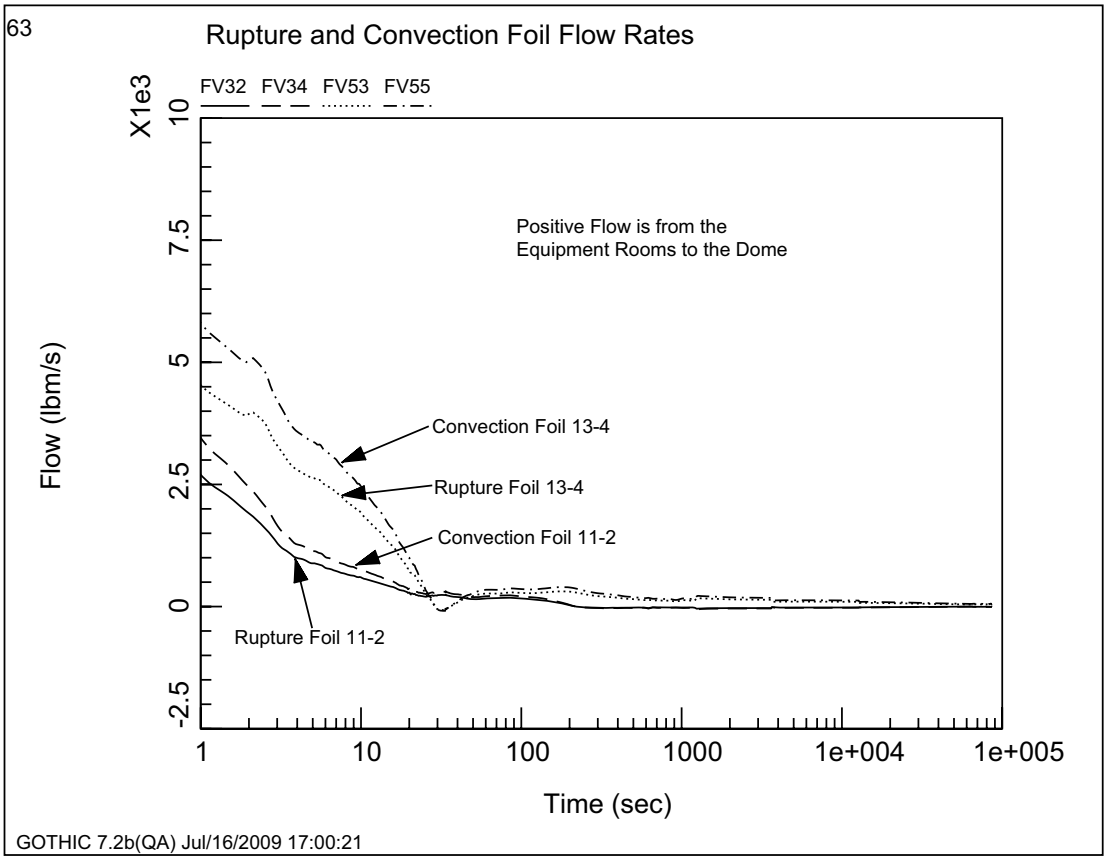
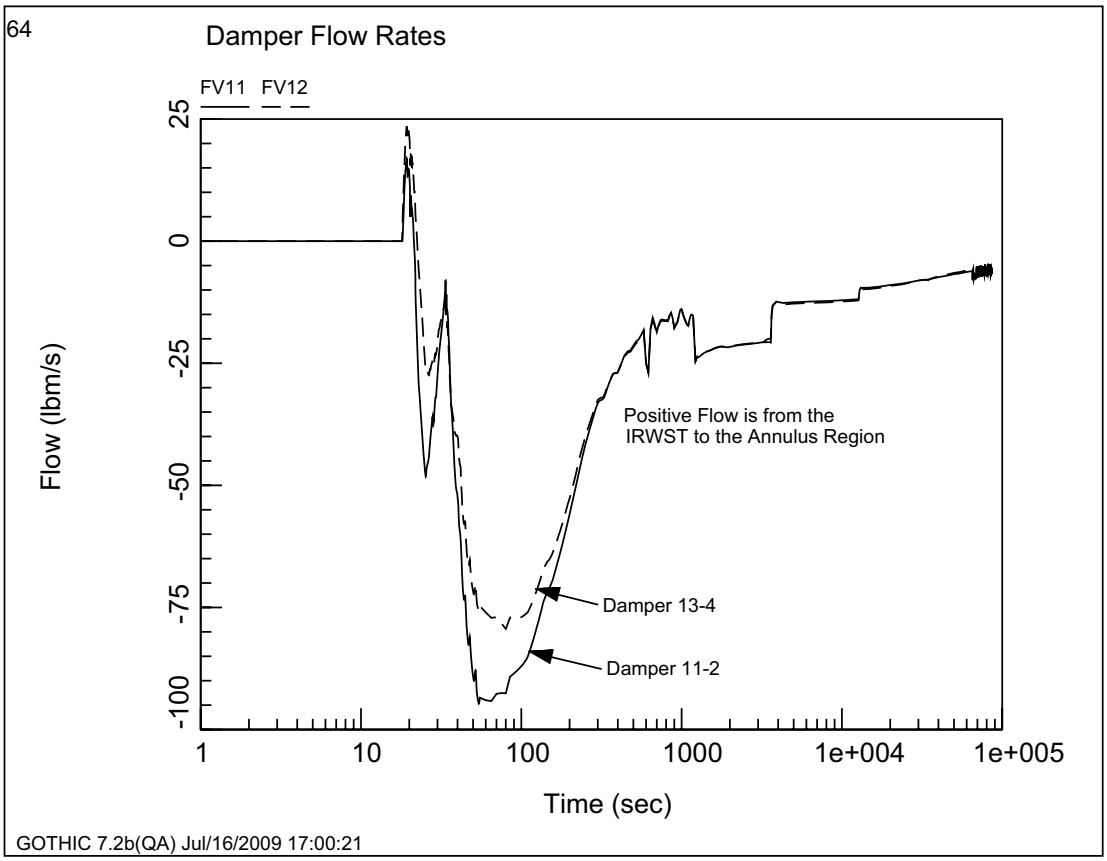


Figure 6.2.1-07d-2—Vapor Flow from the Annulus Region to the Equipment Area via the Dampers



Flow Rates Internal to the Equipment Area

Figure 6.2.1-07d-3—Flow into the Spreading Room via Drain Paths

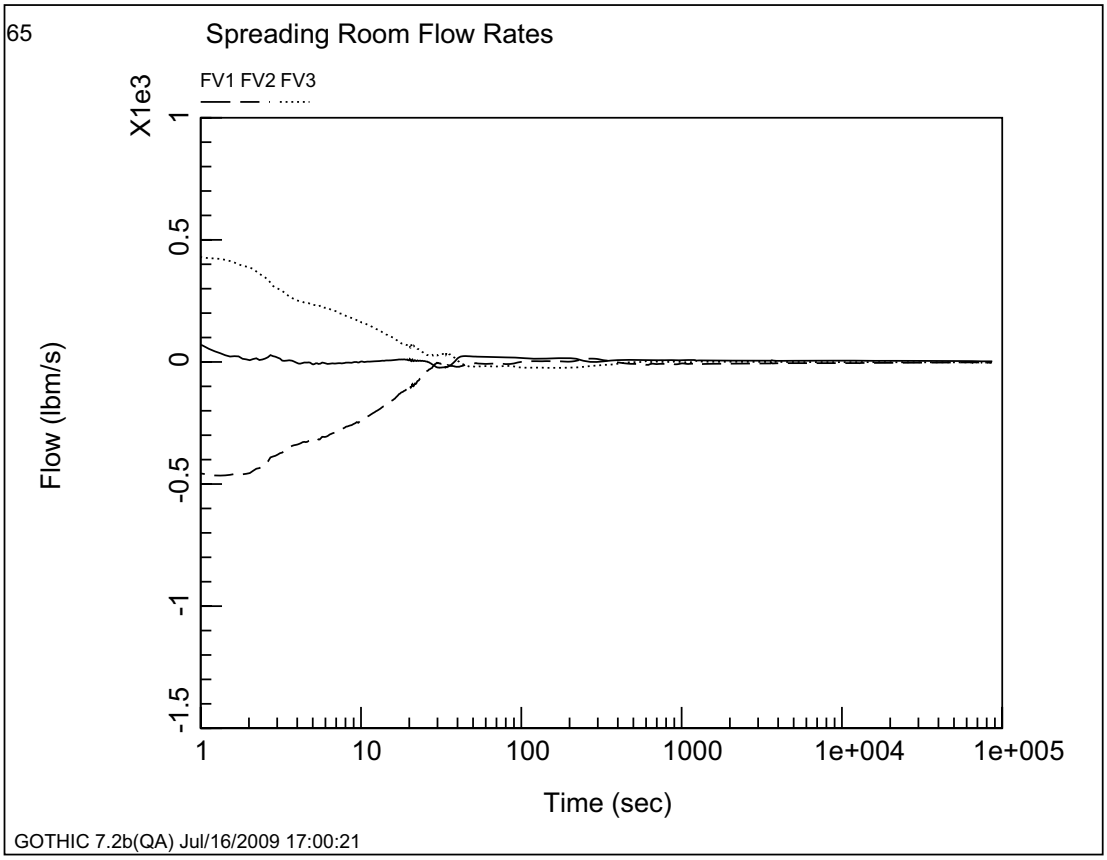


Figure 6.2.1-07d-4—Vapor Flow into the IRWST via Drain Paths (4, 5, 6, 7, 8)

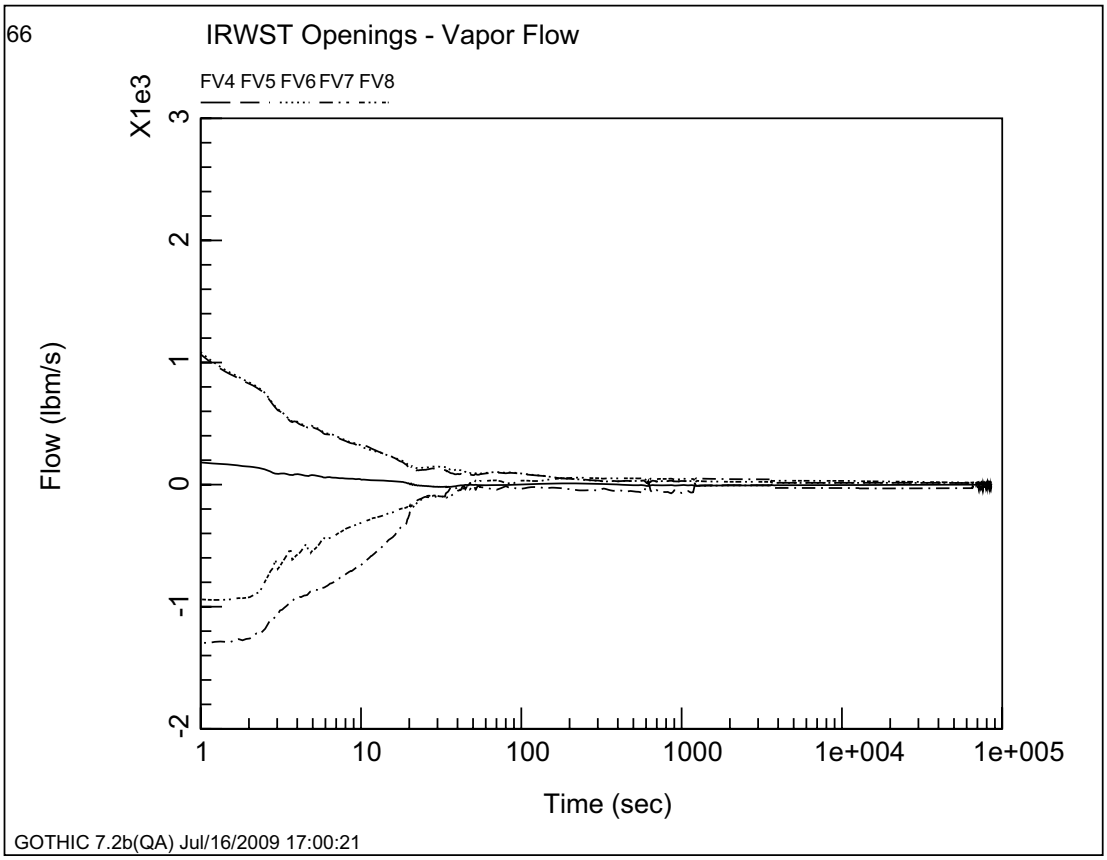


Figure 6.2.1-07d-5—Vapor Flow into the IRWST via Drain Paths (9, 10)

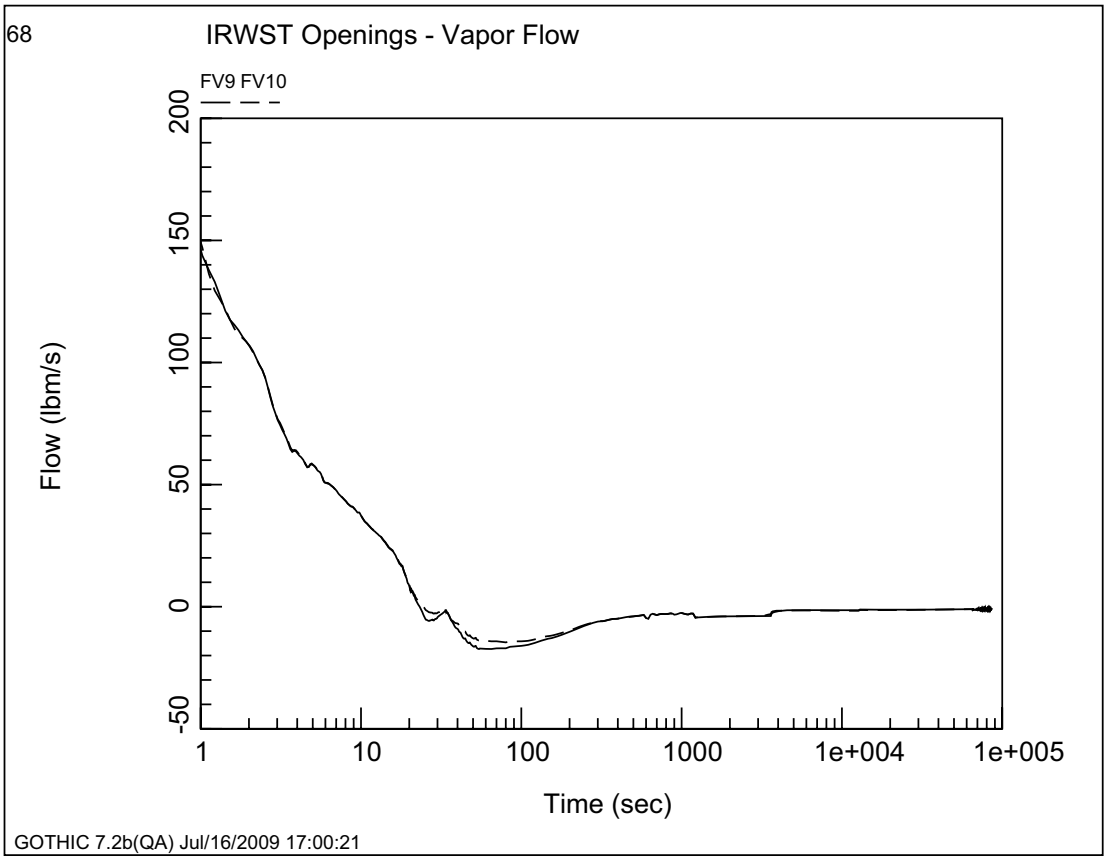


Figure 6.2.1-07d-6—Water Flow into the IRWST via Drain Paths

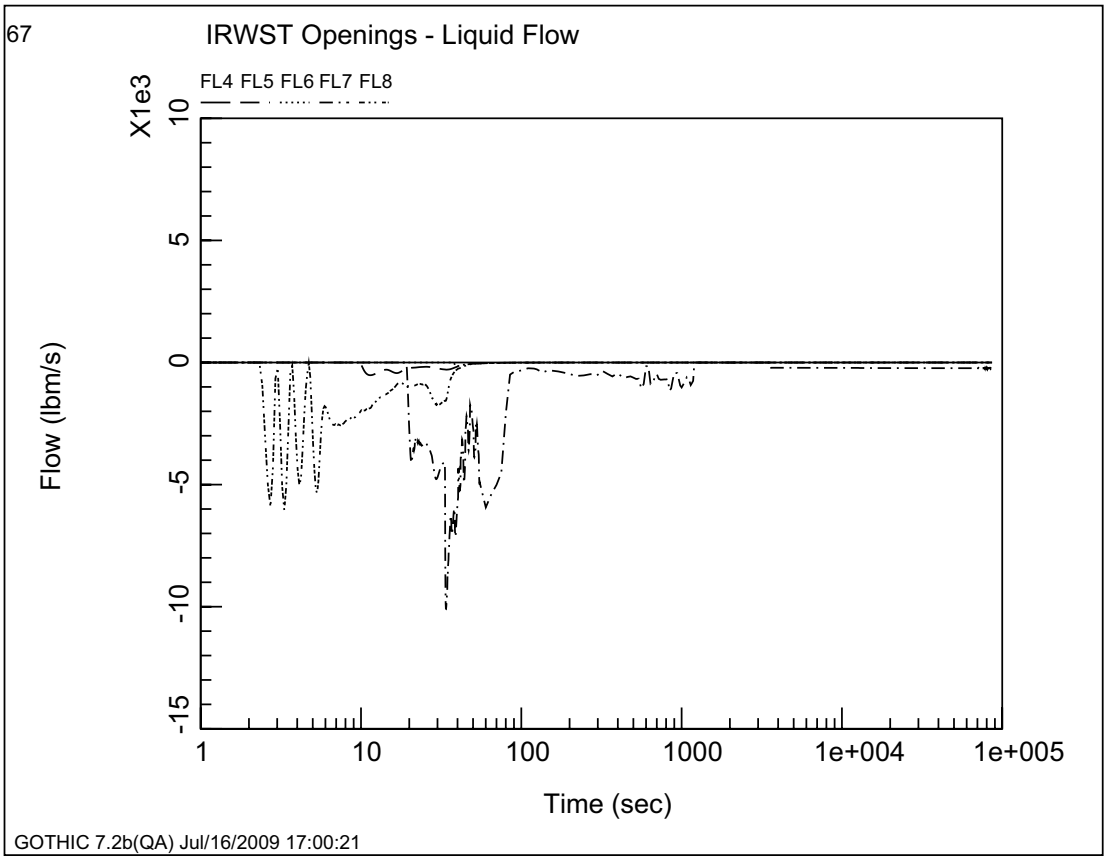


Figure 6.2.1-07d-7—Water Flow into the IRWST via Drain Paths

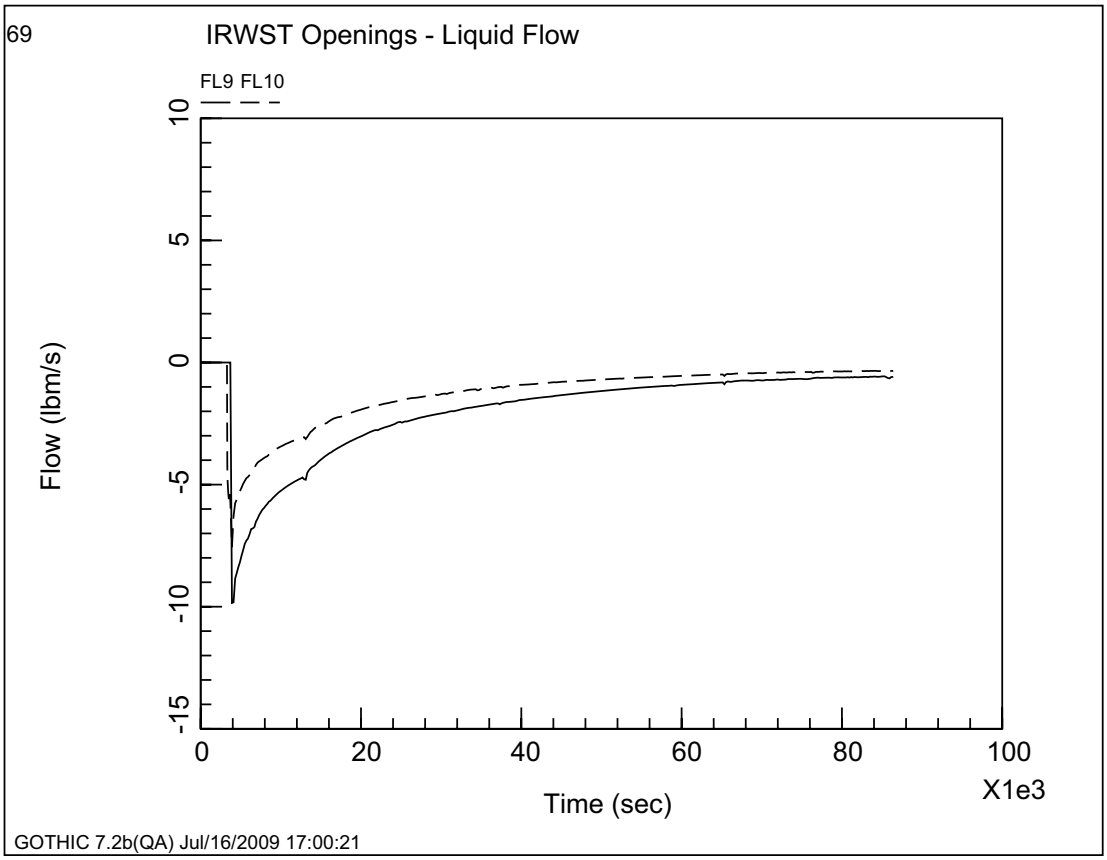


Figure 6.2.1-07d-8—Flow from the Loop 1 Lower Equipment Room to Adjacent Rooms

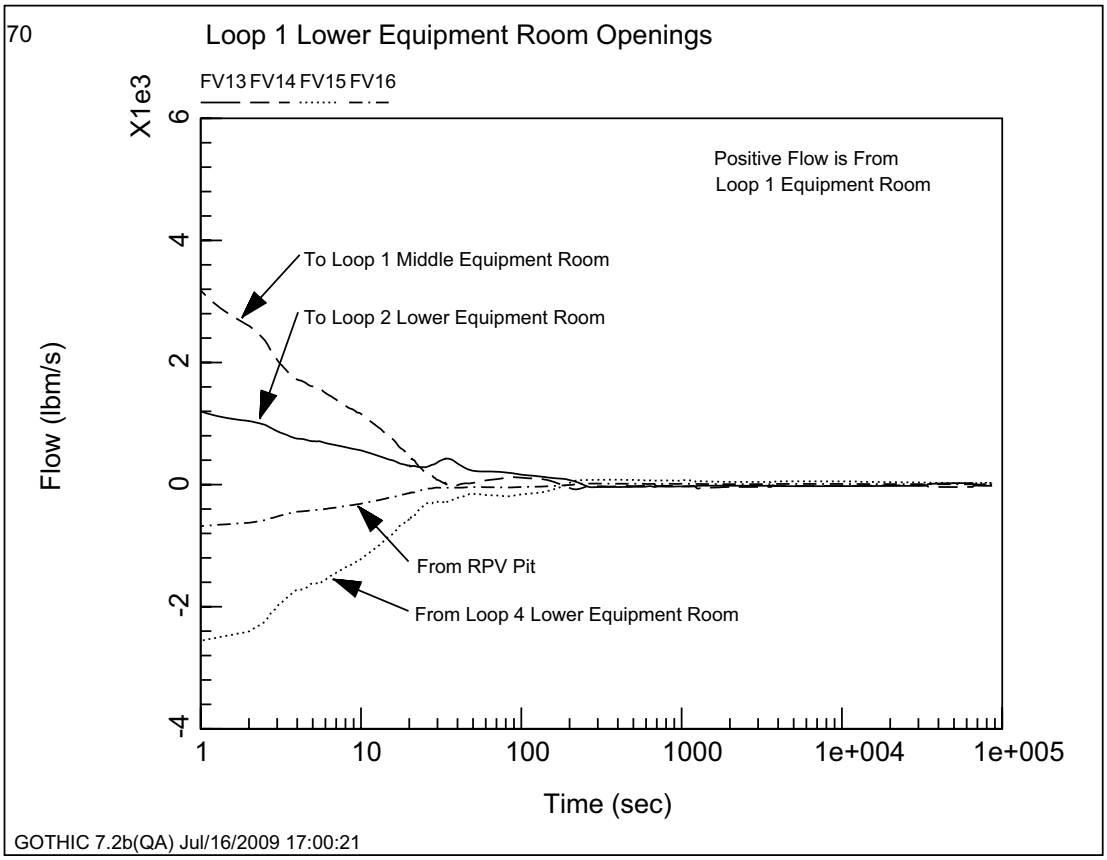


Figure 6.2.1-07d-9—Flow from the Loop 1 Lower Equipment Room to Adjacent Rooms

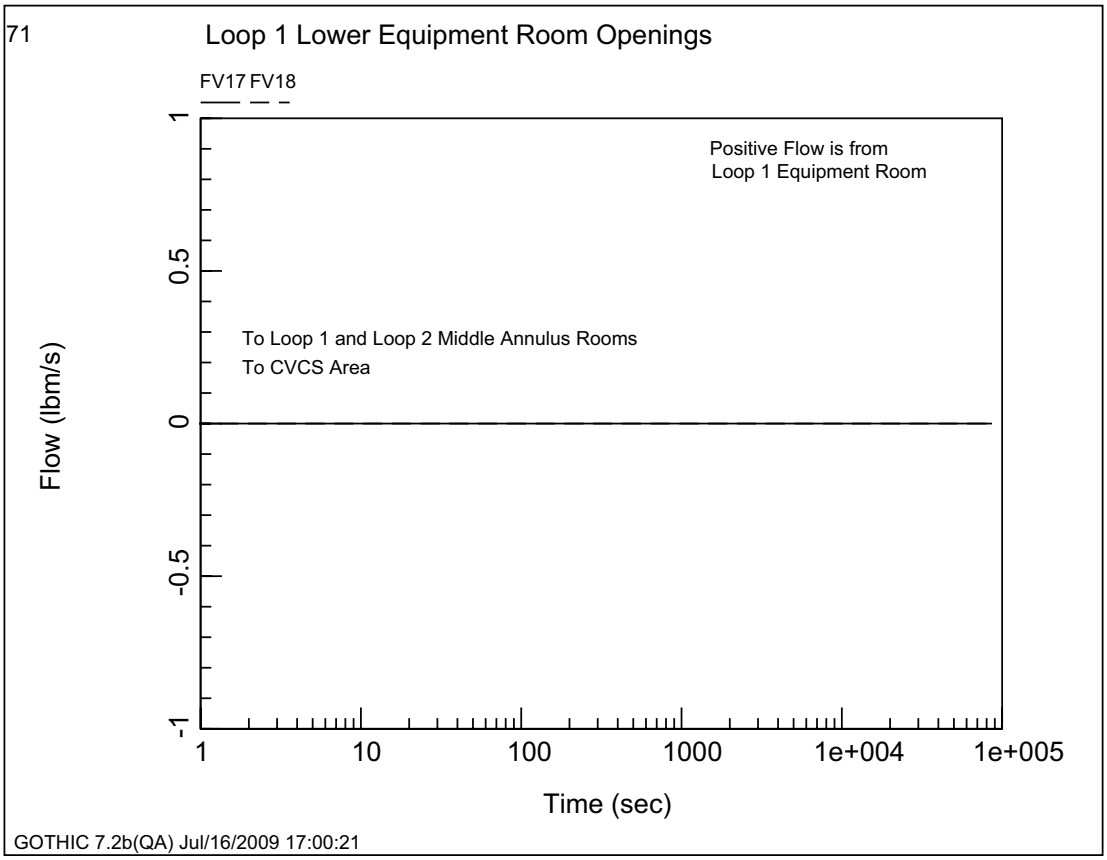


Figure 6.2.1-07d-10—Flow from the Loop 2 Lower Equipment Room to Adjacent Rooms

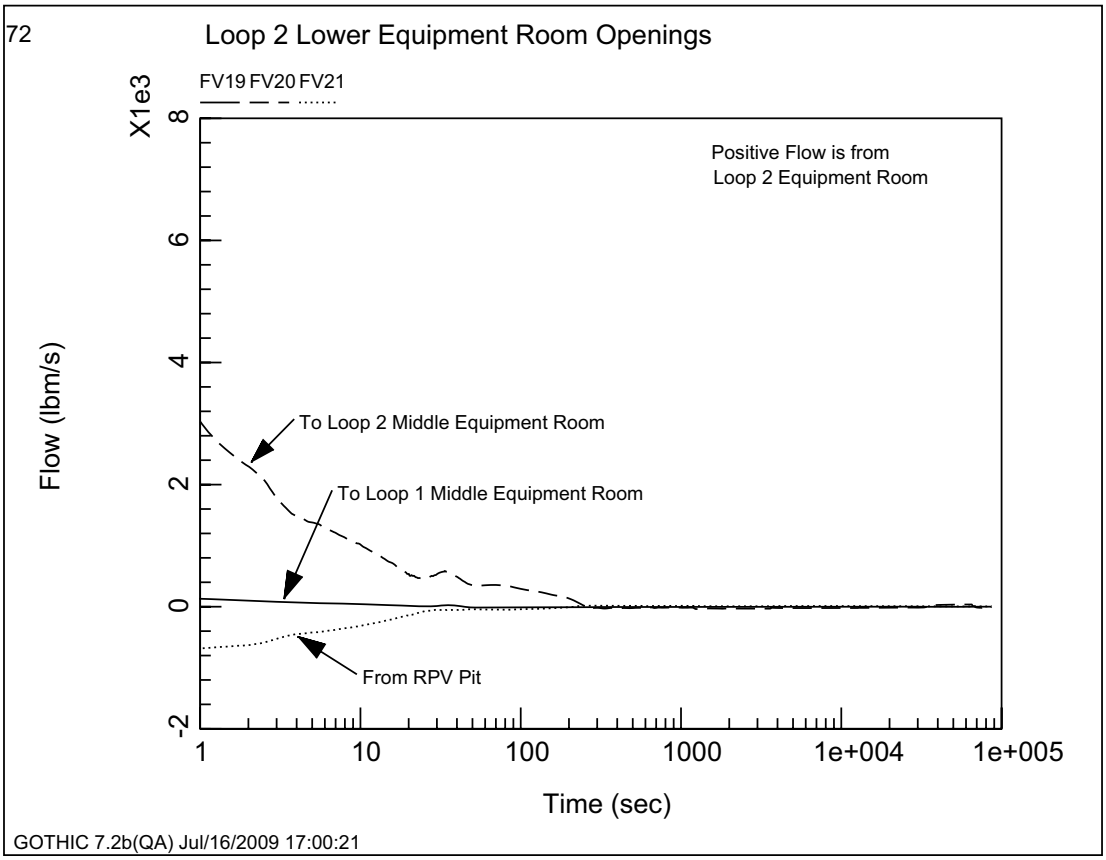


Figure 6.2.1-07d-11—Flow from the Loop 2 Lower Equipment Room to Adjacent Rooms

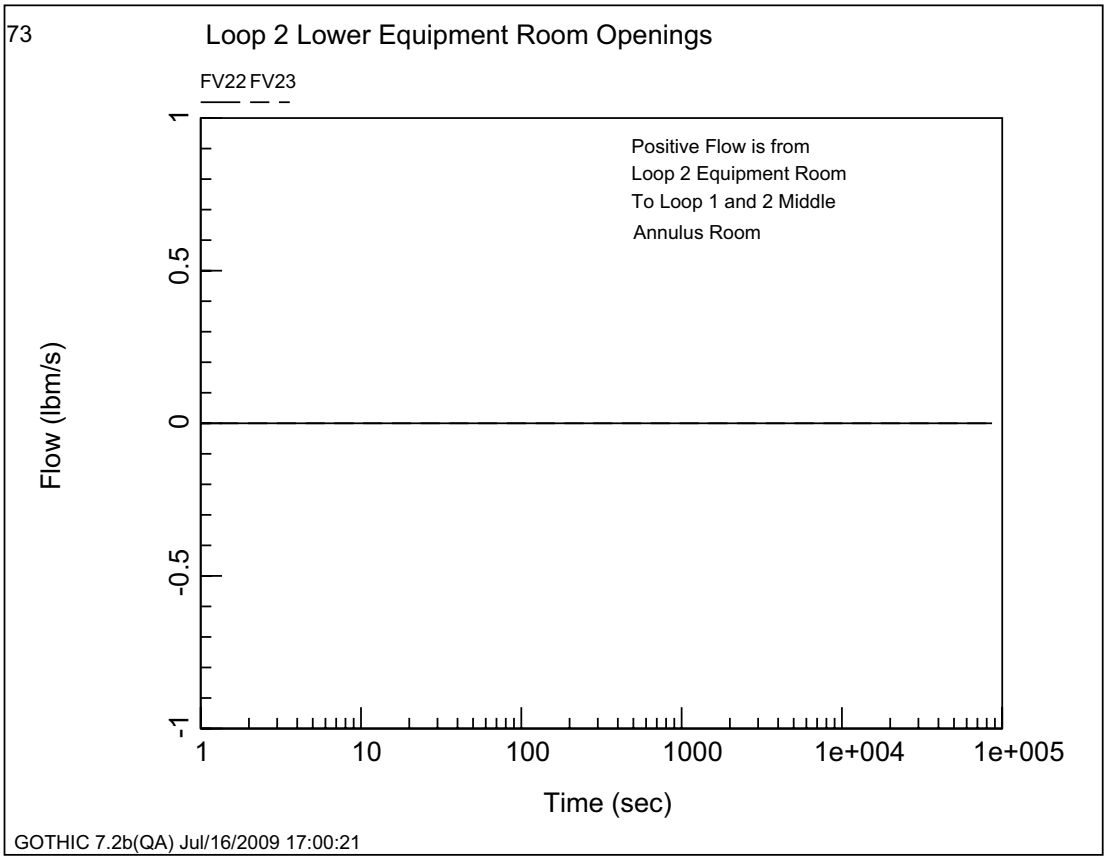


Figure 6.2.1-07d-12—Flow from the Loop 1 Middle Equipment Room to Adjacent Rooms

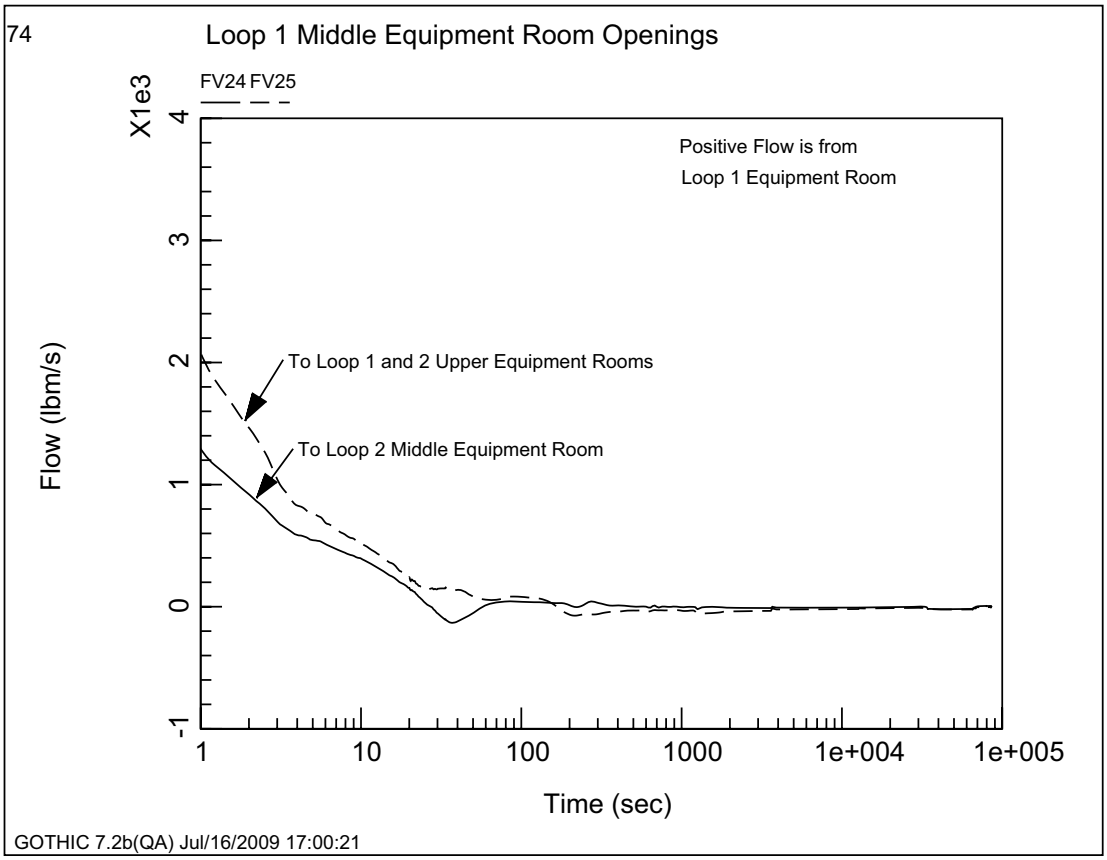


Figure 13—Flow from the Loop 1 Middle Equipment Room to Adjacent Rooms

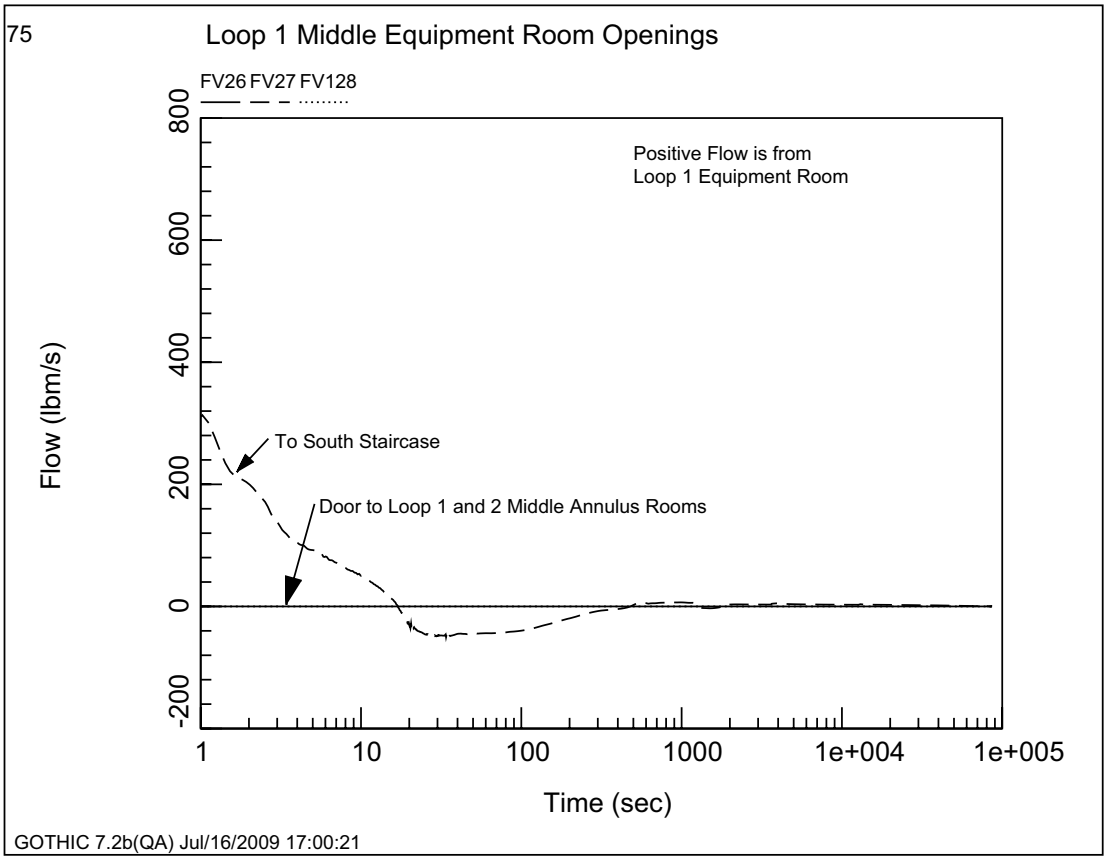


Figure 6.2.1-07d-14—Flow from the Loop 2 Middle Equipment Room to Adjacent Rooms

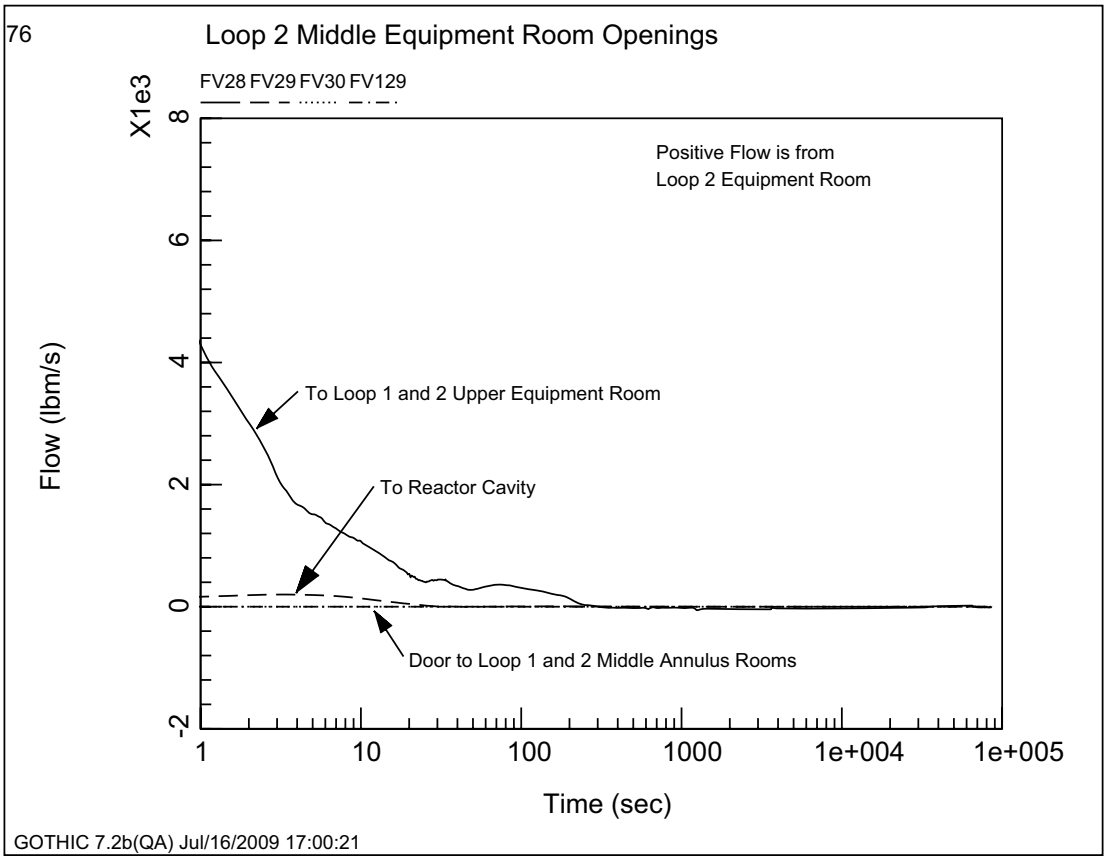


Figure 6.2.1-07d-15—Flow from the Loop 1 and 2 Upper Equipment Room to Adjacent Rooms

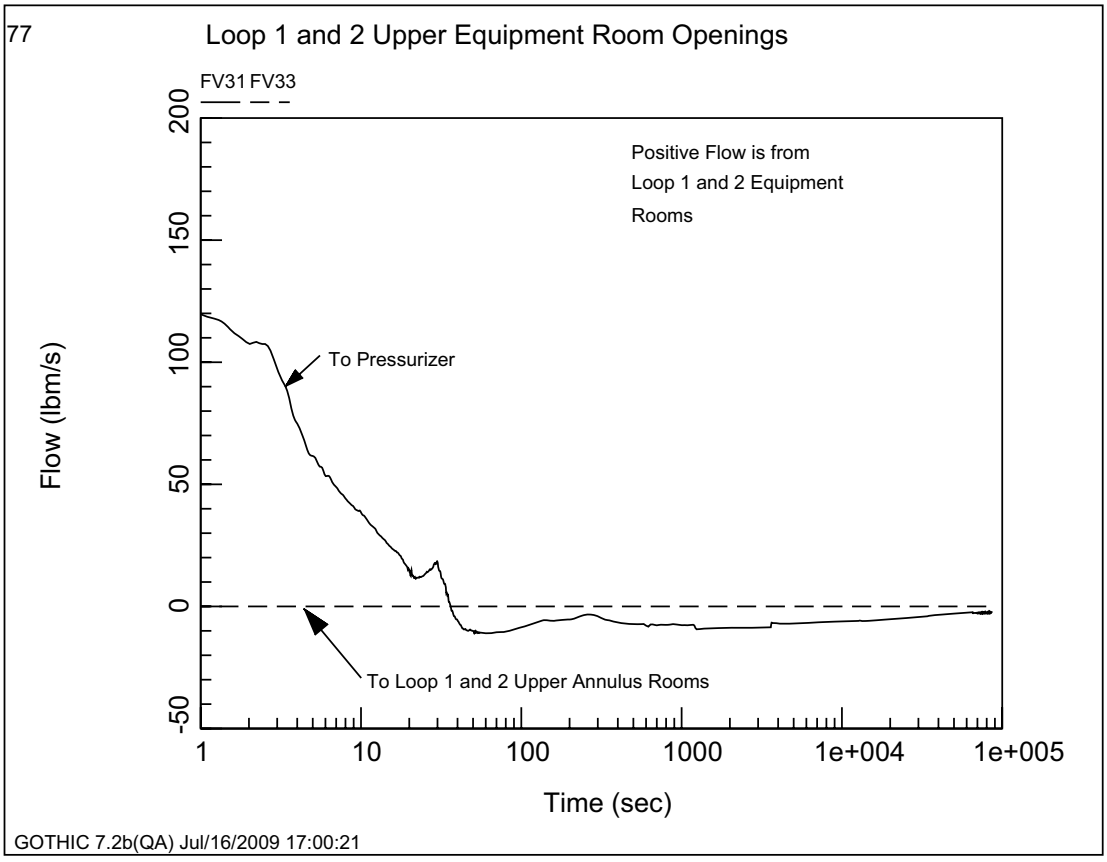


Figure 6.2.1-07d-16—Flow from the Reactor Pressure Vessel Pit to Adjacent Rooms

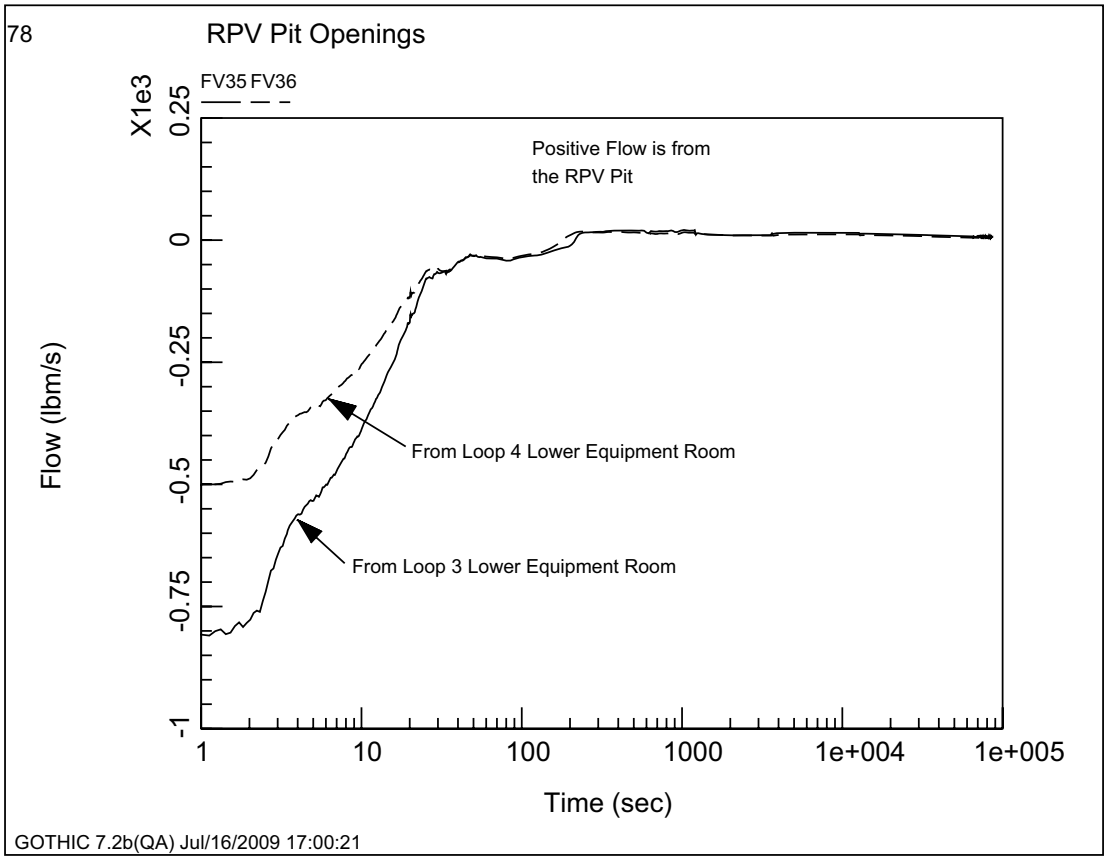


Figure 6.2.1-07d-17—Flow from the Reactor Cavity to Adjacent Rooms

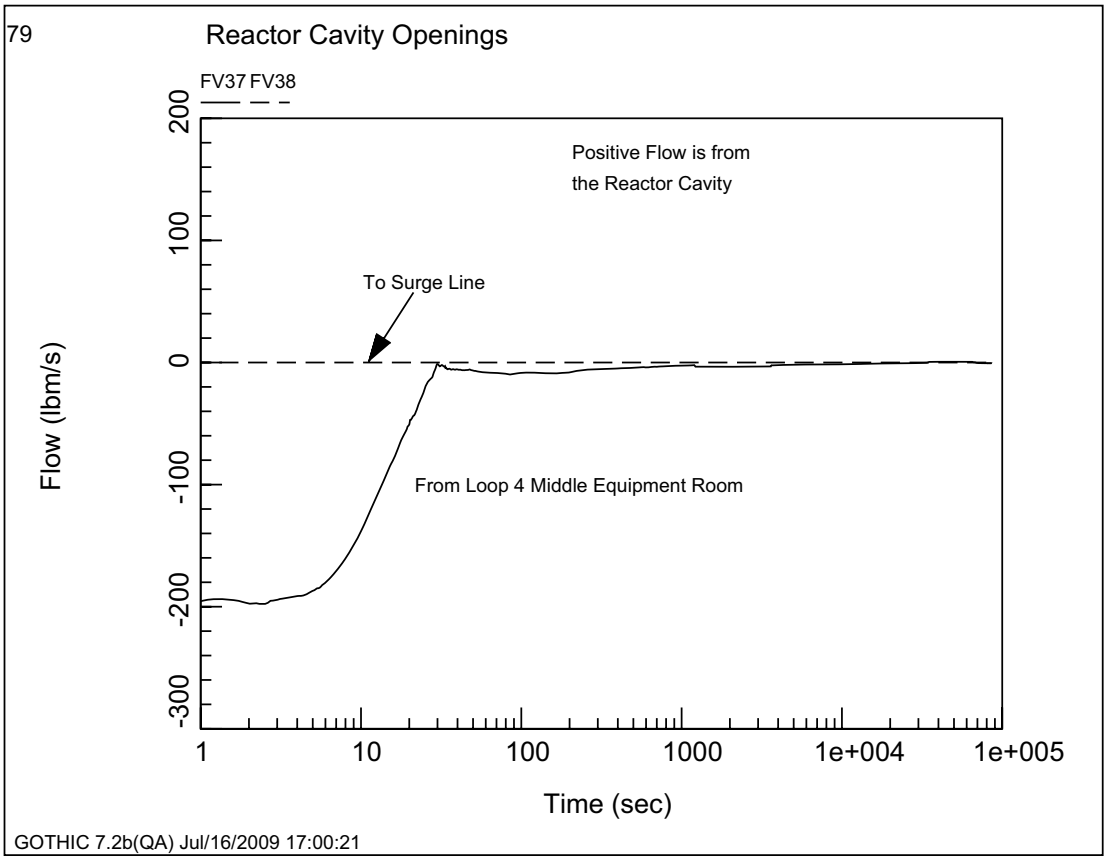


Figure 6.2.1-07d-18—Flow from the Loop 3 Lower Equipment Room to Adjacent Rooms

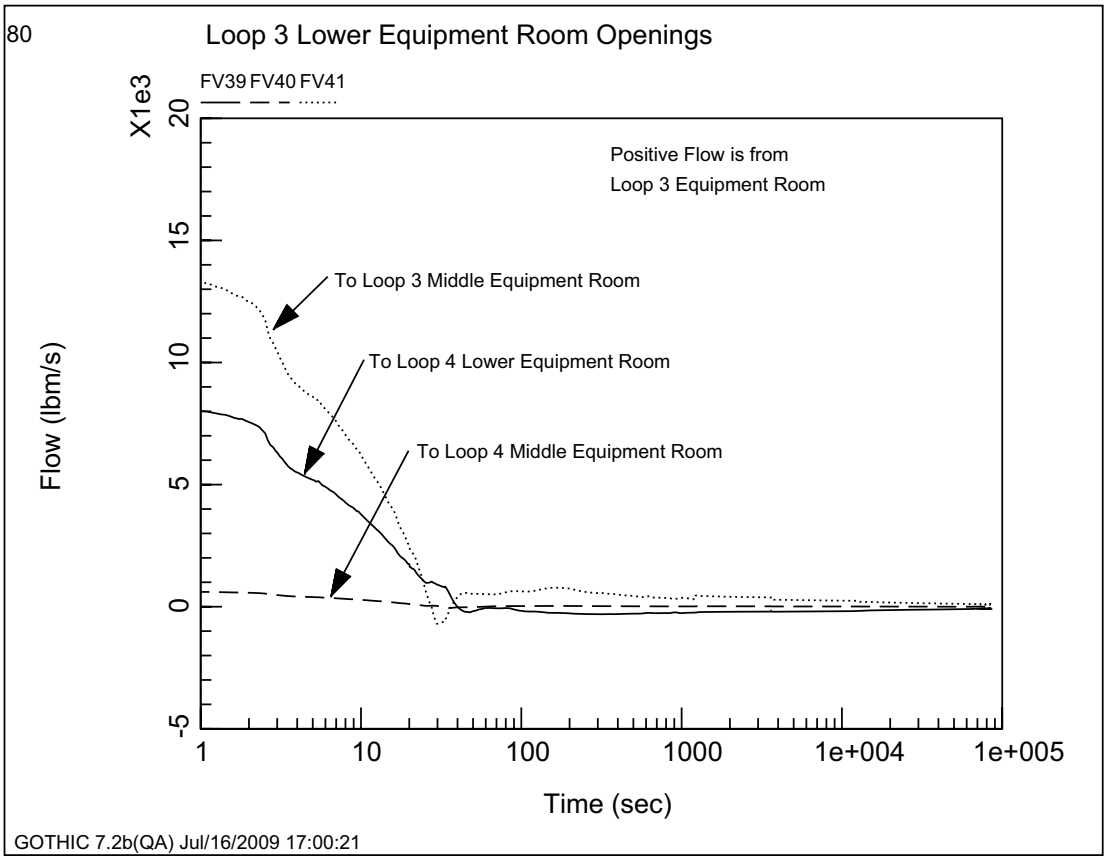


Figure 6.2.1-07d-19—Flow from the Loop 3 Lower Equipment Room to Adjacent Rooms

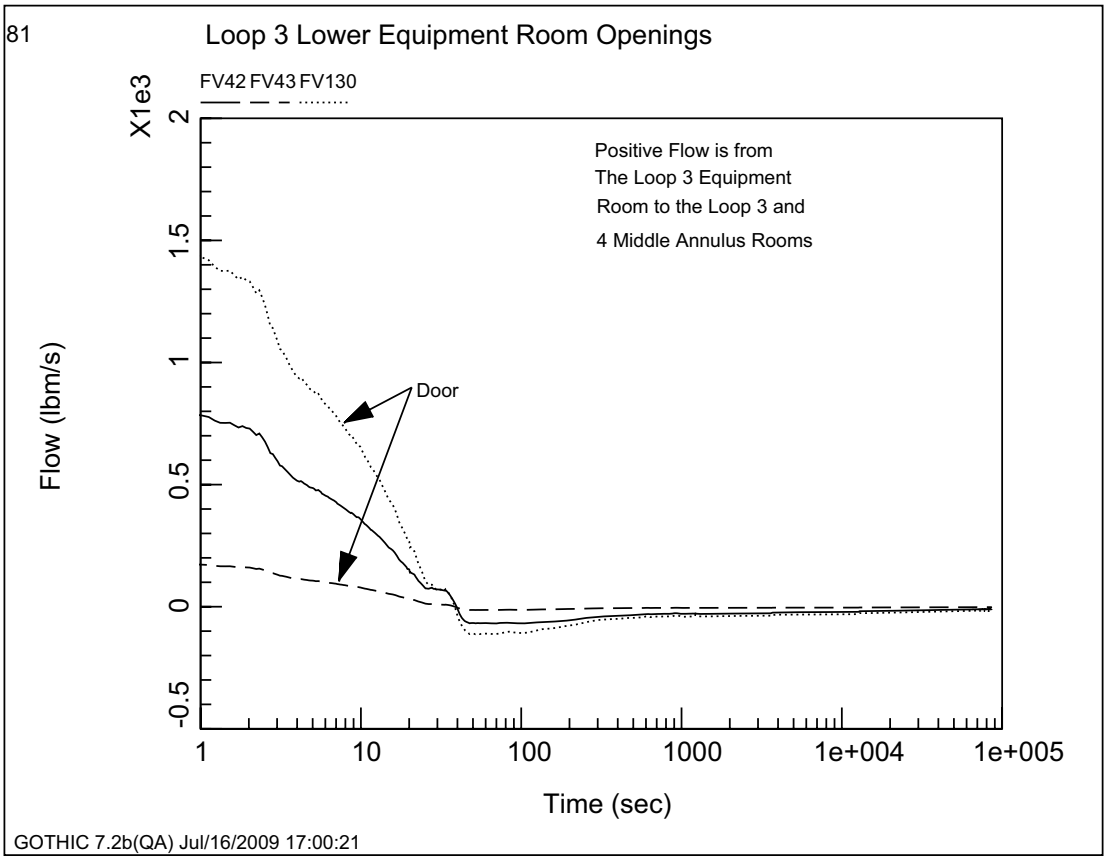


Figure 6.2.1-07d-20—Flow from the Loop 4 Lower Equipment Room to Adjacent Rooms

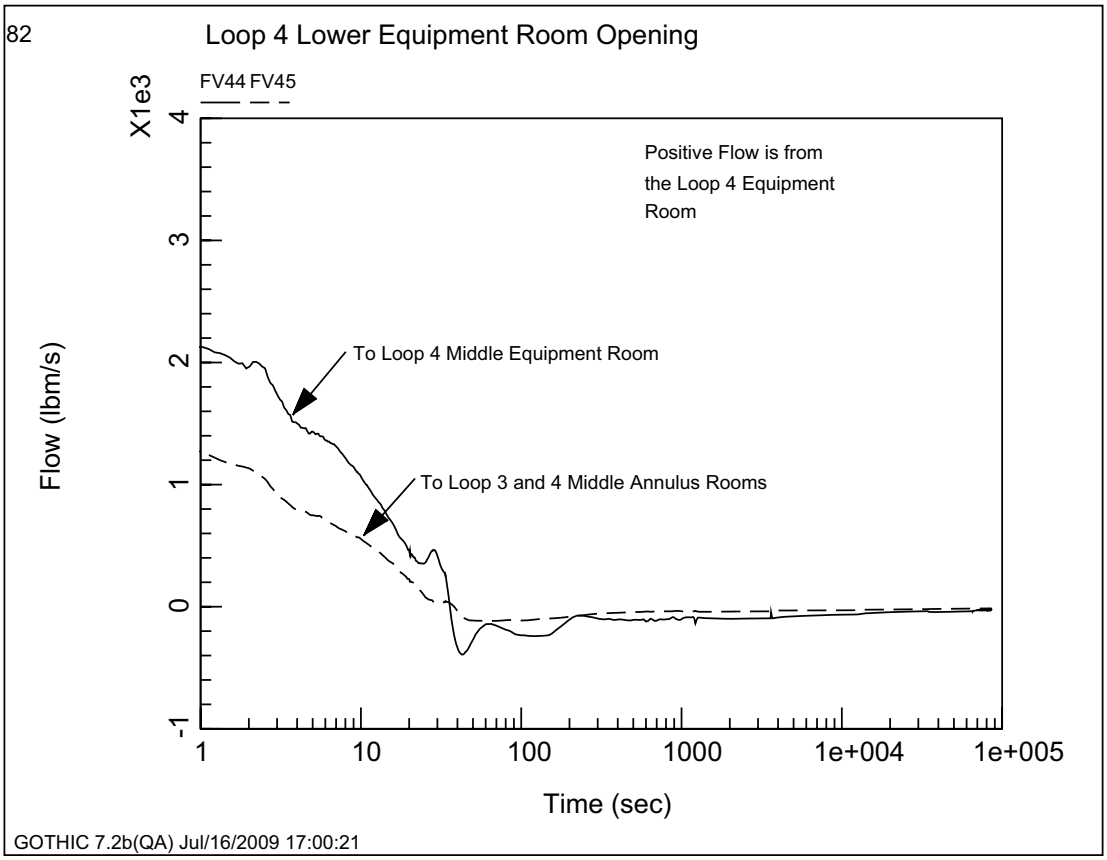


Figure 6.2.1-07d-21—Flow from the Loop 3 Middle Equipment Room to Adjacent Rooms

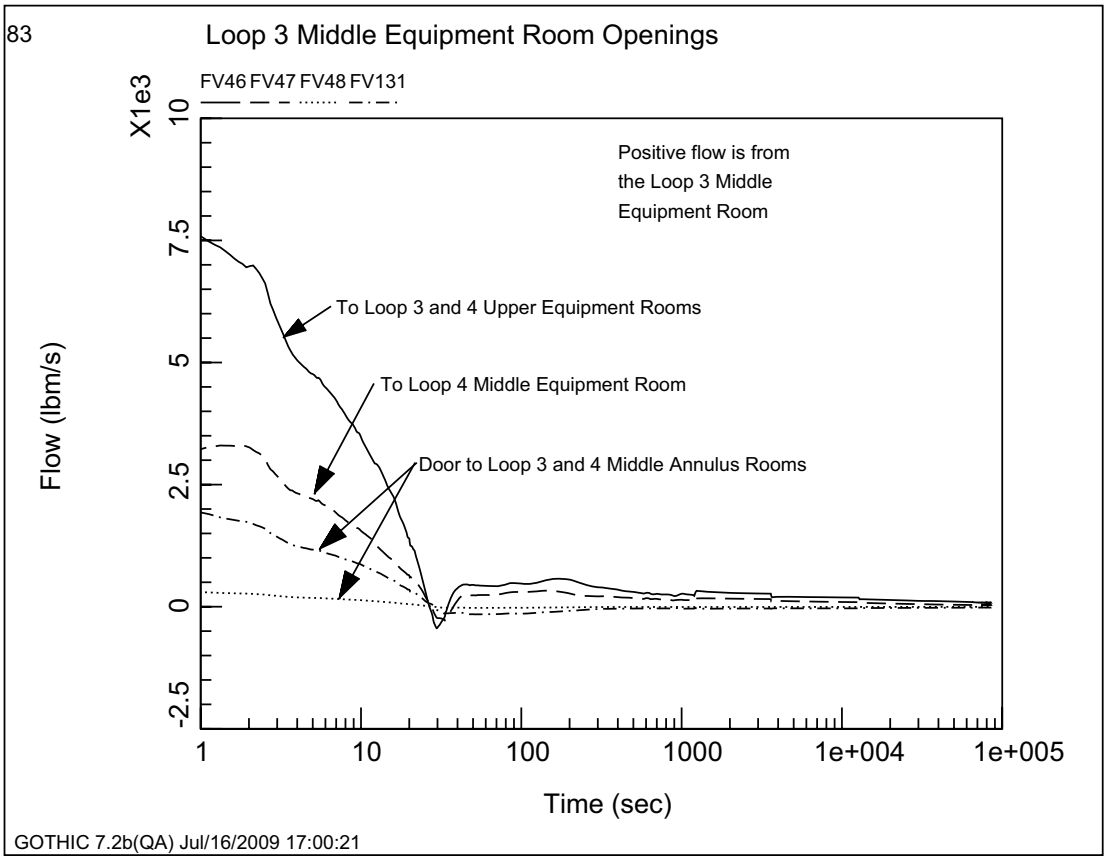


Figure 6.2.1-07d-22—Flow from the Loop 4 Middle Equipment Room to Adjacent Rooms

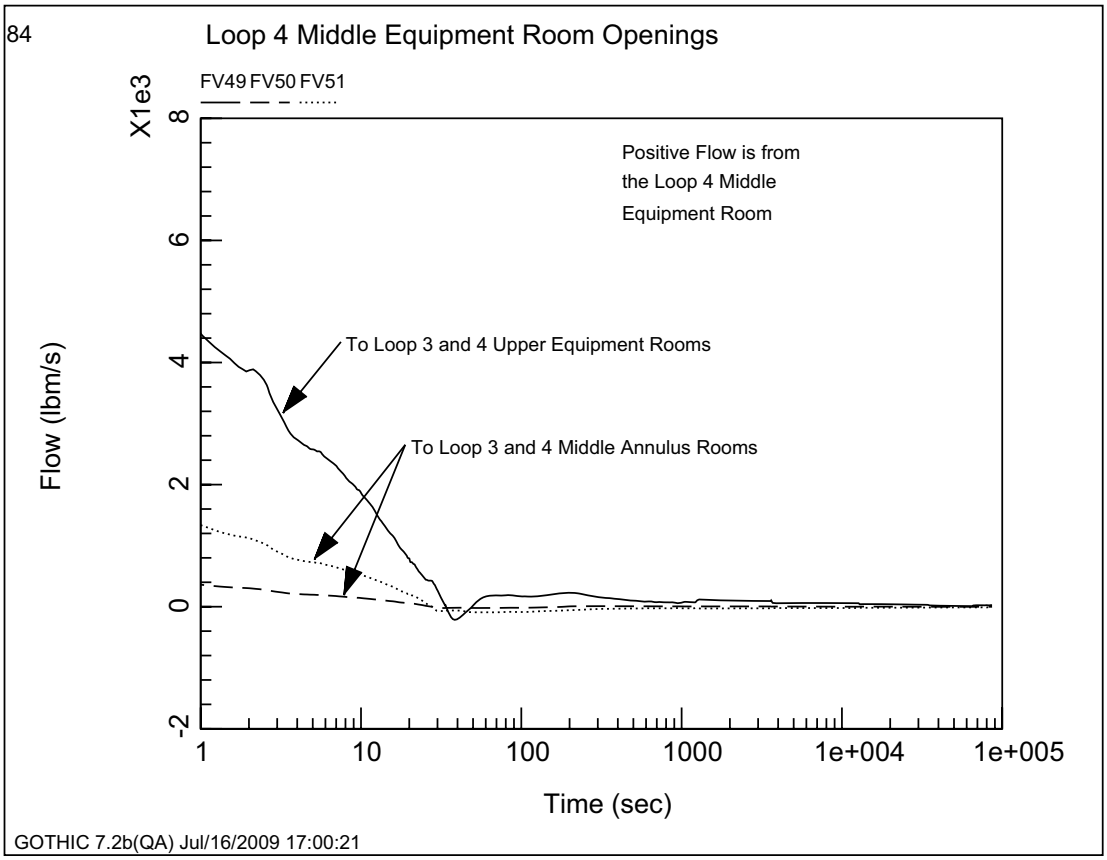


Figure 6.2.1-07d-23—Flow from the Loop 3 and 4 Upper Equipment Room to Adjacent Rooms

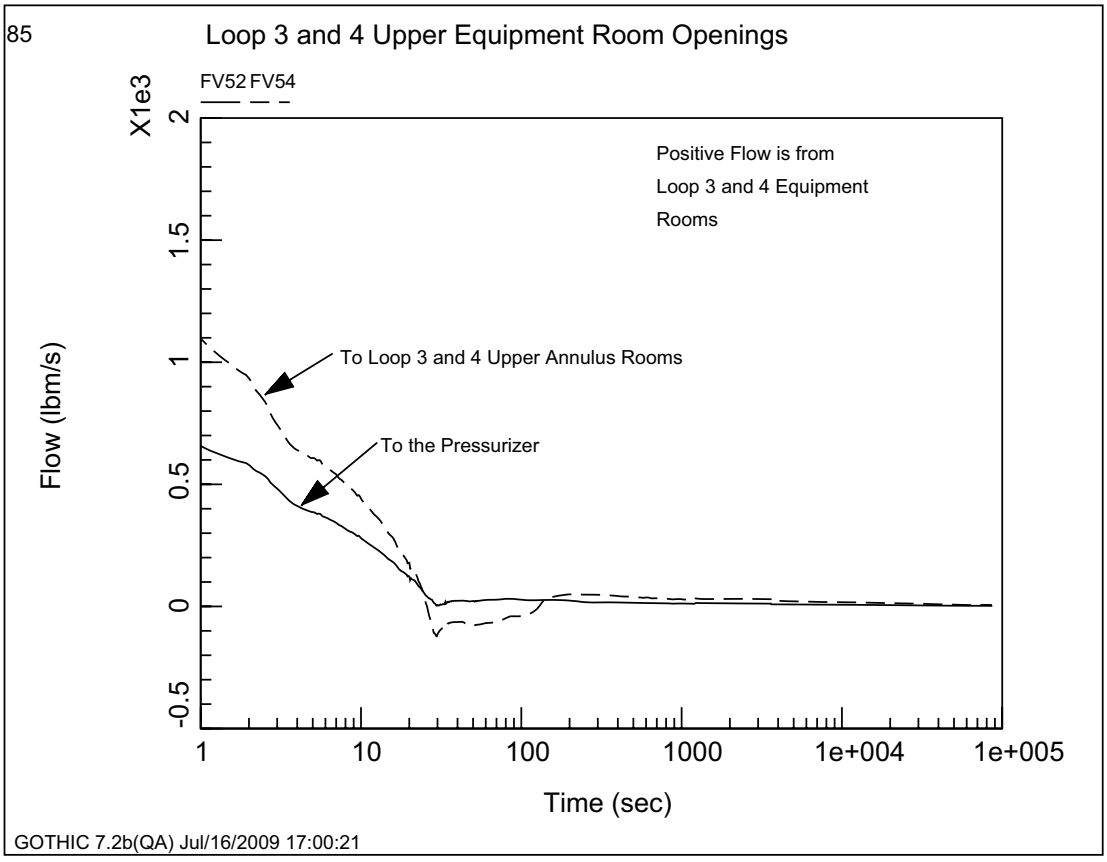


Figure 6.2.1-07d-24—Flow from the Pressurizer Surge Line Room to Adjacent Rooms

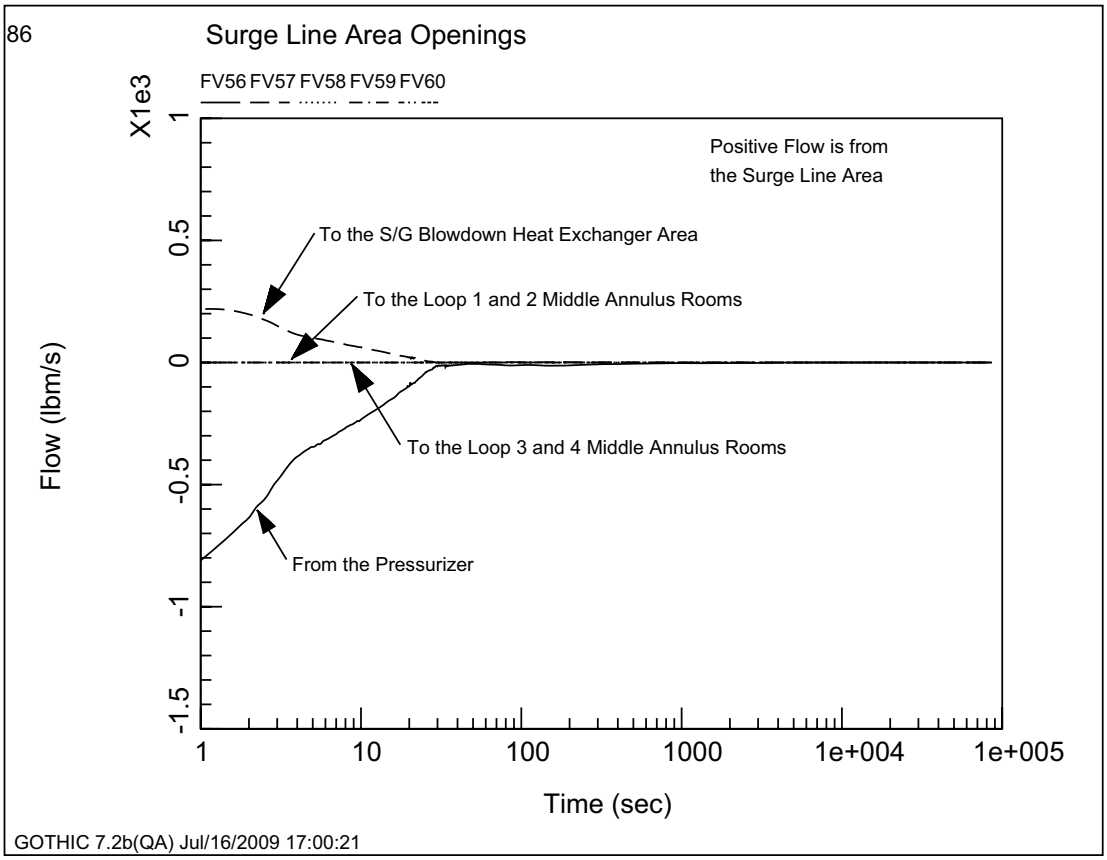
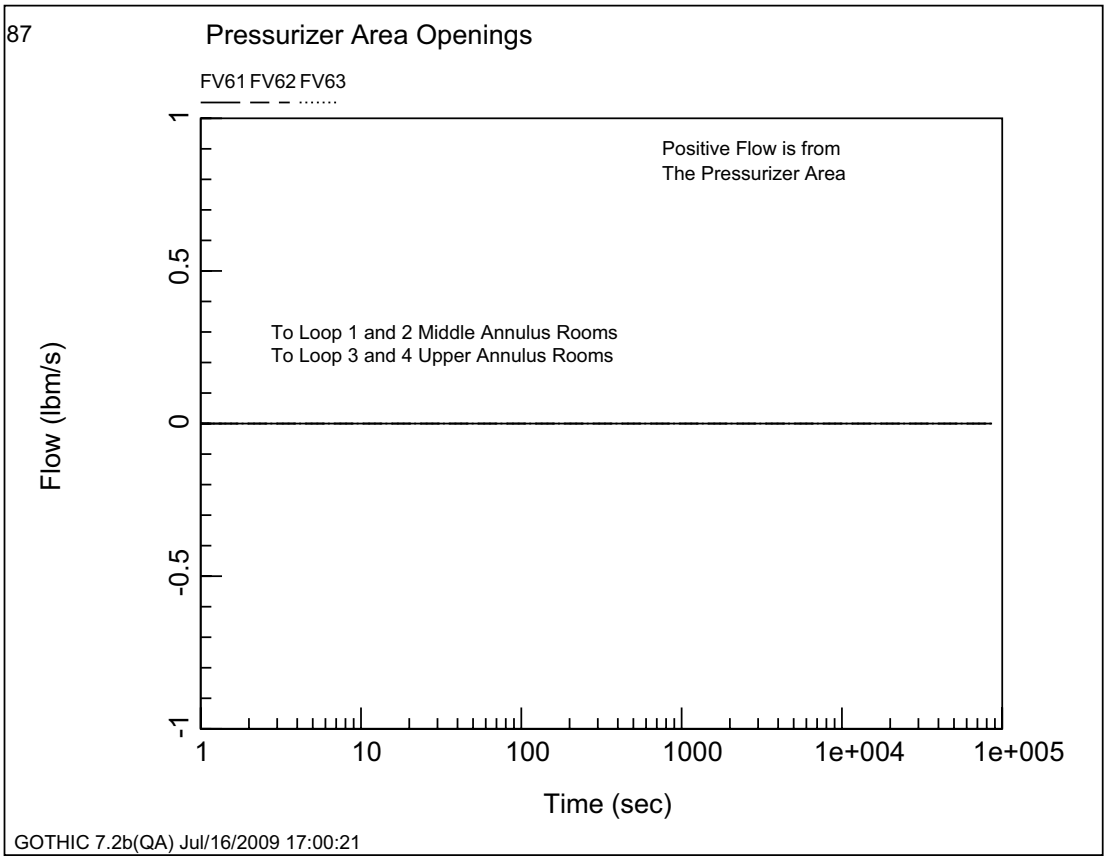


Figure 6.2.1-07d-25—Flow from the Pressurizer Room to Adjacent Rooms



Flow Rates Internal to the Accessible Area

Figure 6.2.1-07d-26—Flow from the CVCS Room to Adjacent Areas

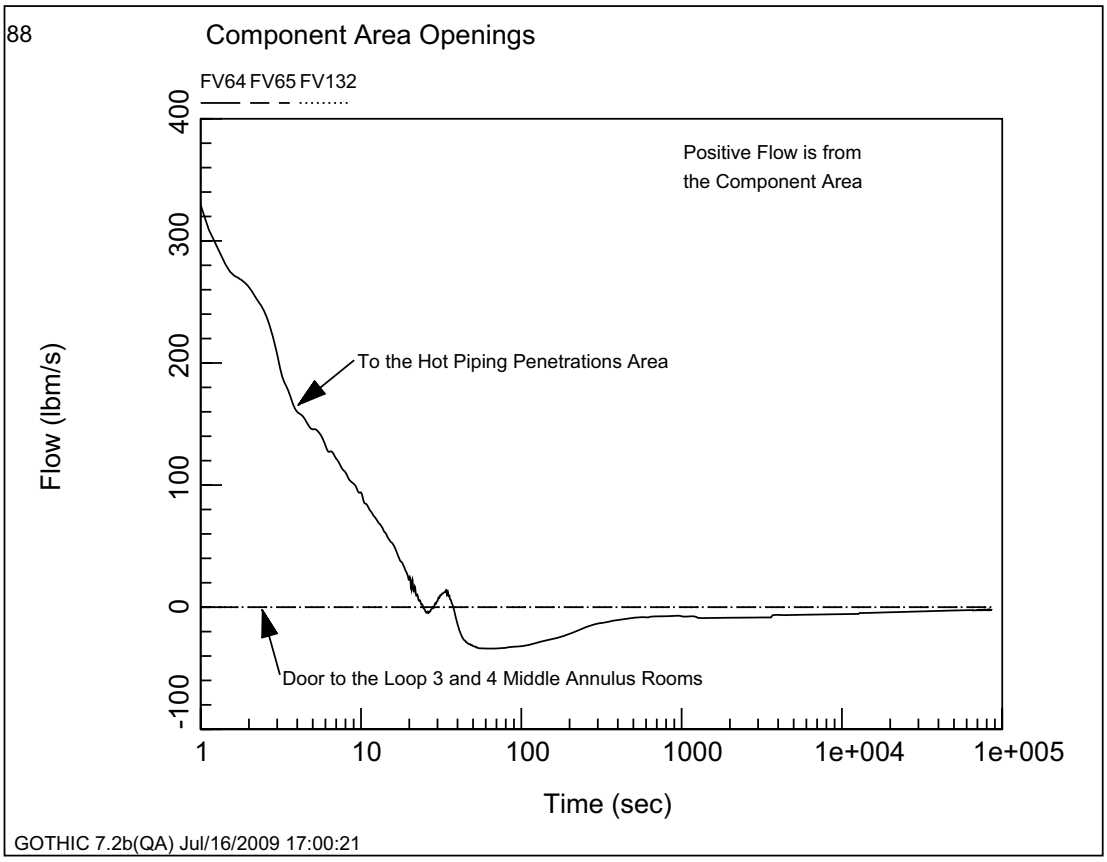


Figure 6.2.1-07d-27—Flow from the Steam Generator Blowdown Heat Exchanger Room to Adjacent Areas

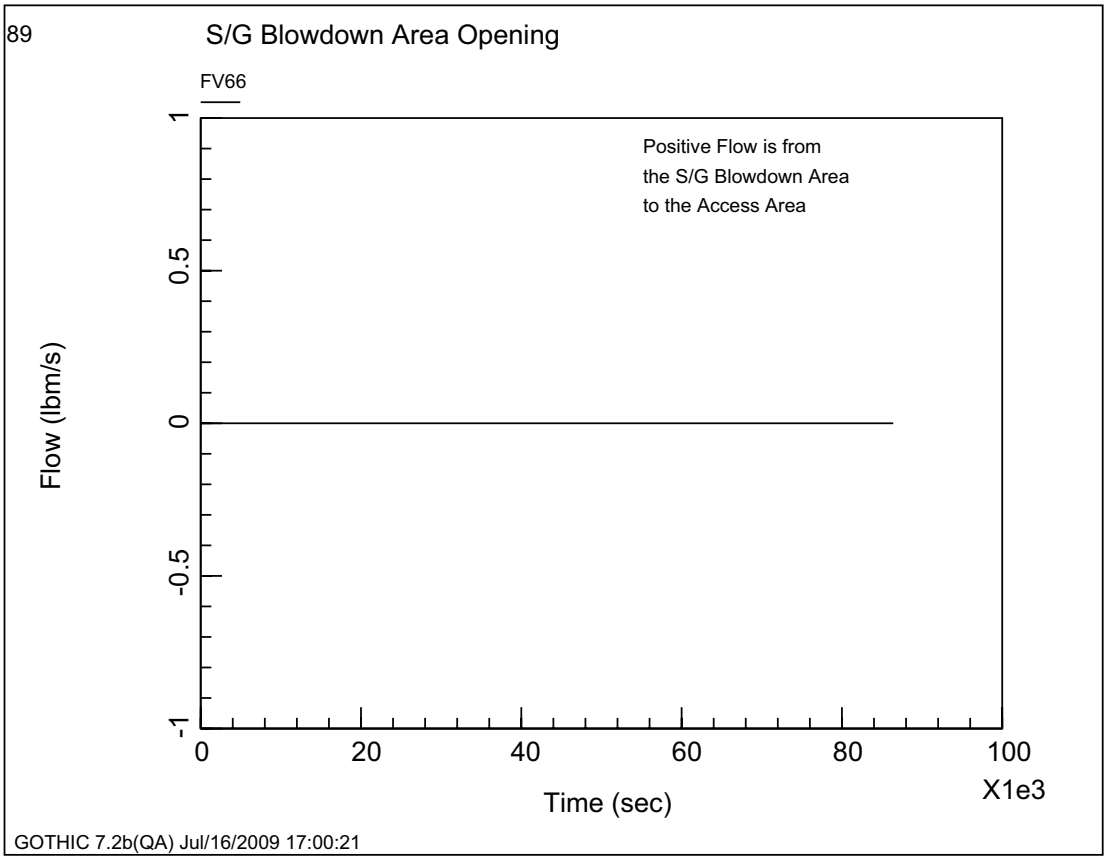


Figure 6.2.1-07d-28—Flow from the Loop 1 and 2 Lower Annulus Room to Adjacent Rooms

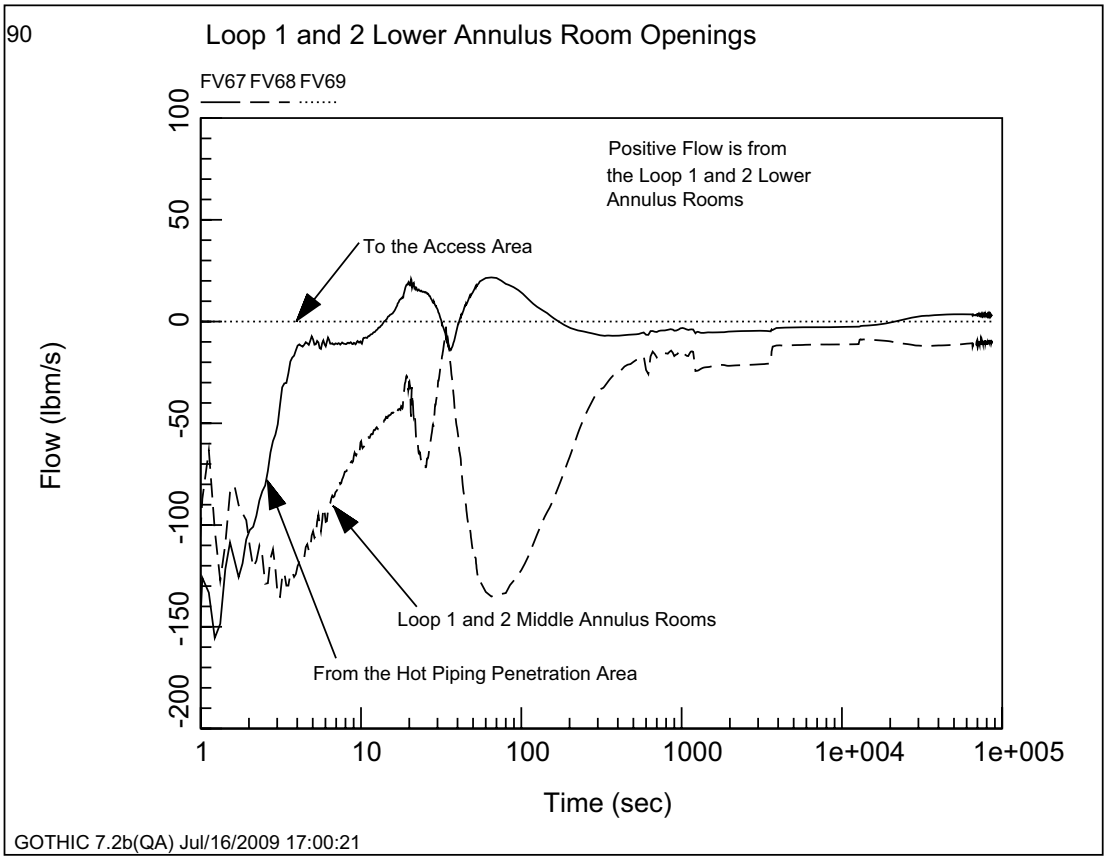


Figure 6.2.1-07d-29—Flow from the Loop 3 and 4 Lower Annulus Room to Adjacent Rooms

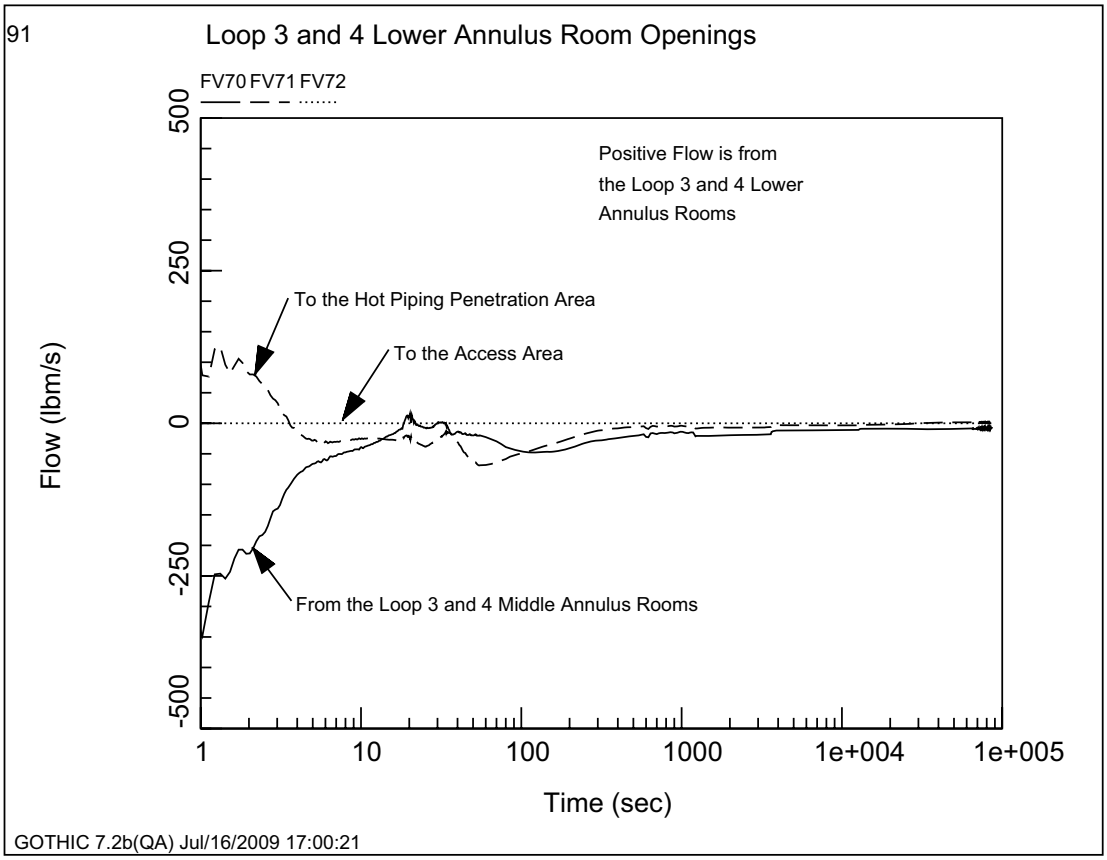


Figure 6.2.1-07d-30—Flow from the Loop 1 and 2 Middle Annulus Room to Adjacent Rooms

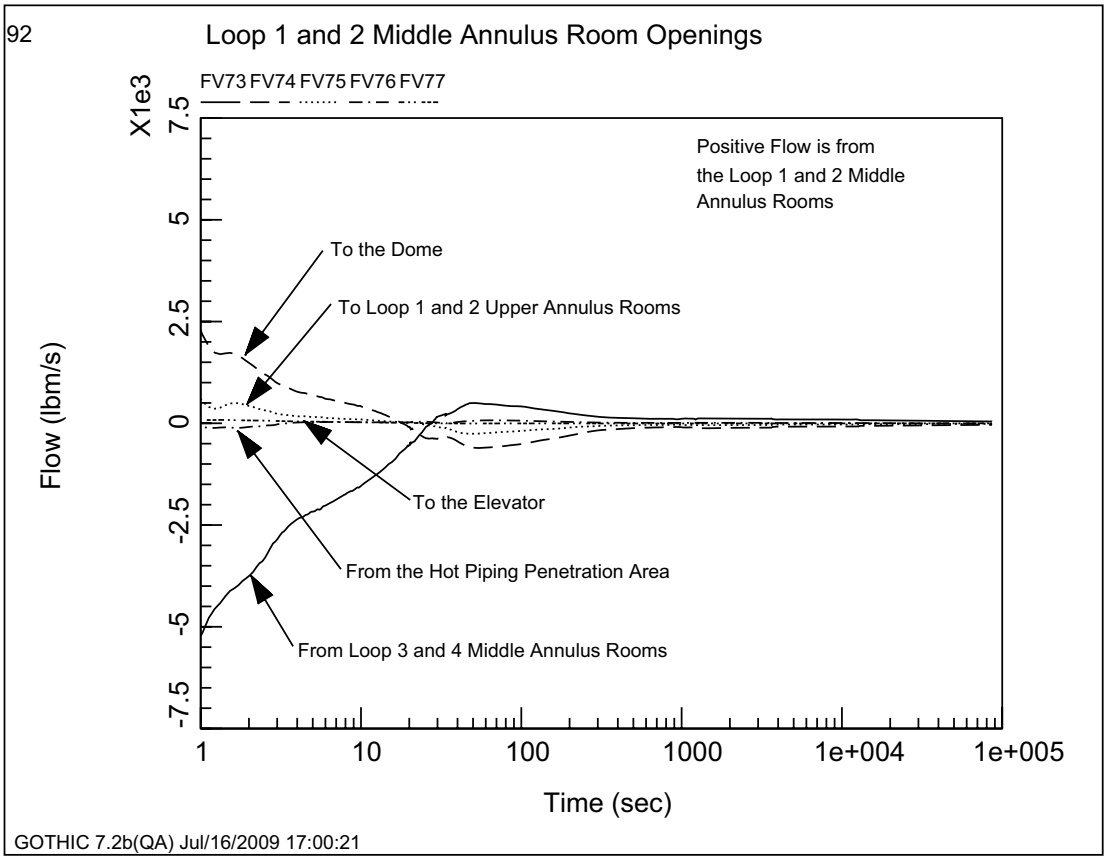


Figure 6.2.1-07d-31—Flow from the Loop 1 and 2 Middle Annulus Room to Adjacent Rooms

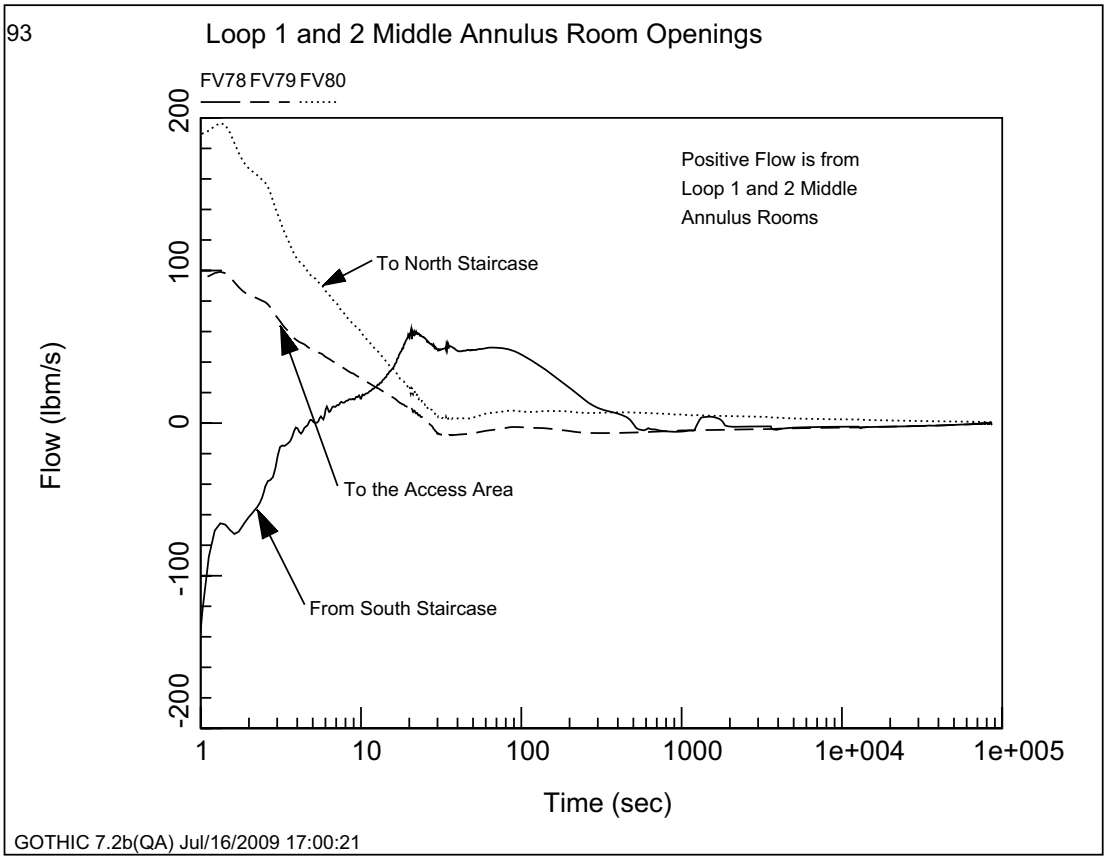


Figure 6.2.1-07d-32—Flow from the Loop 3 and 4 Middle Annulus Room to Adjacent Rooms

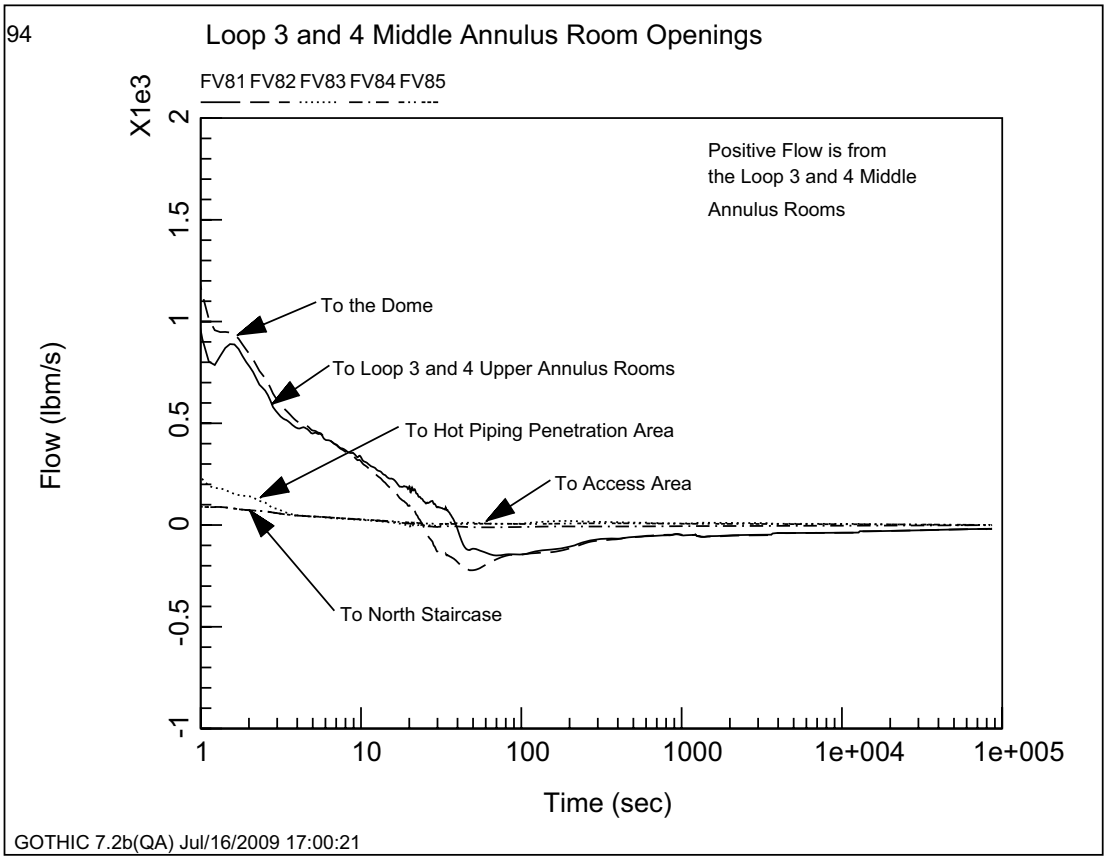


Figure 6.2.1-07d-33—Flow from the Loop 1 and 2 Upper Annulus Room to Adjacent Rooms

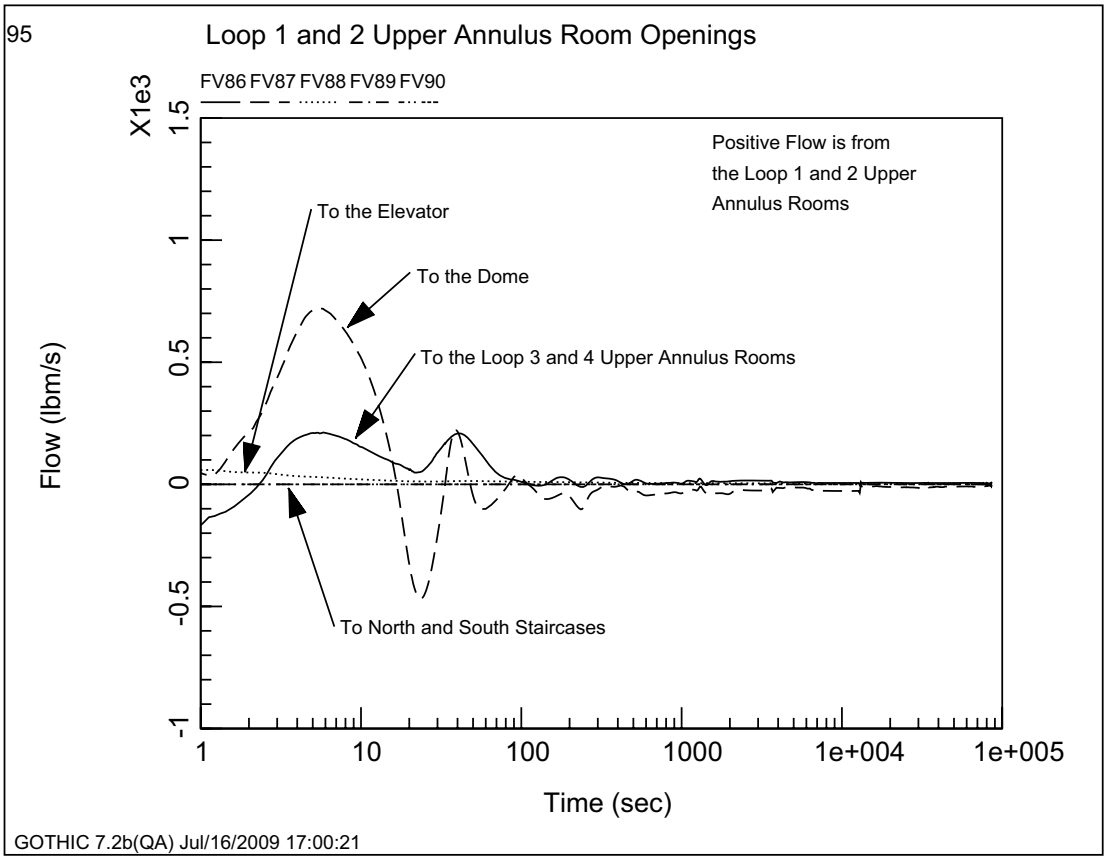


Figure 6.2.1-07d-34—Flow from the Loop 3 and 4 Upper Annulus Room to the North Stairwell

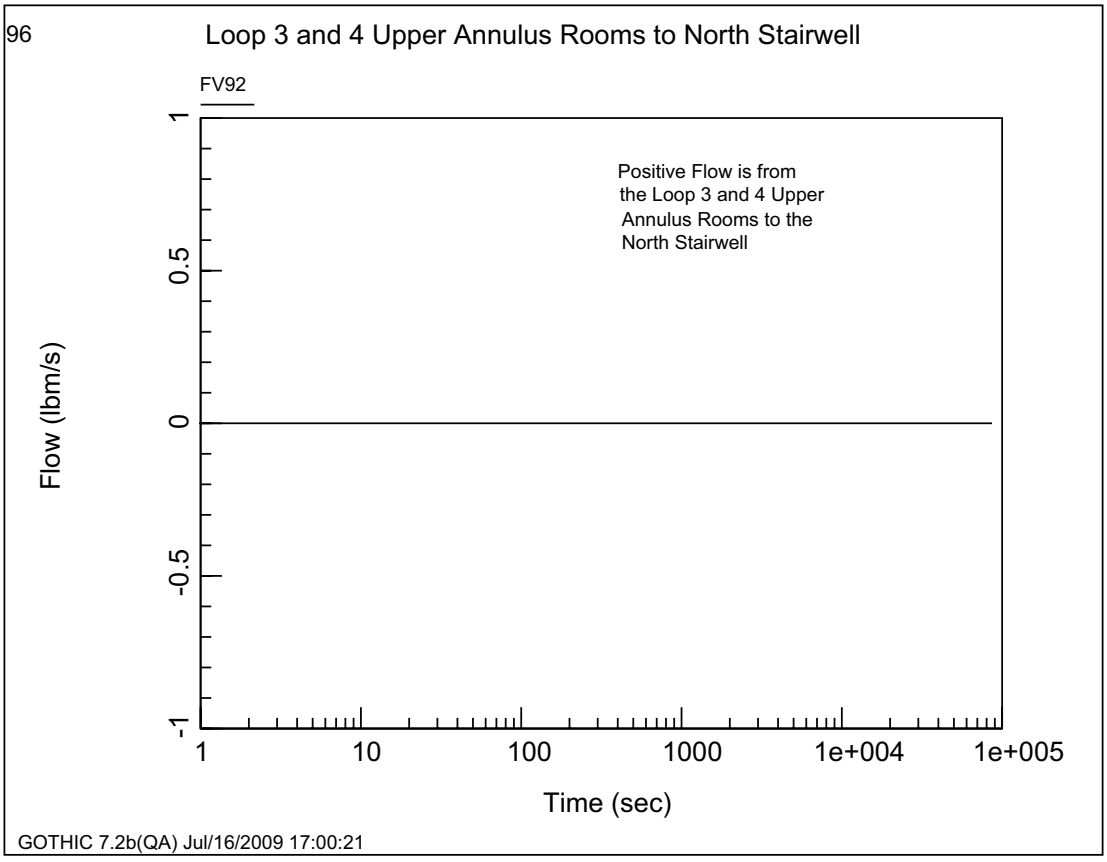


Figure 6.2.1-07d-35—Flow from the Loop 1 and 2 Upper Annulus Room to the Containment Dome

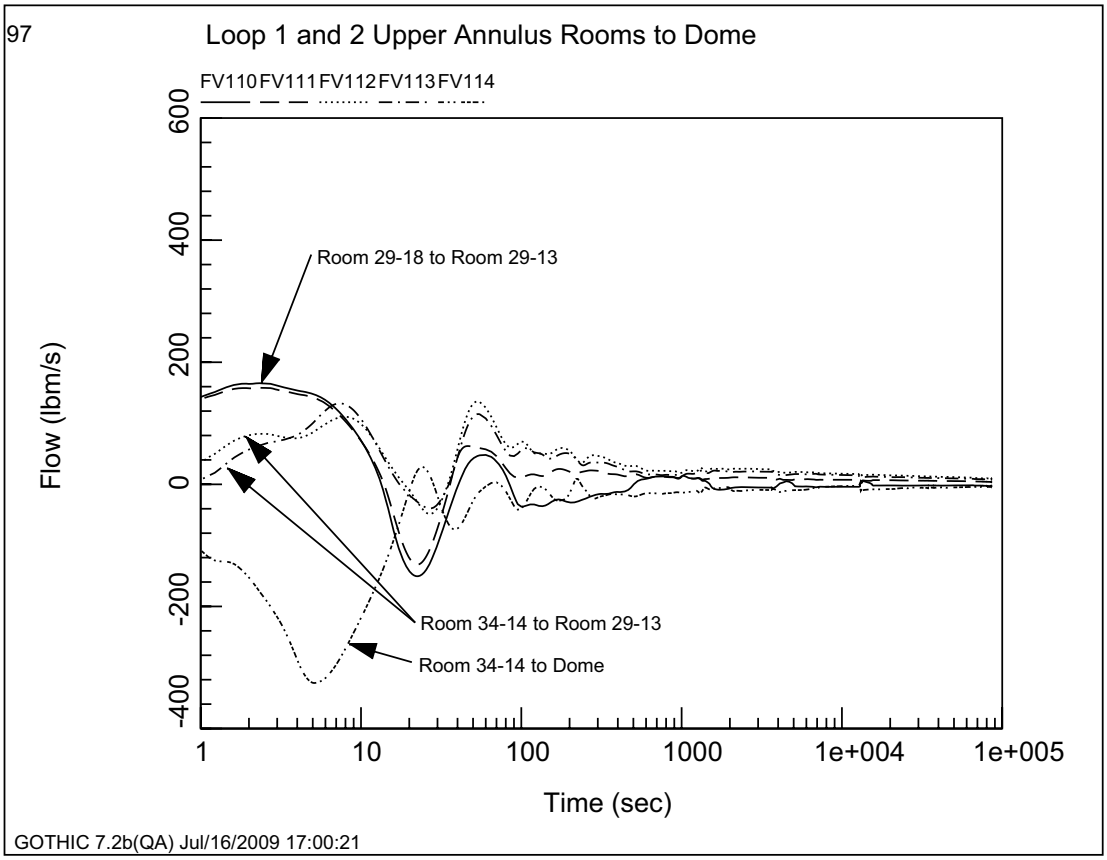


Figure 6.2.1-07d-36—Flow from the Loop 1 and 2 Upper Annulus Room to the Containment Dome

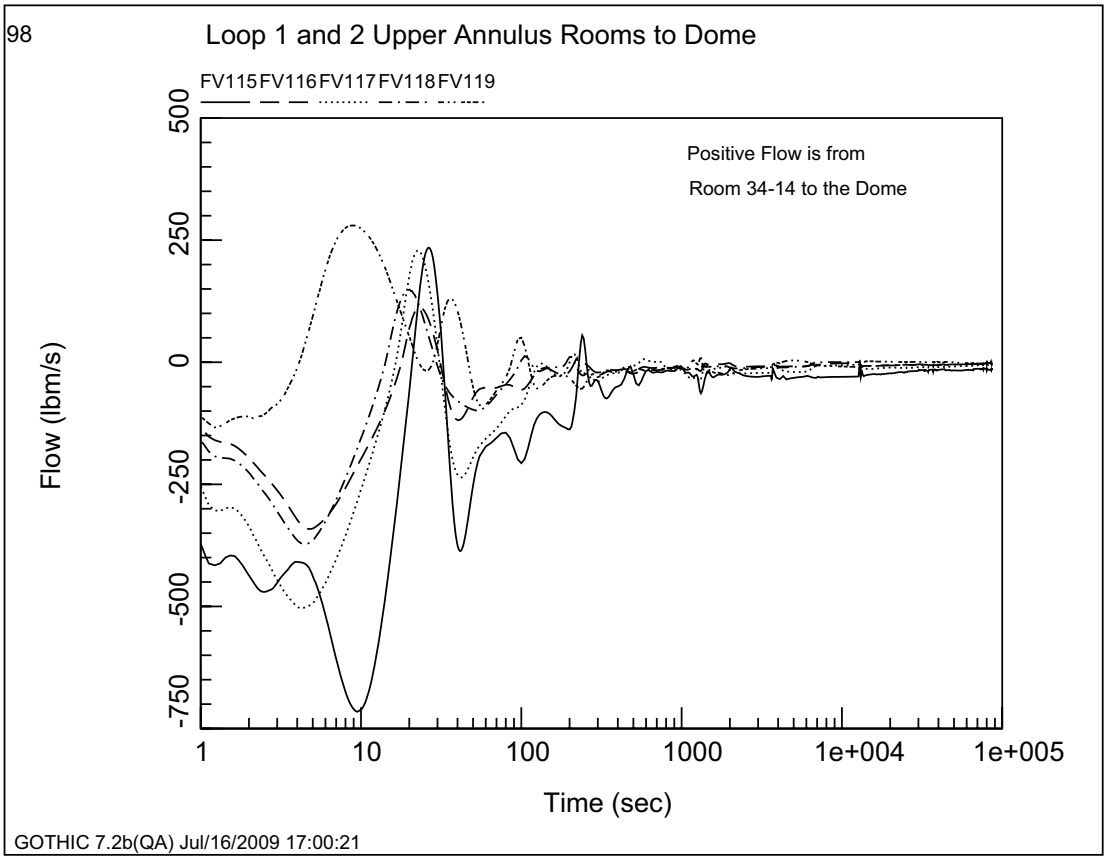


Figure 6.2.1-07d-37—Flow from the Loop 3 and 4 Upper Annulus Room to the Containment Dome

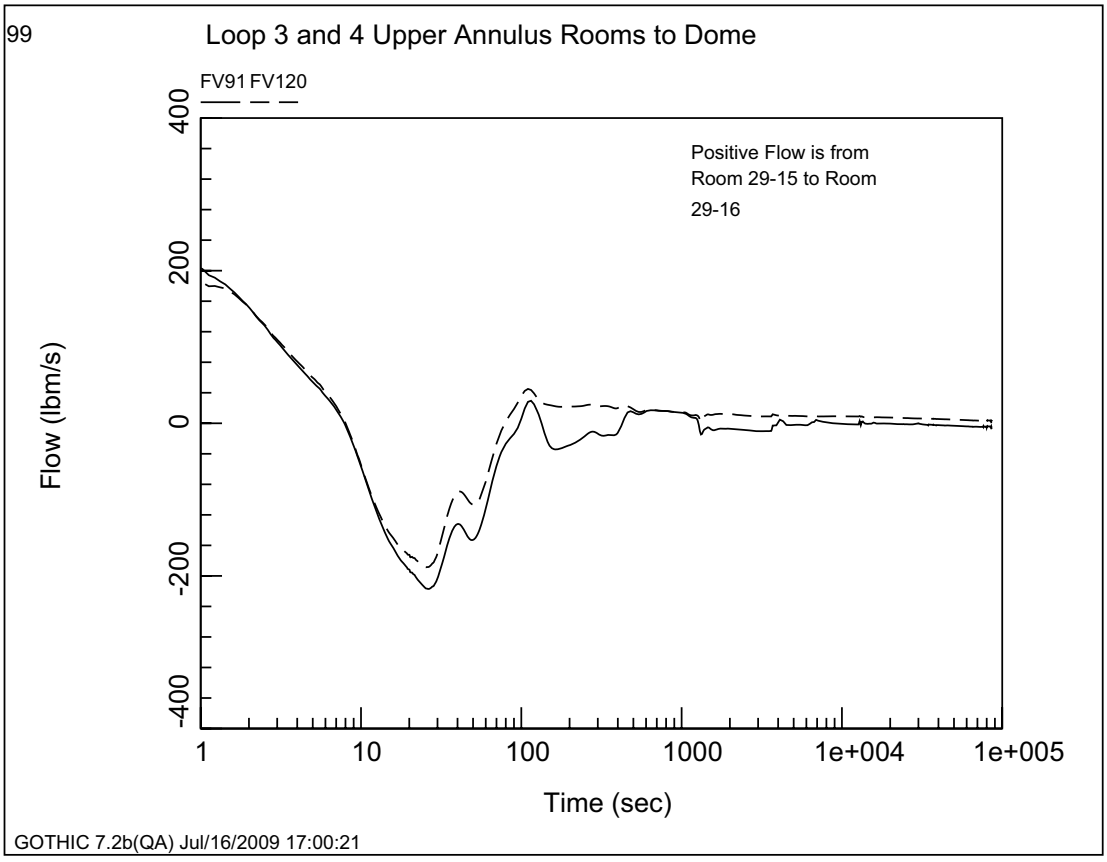


Figure 6.2.1-07d-38—Flow from the Loop 3 and 4 Upper Annulus Room to the Containment Dome

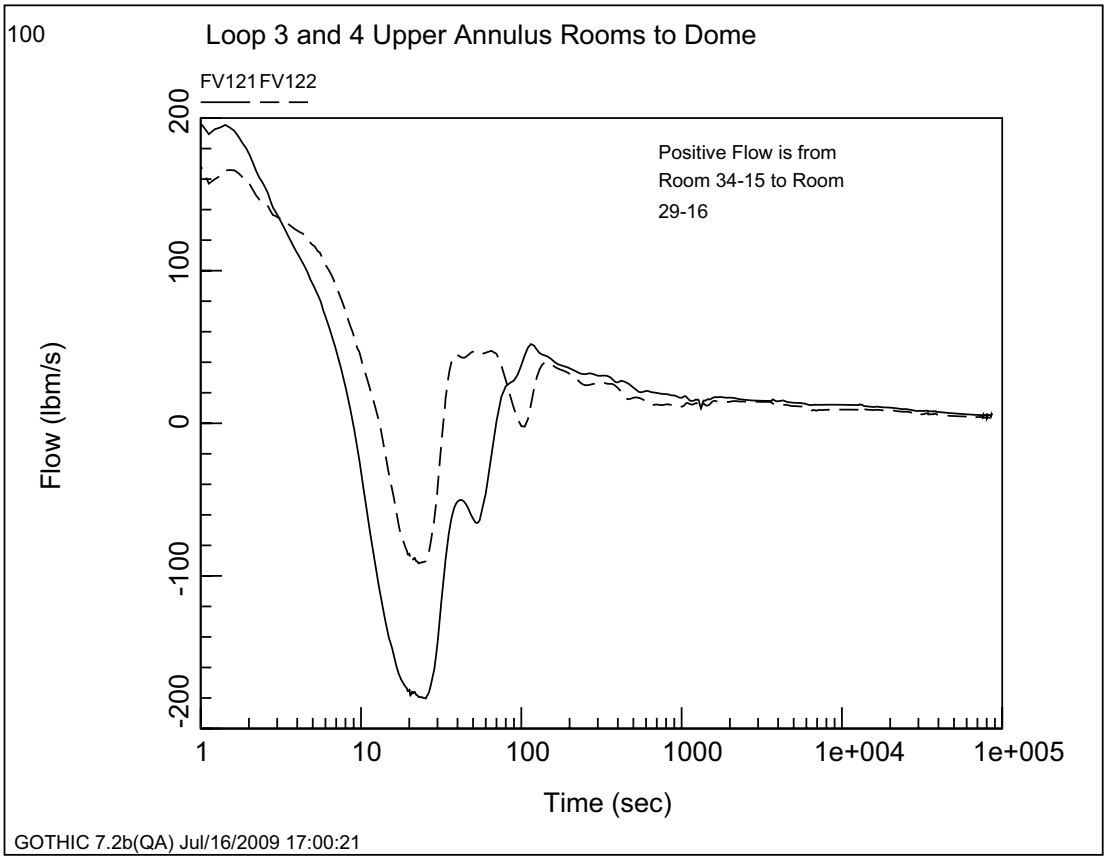


Figure 6.2.1-07d-39—Flow from the Loop 3 and 4 Upper Annulus Room to the Containment Dome

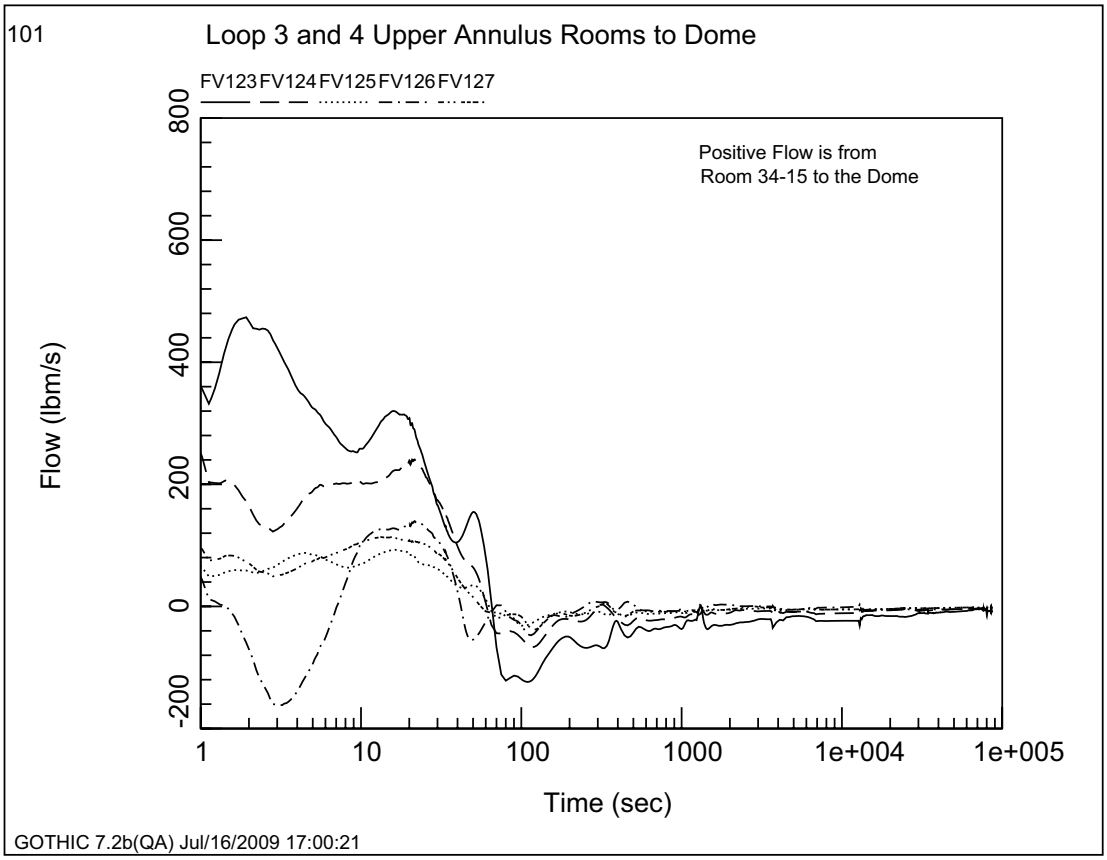


Figure 6.2.1-07d-40—Flow from the Containment Dome to the South Stairwell

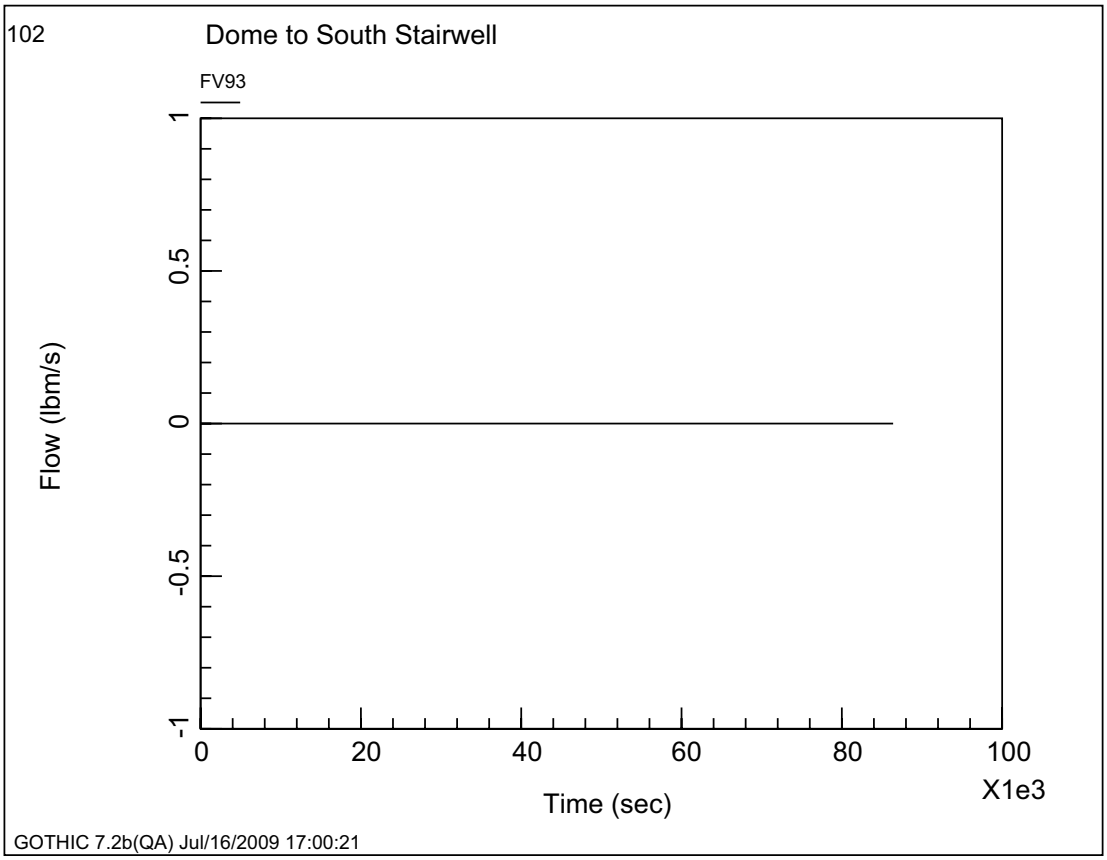
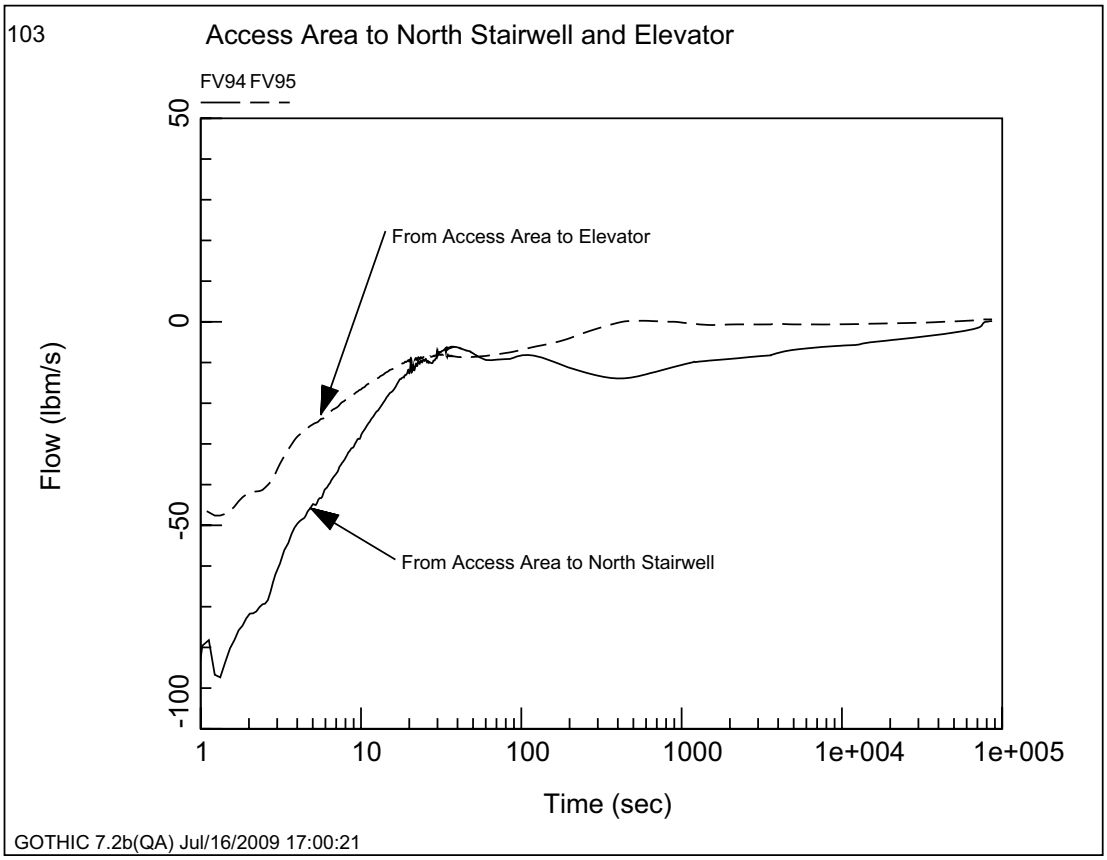


Figure 6.2.1-07d-41—Flow from the Access Area to the North Staircase and the Elevator



Containment Internal Temperatures

Equipment Area

Figure 6.2.1-07d-42—Temperature in the Spreading Room and IRWST

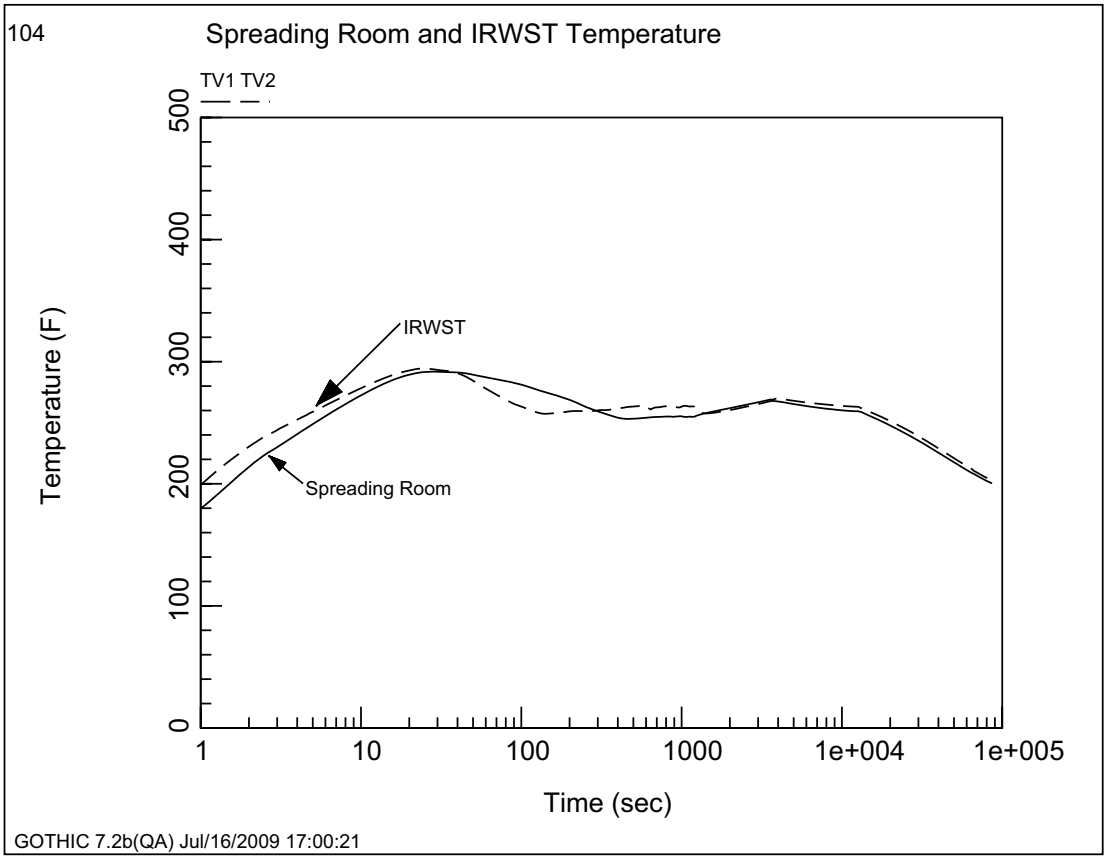


Figure 6.2.1-07d-43—Temperature in the Lower Equipment Rooms

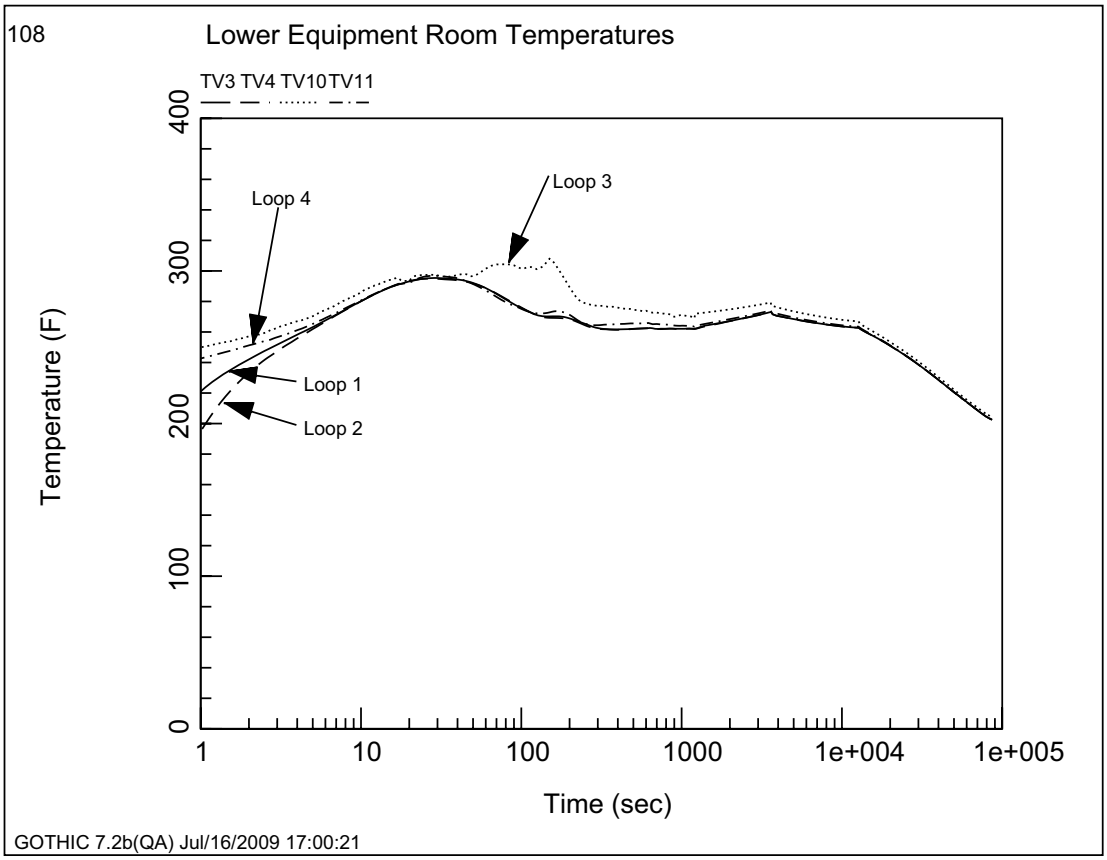


Figure 6.2.1-07d-44—Temperature in the Middle Equipment Rooms

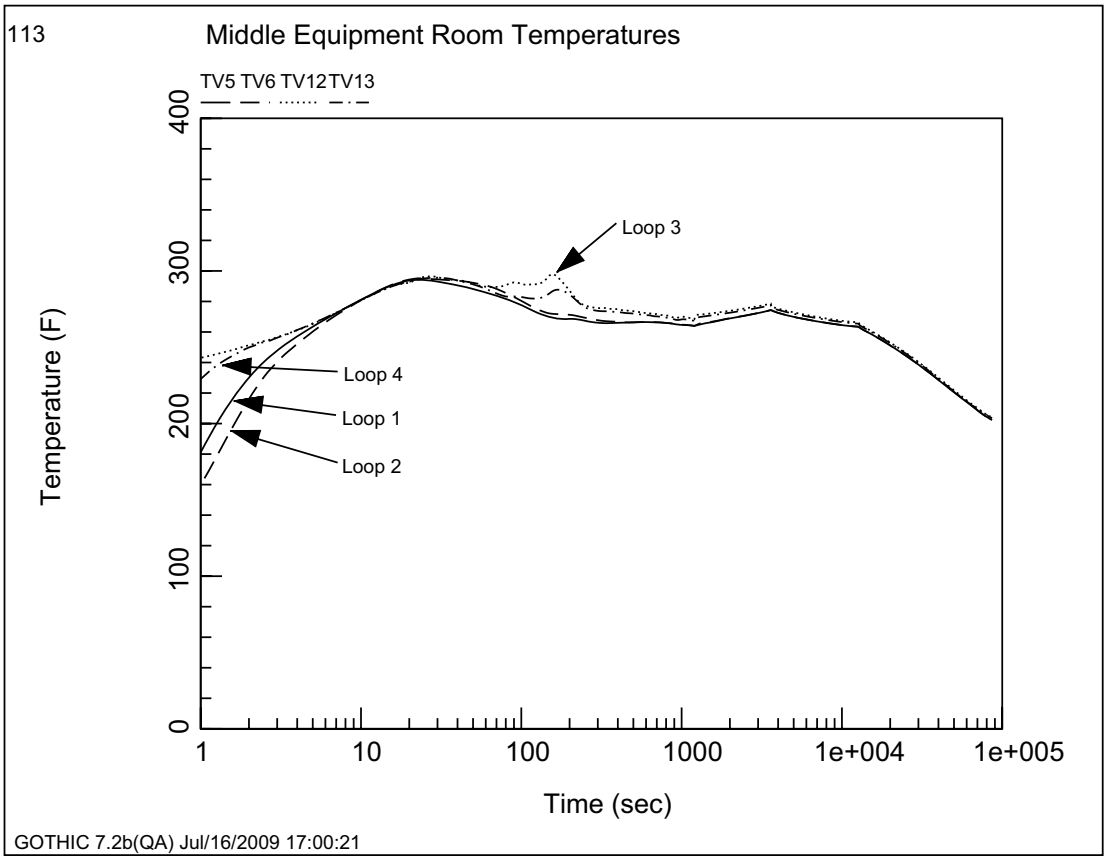


Figure 6.2.1-07d-45—Temperature in the Upper Equipment Rooms

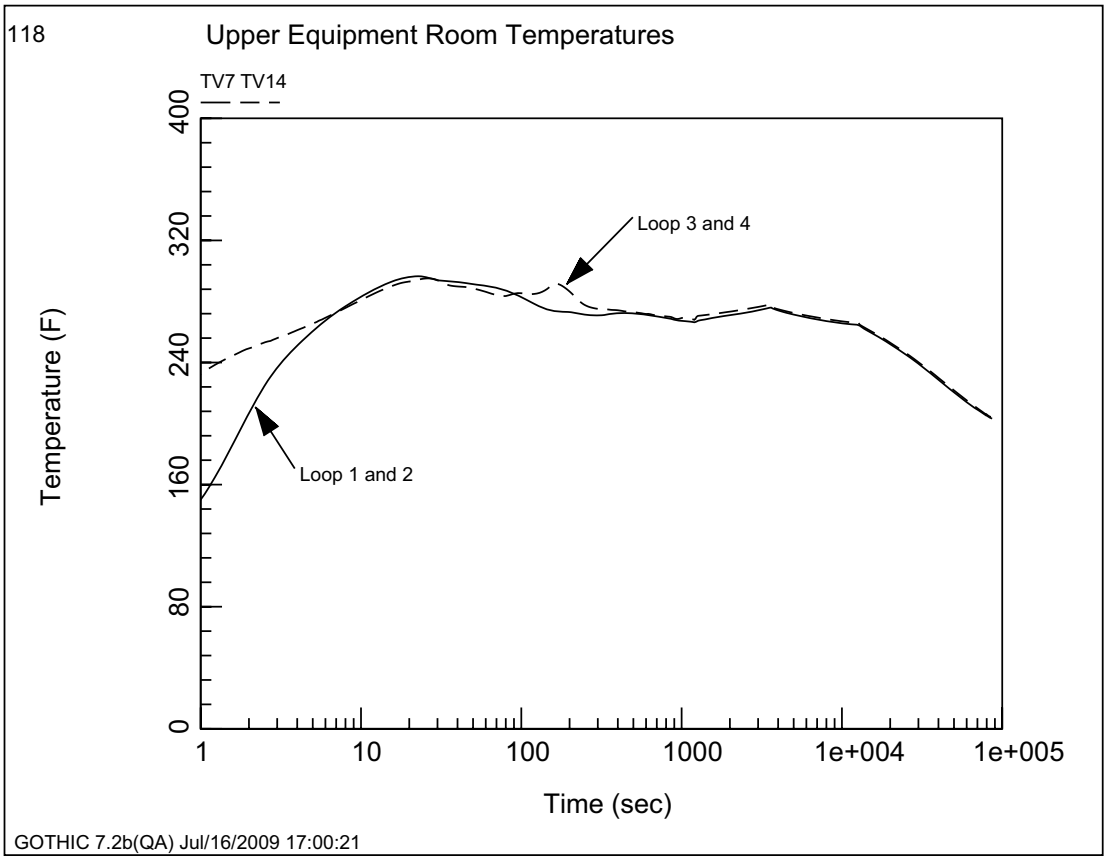


Figure 6.2.1-07d-46—Temperature in the Reactor Pressure Vessel Pit and Reactor Cavity

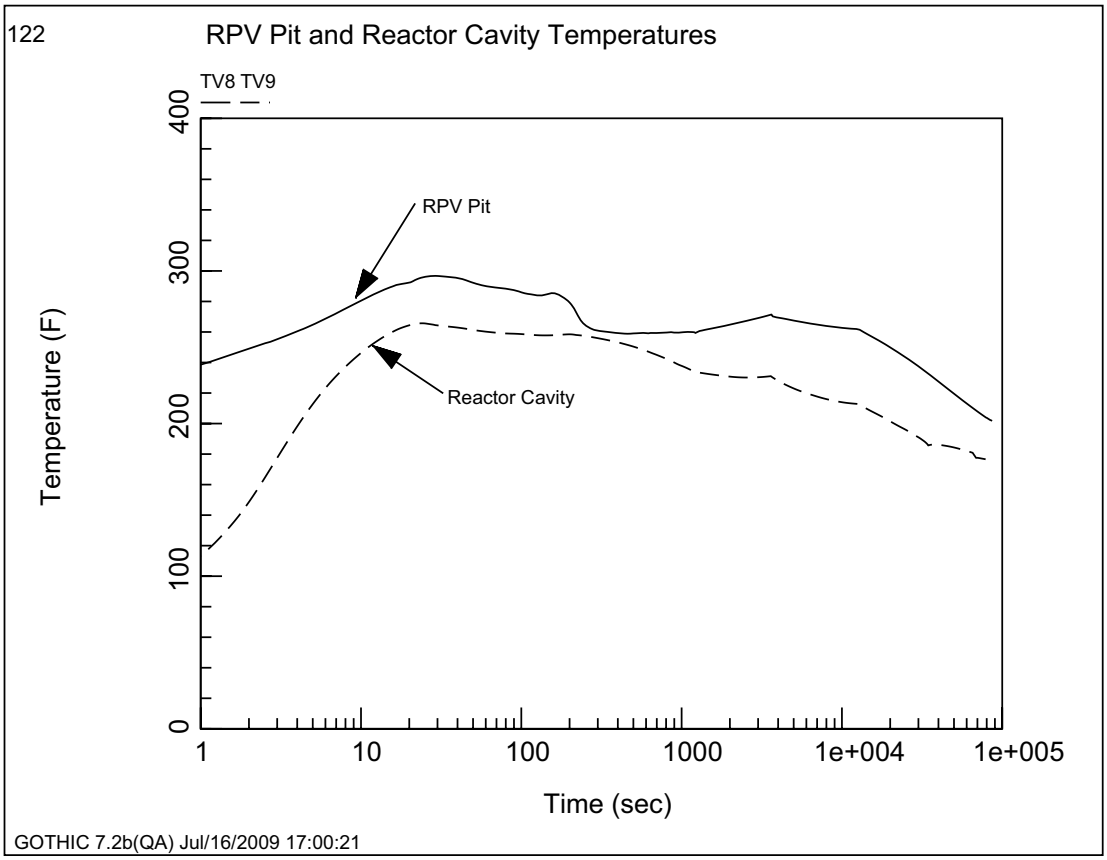
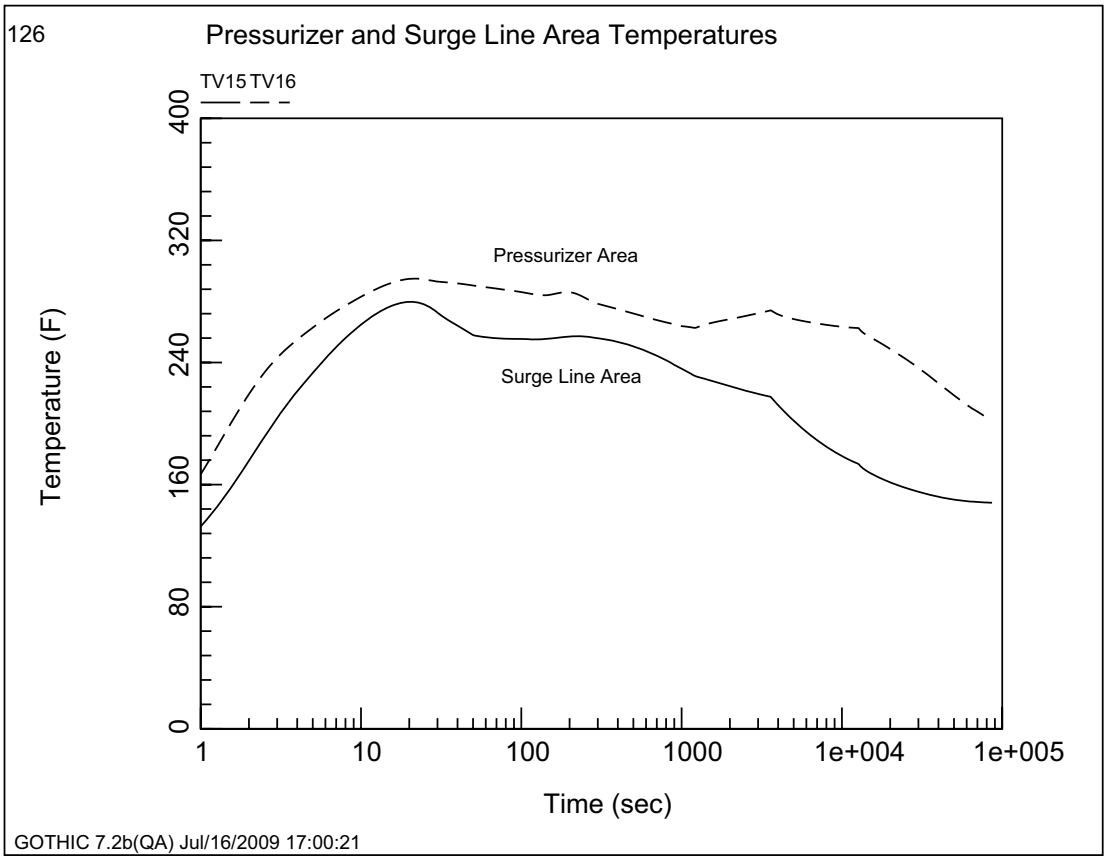


Figure 6.2.1-07d-47—Temperature in the Pressurizer and Surge Line Rooms



Accessible Area

Figure 6.2.1-07d-48—Temperature in the CVCS and Steam Generator Blowdown Heat Exchanger Rooms

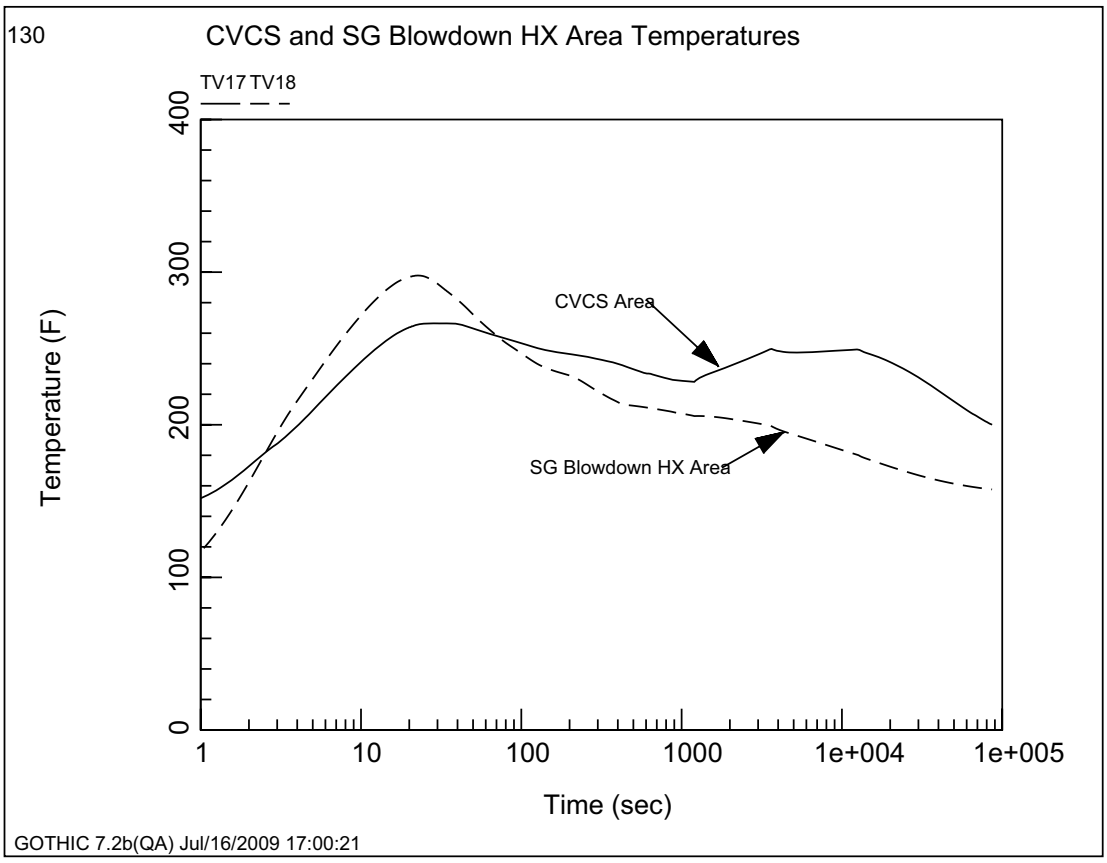


Figure 6.2.1-07d-49—Temperature in the Lower Annulus Rooms

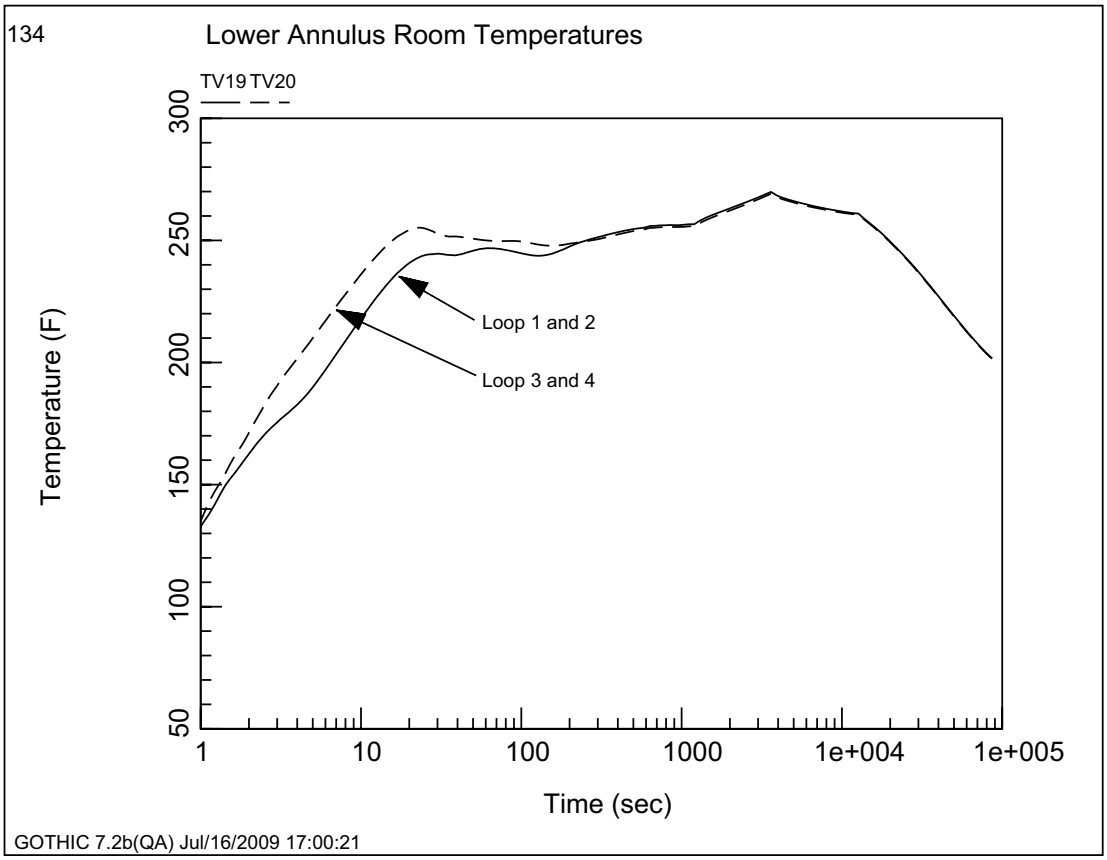


Figure 6.2.1-07d-50—Temperature in the Middle Annulus Rooms

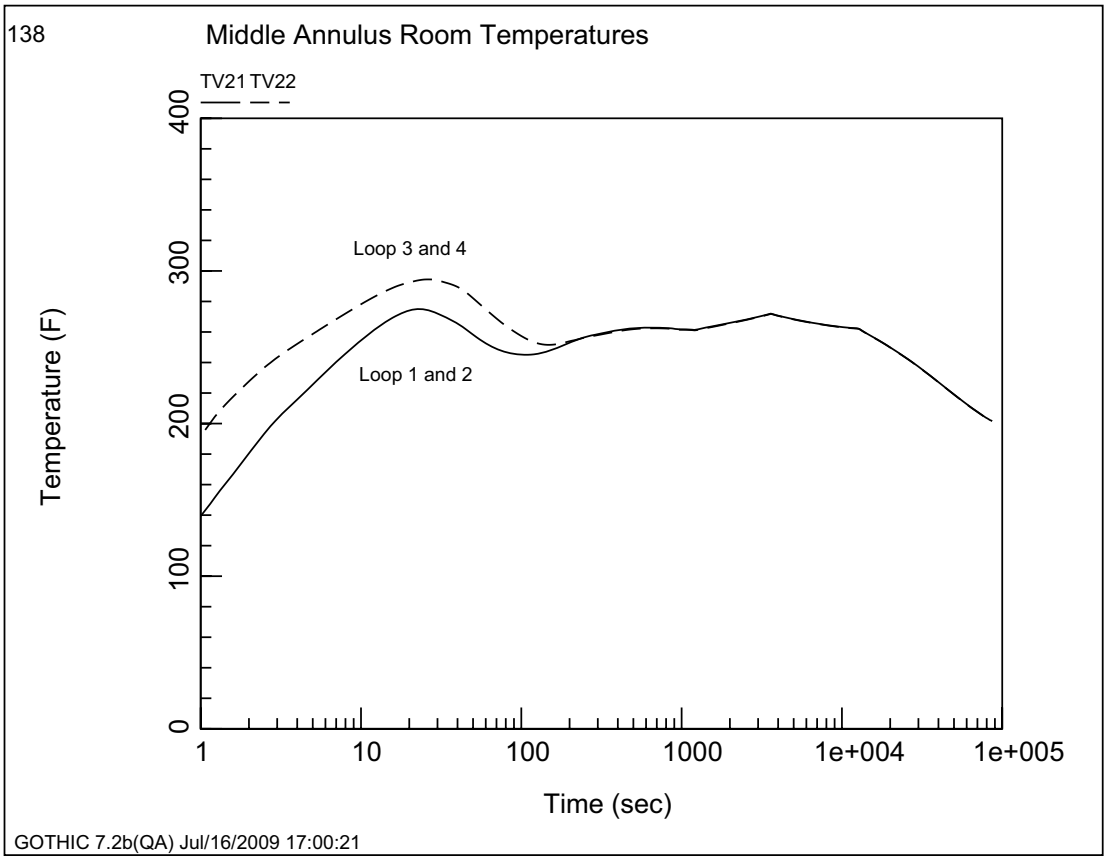


Figure 6.2.1-07d-51—Temperature in the Upper Annulus Rooms

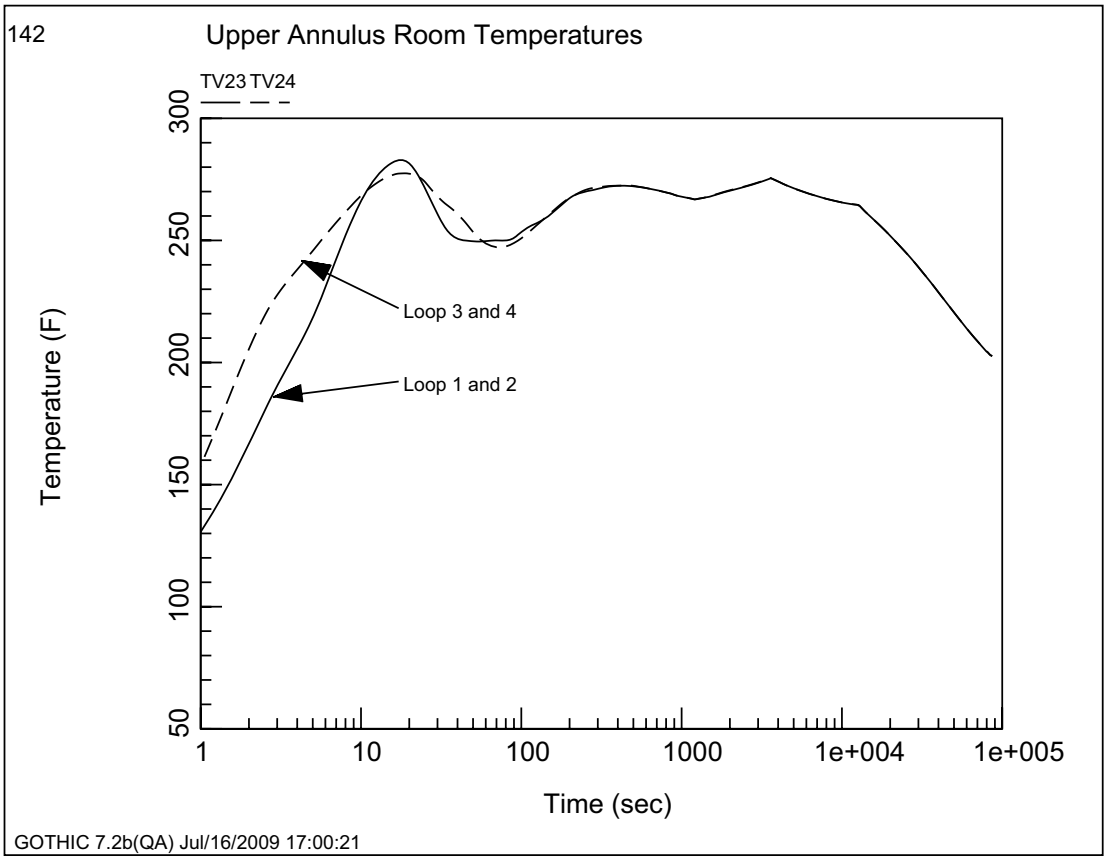


Figure 6.2.1-07d-52—Temperature in the Access and Hot Piping Penetration Rooms

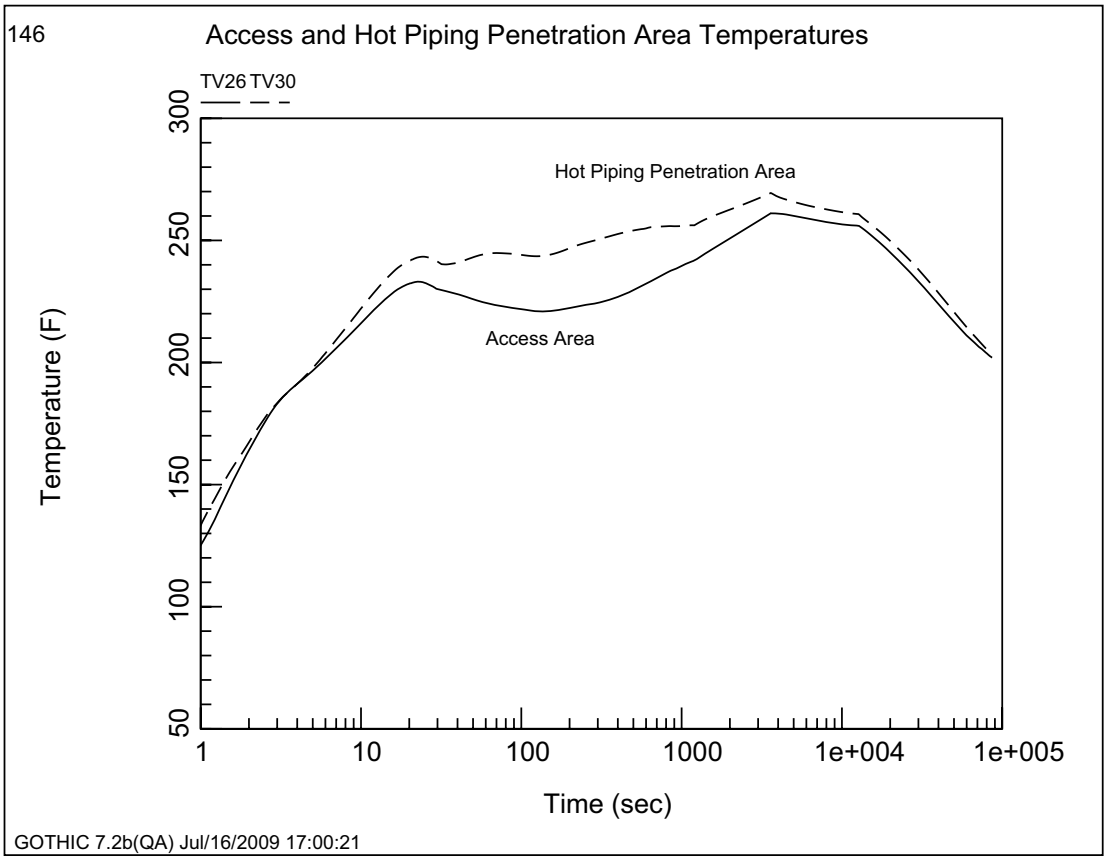


Figure 6.2.1-07d-53—Temperature in the North and South Staircases and in the Elevator

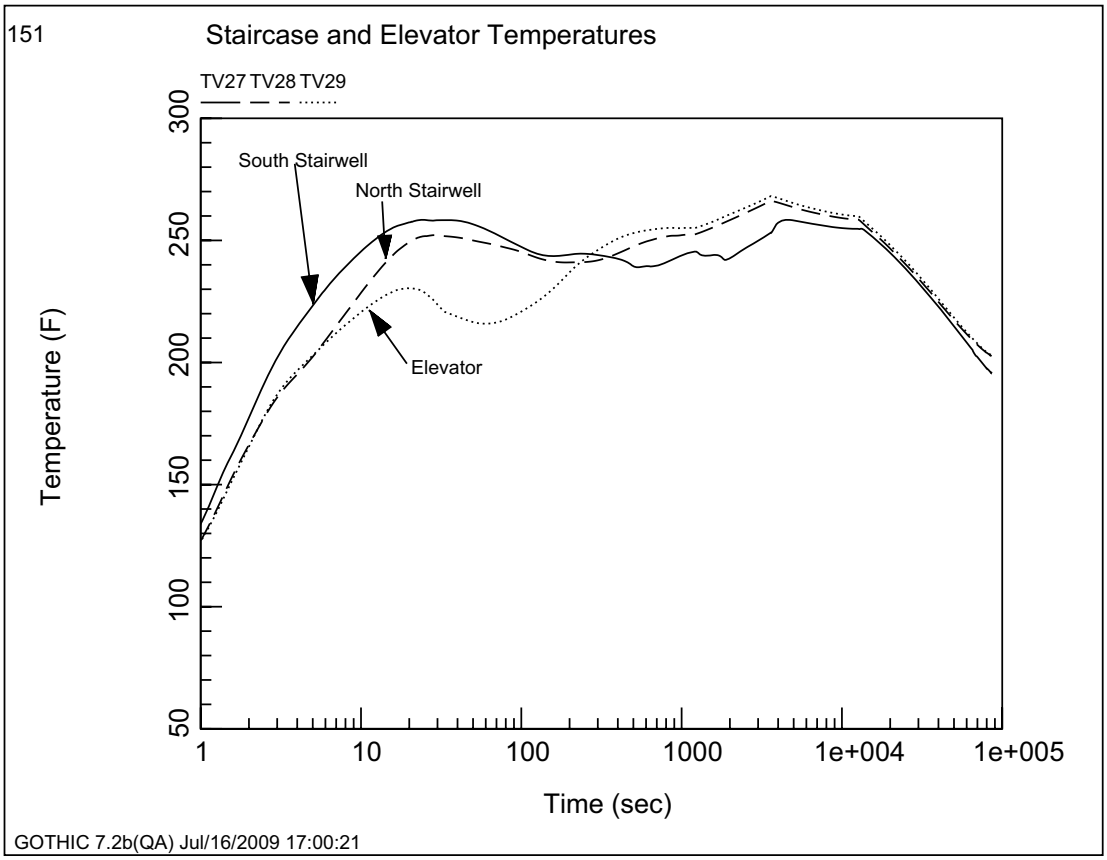


Figure 6.2.1-07d-54—Temperature in the Containment Dome between the 63.98 ft and 71.53 ft Elevations

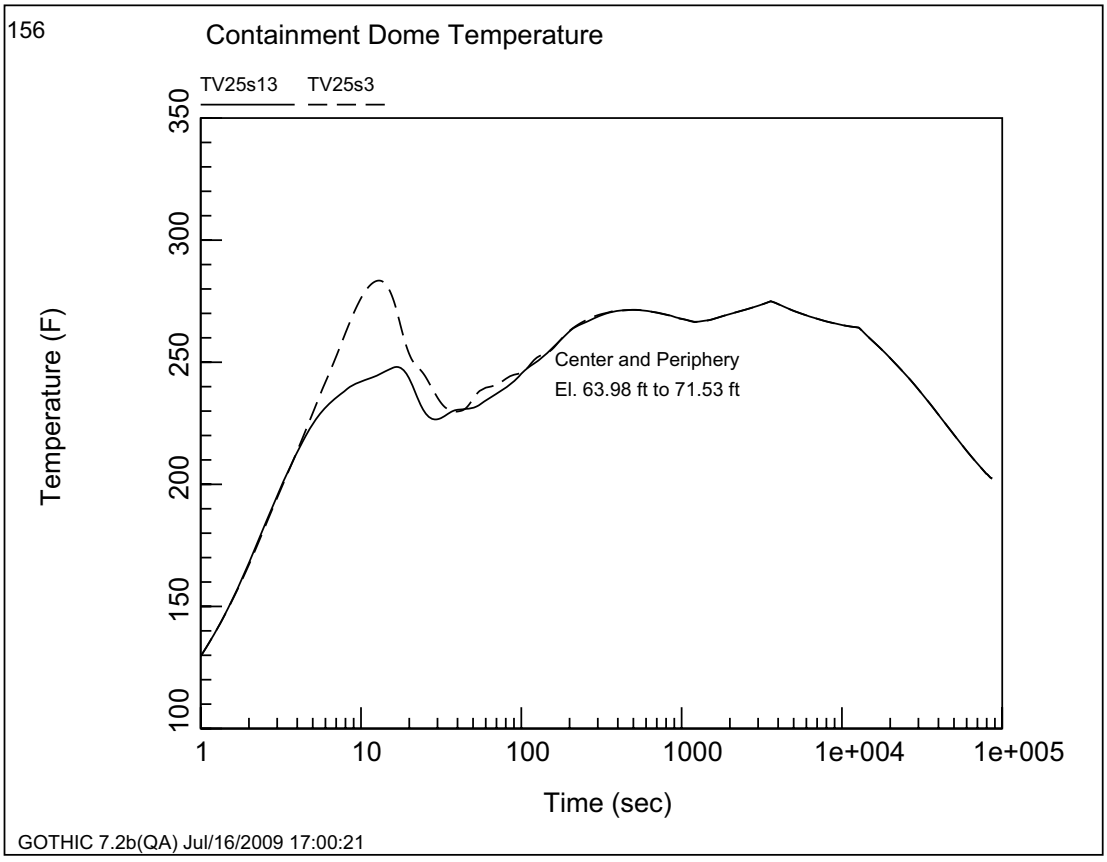


Figure 6.2.1-07d-55—Temperature in the Containment Dome between the 79.07 ft and 86.29 ft Elevations

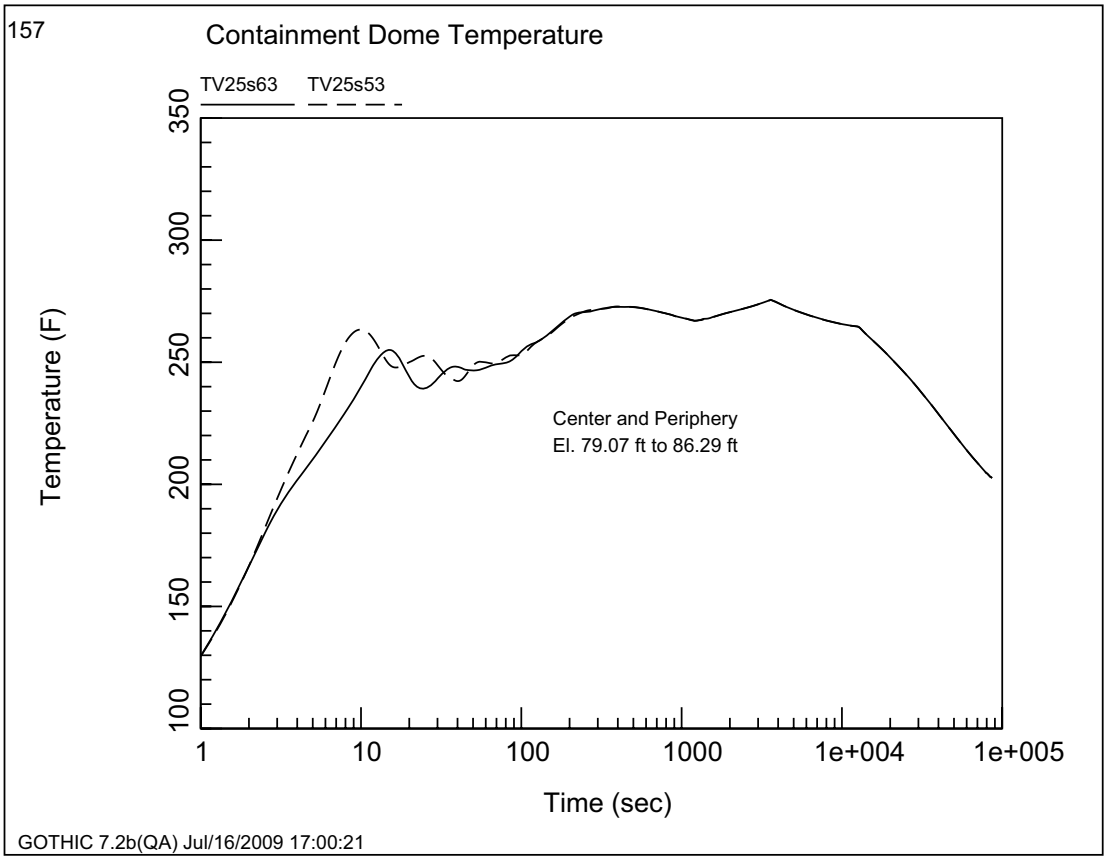


Figure 6.2.1-07d-56—Temperature in the Containment Dome between the 93.51 ft and 98.59 ft Elevations

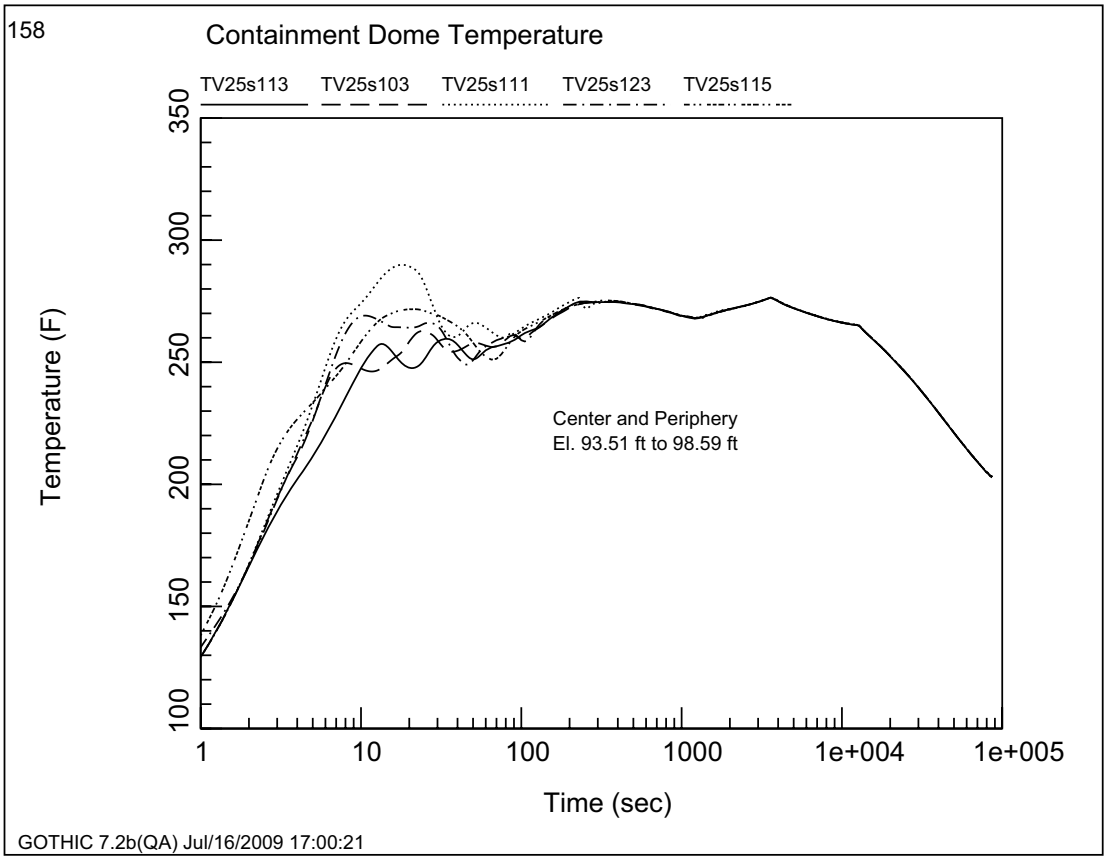


Figure 6.2.1-07d-57—Temperature in the Containment Dome between the 103.68 ft and 108.76 ft Elevations

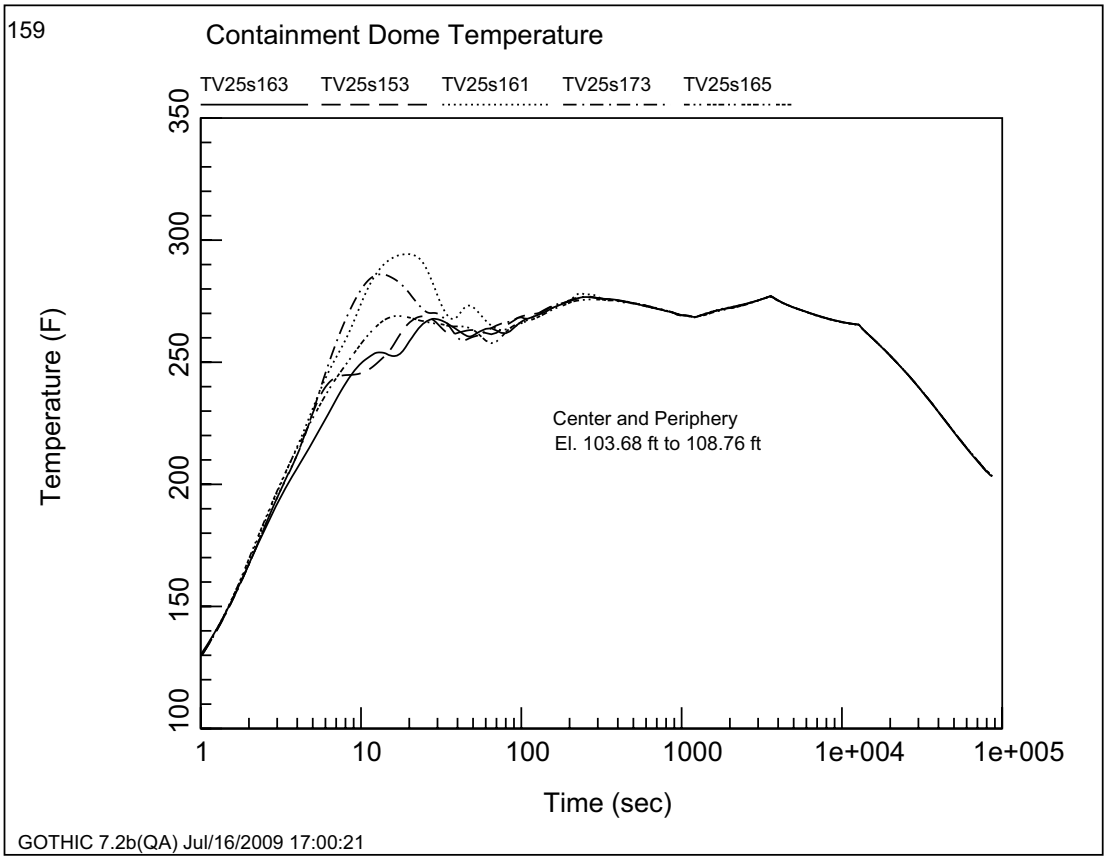


Figure 6.2.1-07d-58—Temperature in the Containment Dome between the 113.85 ft and 119.90 ft Elevations

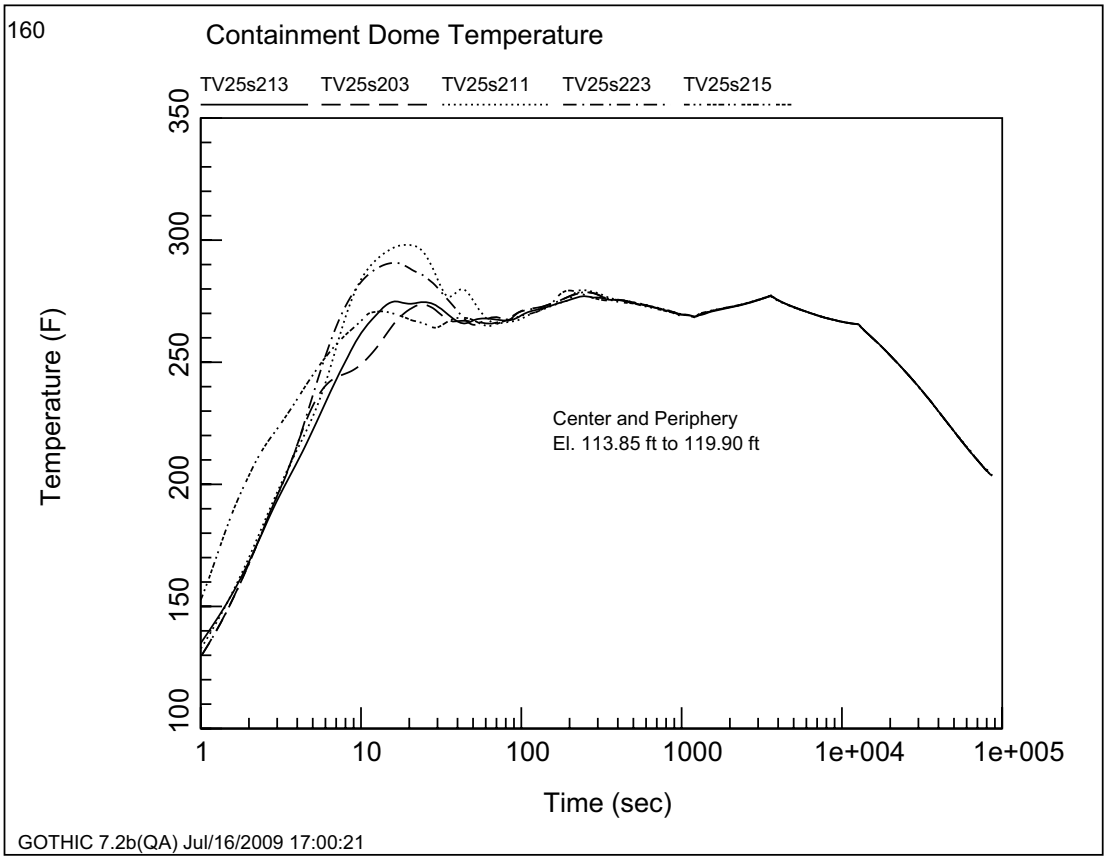


Figure 6.2.1-07d-59—Temperature in the Containment Dome between the 125.95 ft and 132.00 ft Elevations

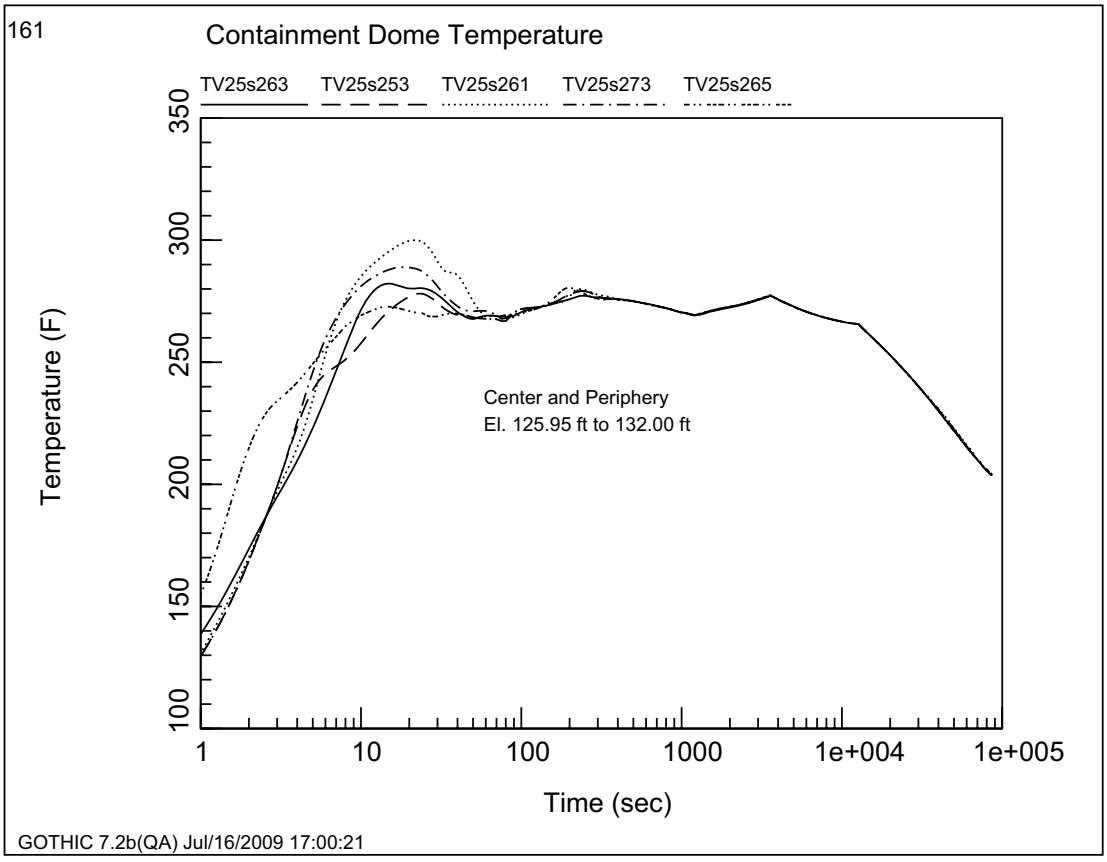


Figure 6.2.1-07d-60—Temperature in the Containment Dome between the 138.05 ft and 144.10 ft Elevations

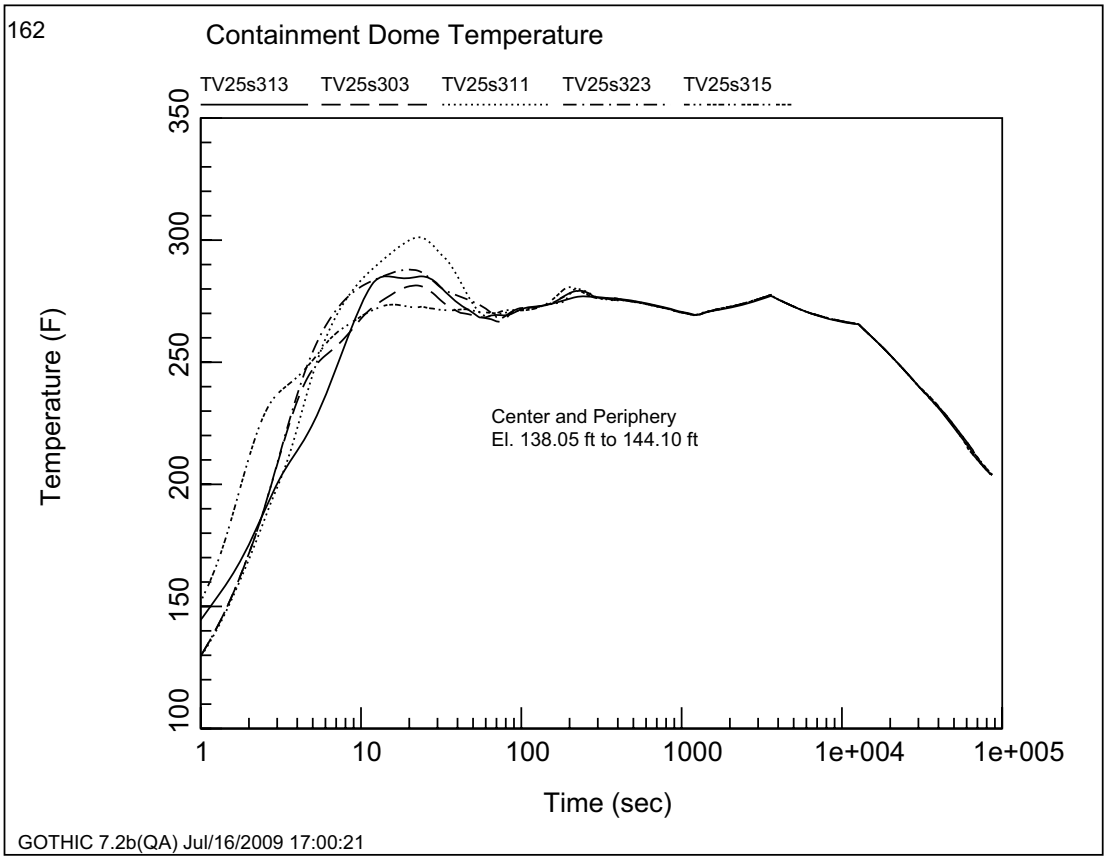


Figure 6.2.1-07d-61—Temperature in the Containment Dome between the 151.12 ft and 158.14 ft Elevations

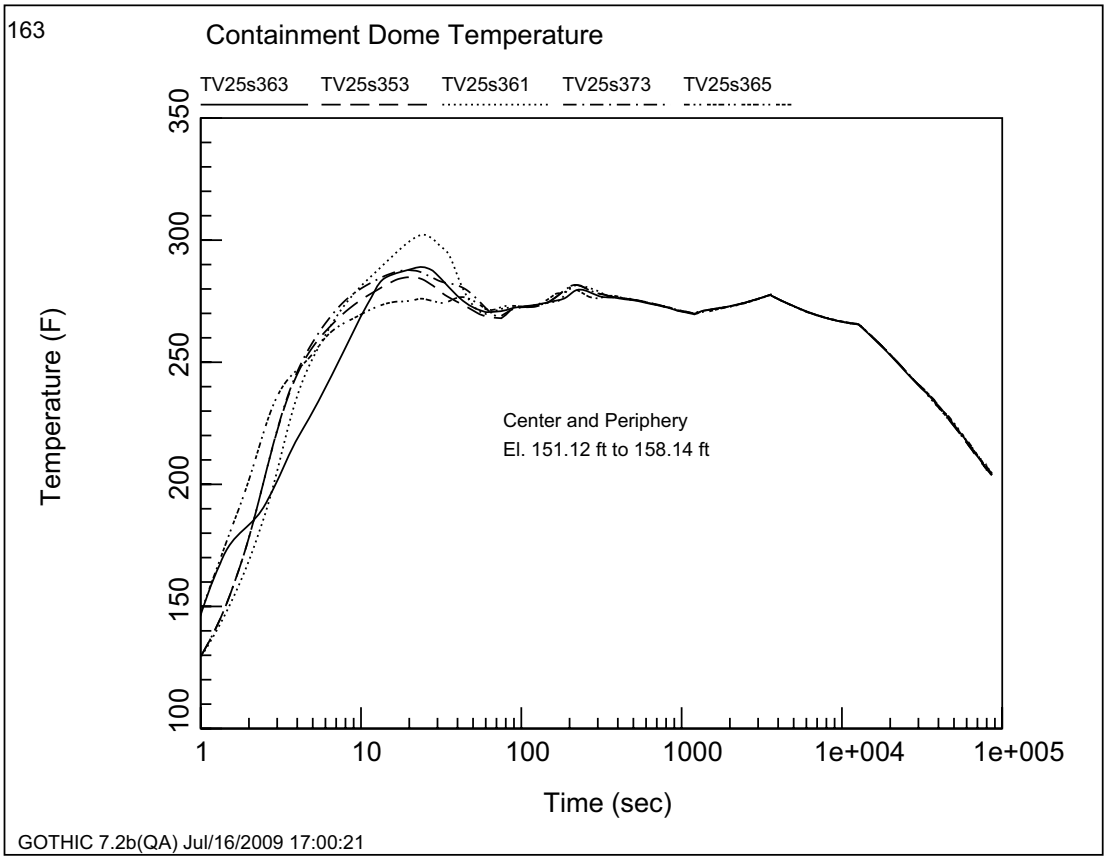


Figure 6.2.1-07d-62—Temperature in the Containment Dome between the 165.16 ft and 172.99 ft Elevations

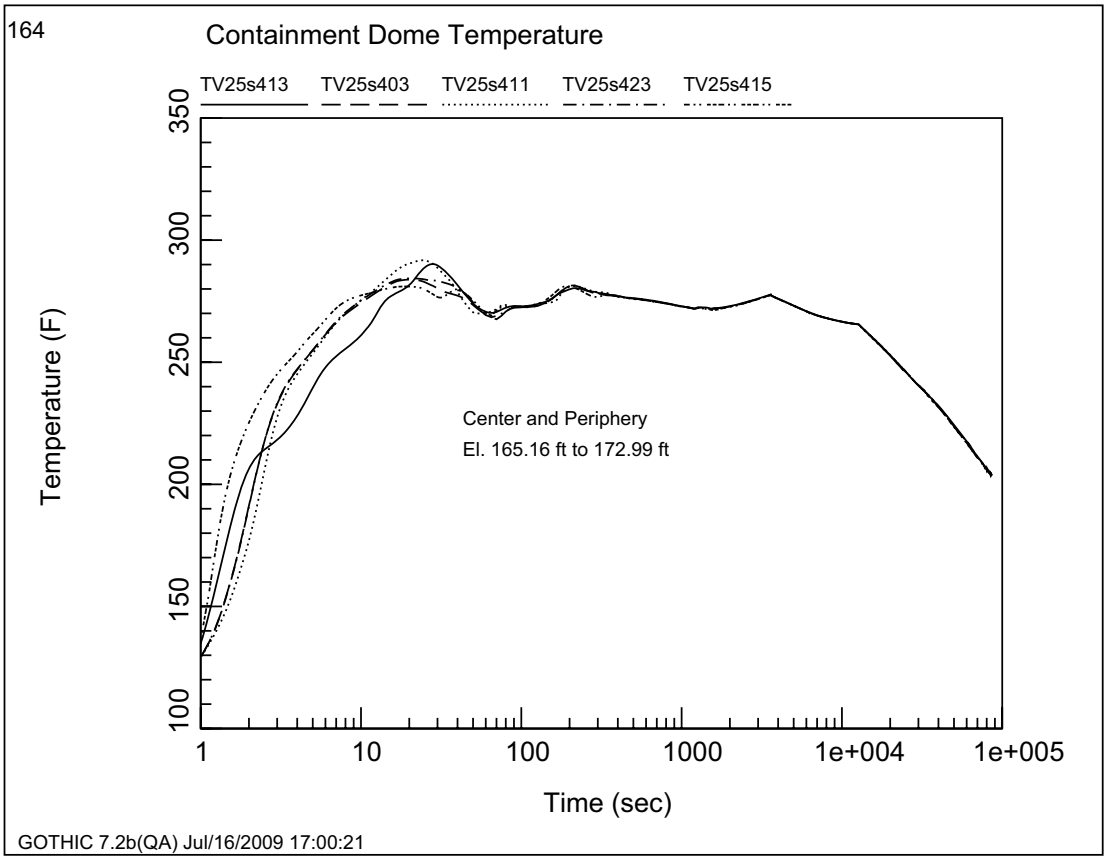
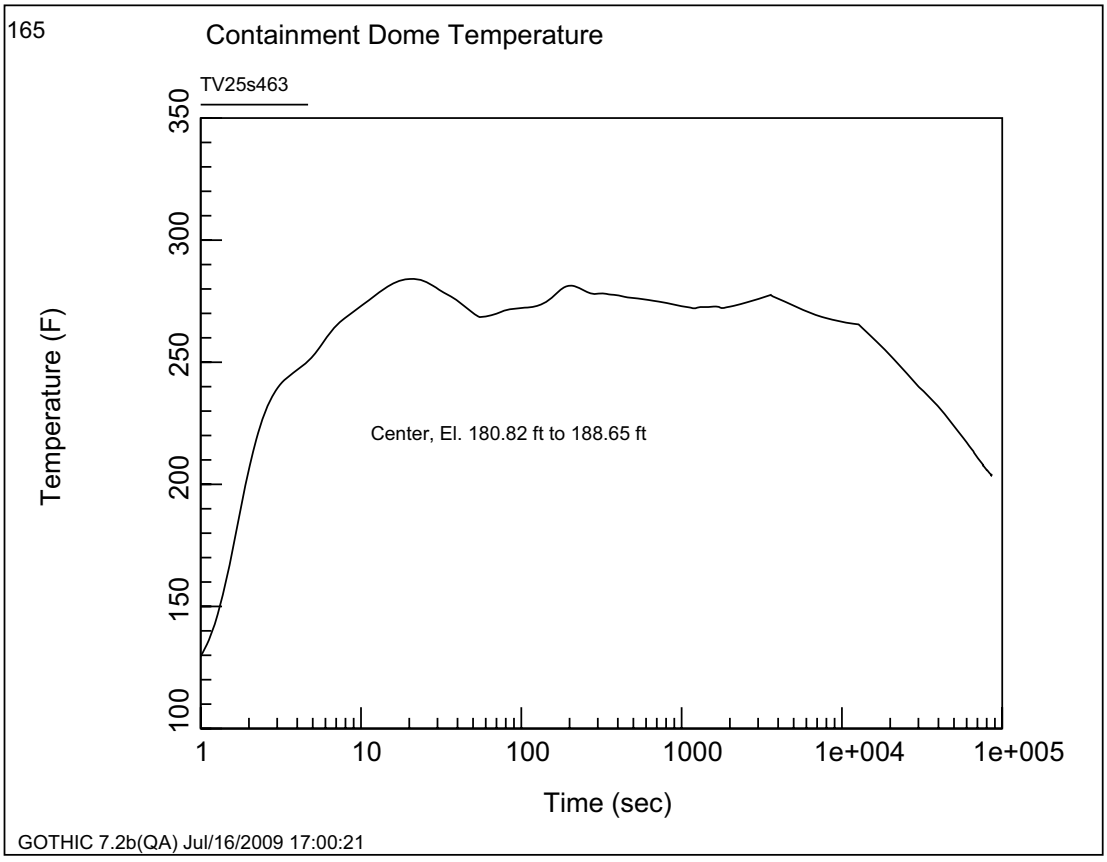


Figure 6.2.1-07d-63—Temperature in the Containment Dome between the 180.82 ft and 188.65 ft Elevations



Containment Internal Pressures

Equipment Area

Figure 6.2.1-07d-64—Pressure in the Spreading Room and IRWST

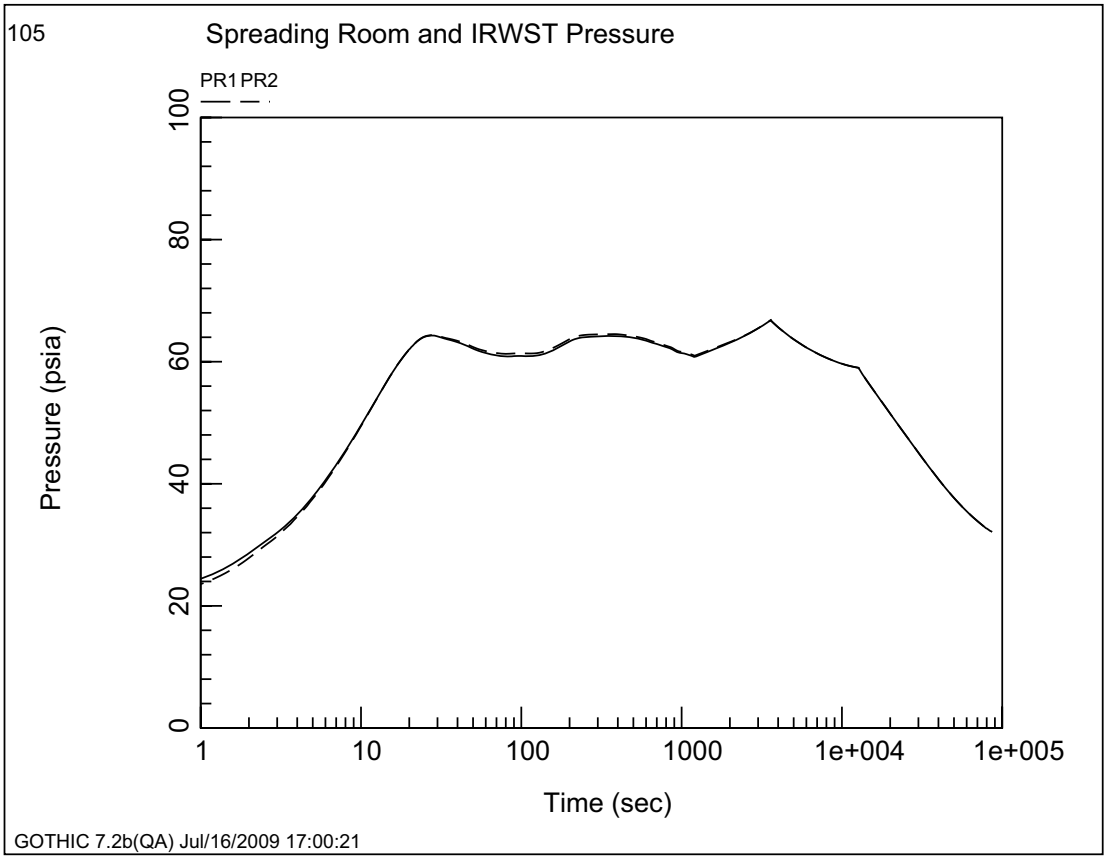


Figure 6.2.1-07d-65—Pressure in the Lower Equipment Rooms

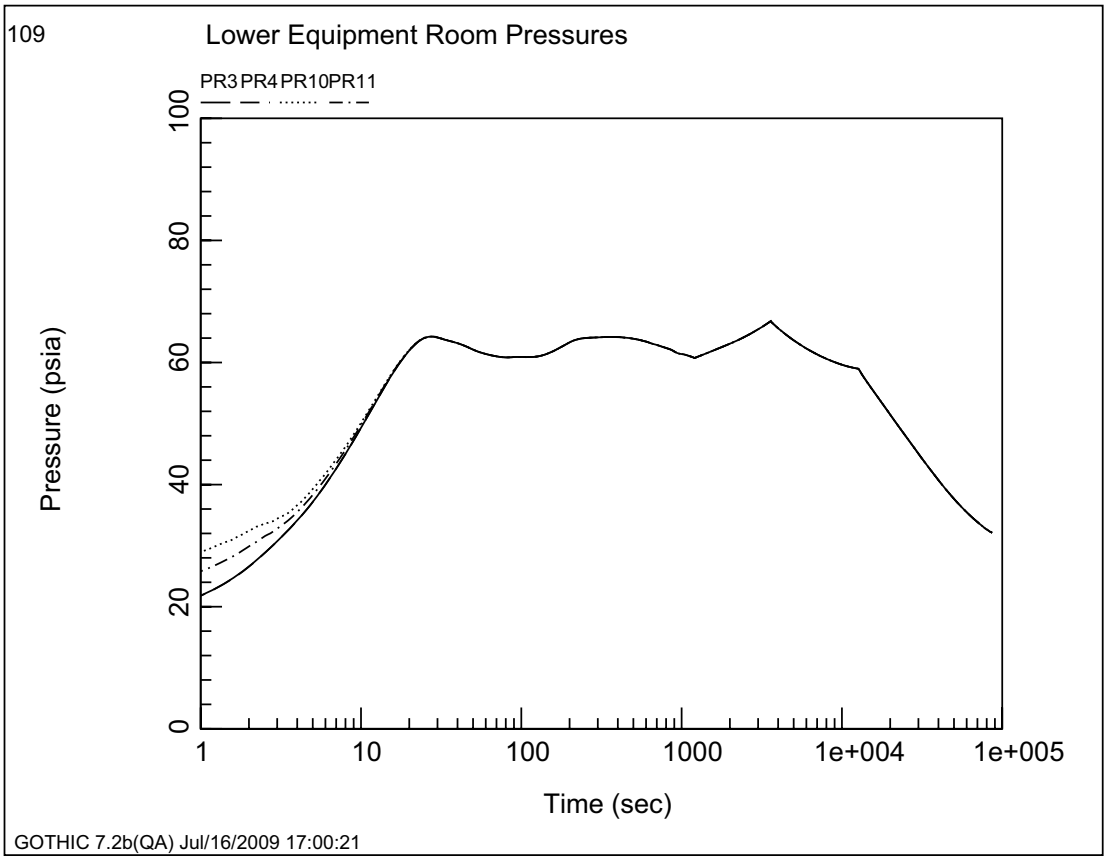


Figure 6.2.1-07d-66—Pressure in the Middle Equipment Rooms

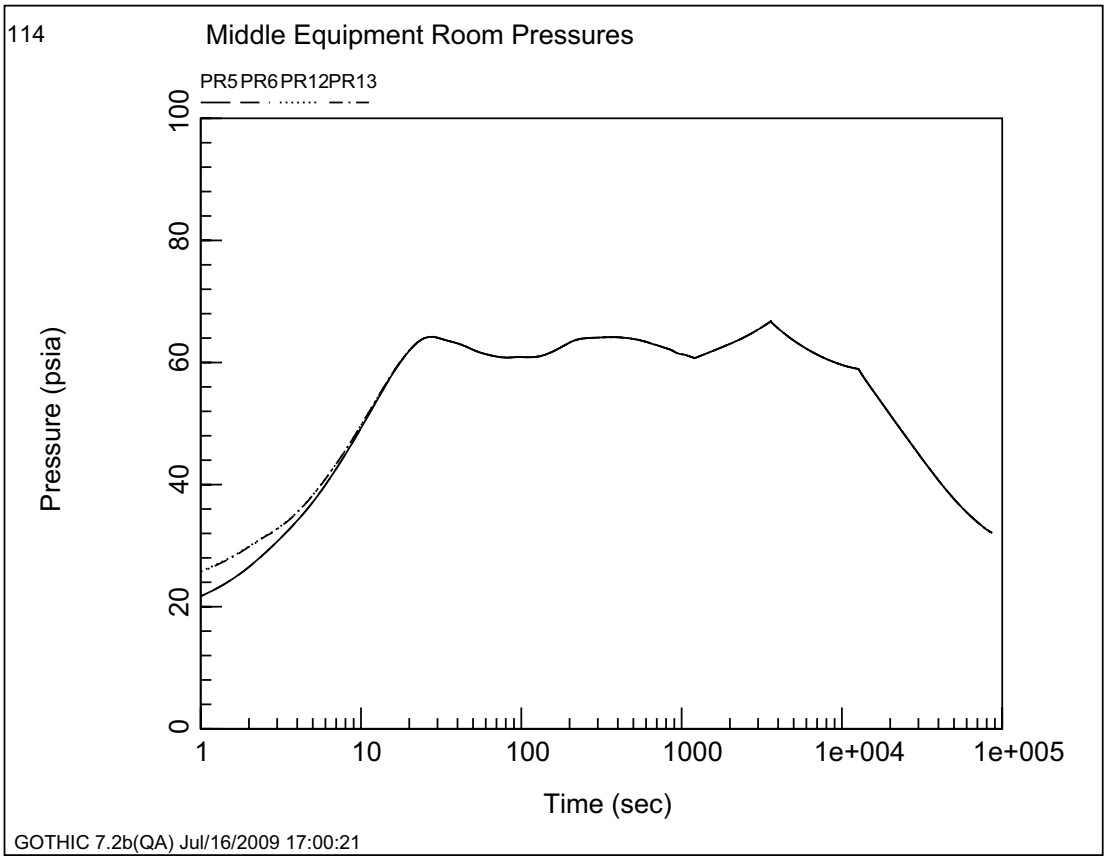


Figure 6.2.1-07d-67—Pressure in the Upper Equipment Rooms

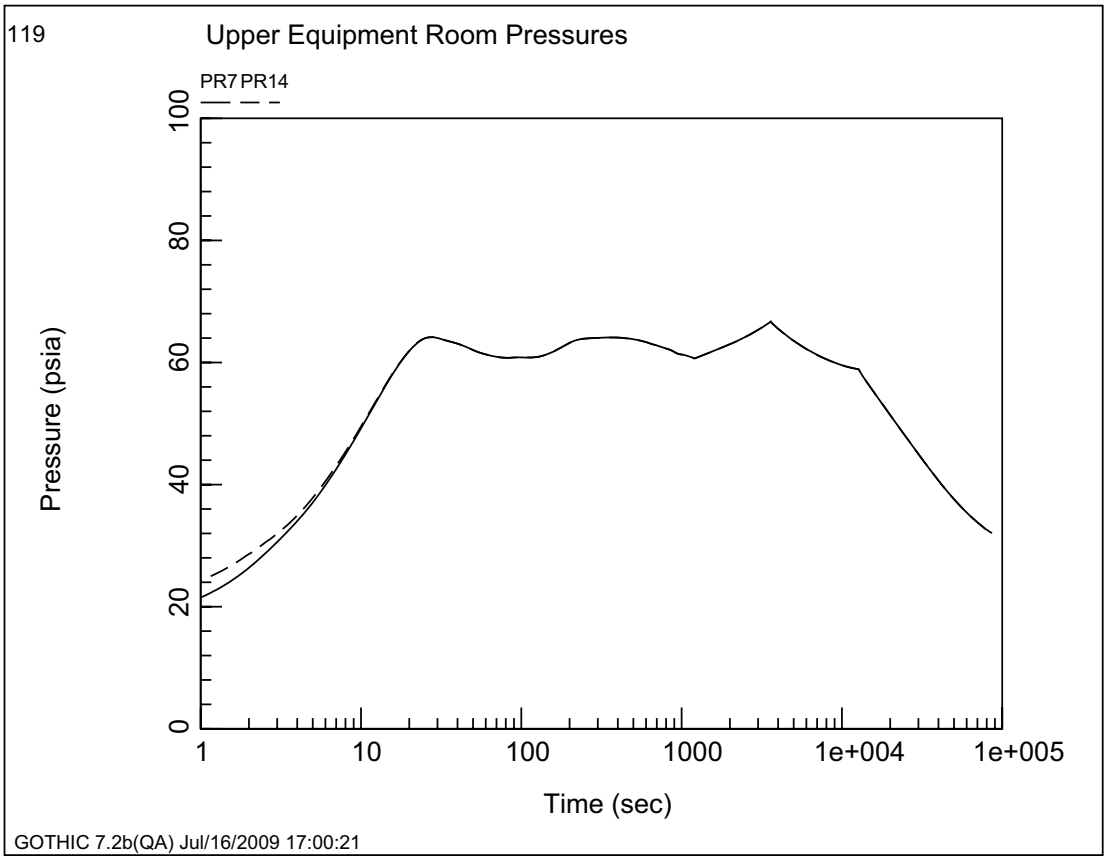


Figure 6.2.1-07d-68—Pressure in the Reactor Pressure Vessel Pit and Reactor Cavity

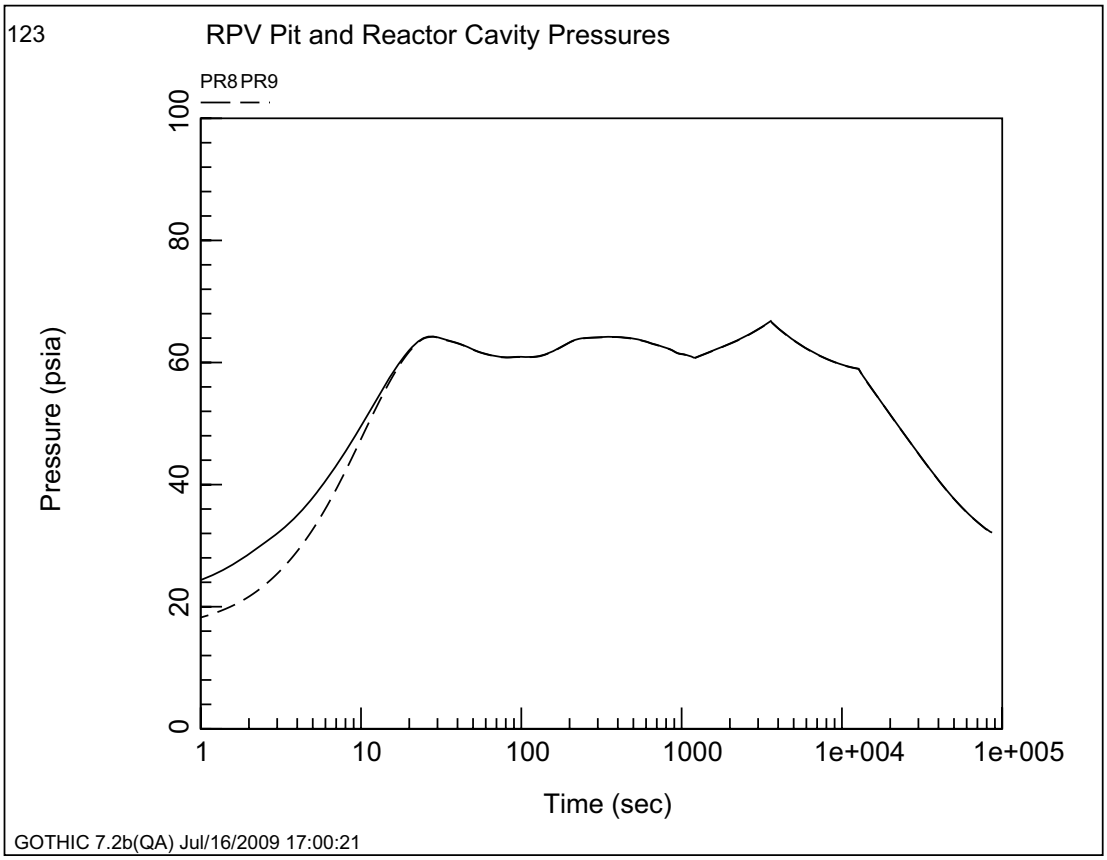
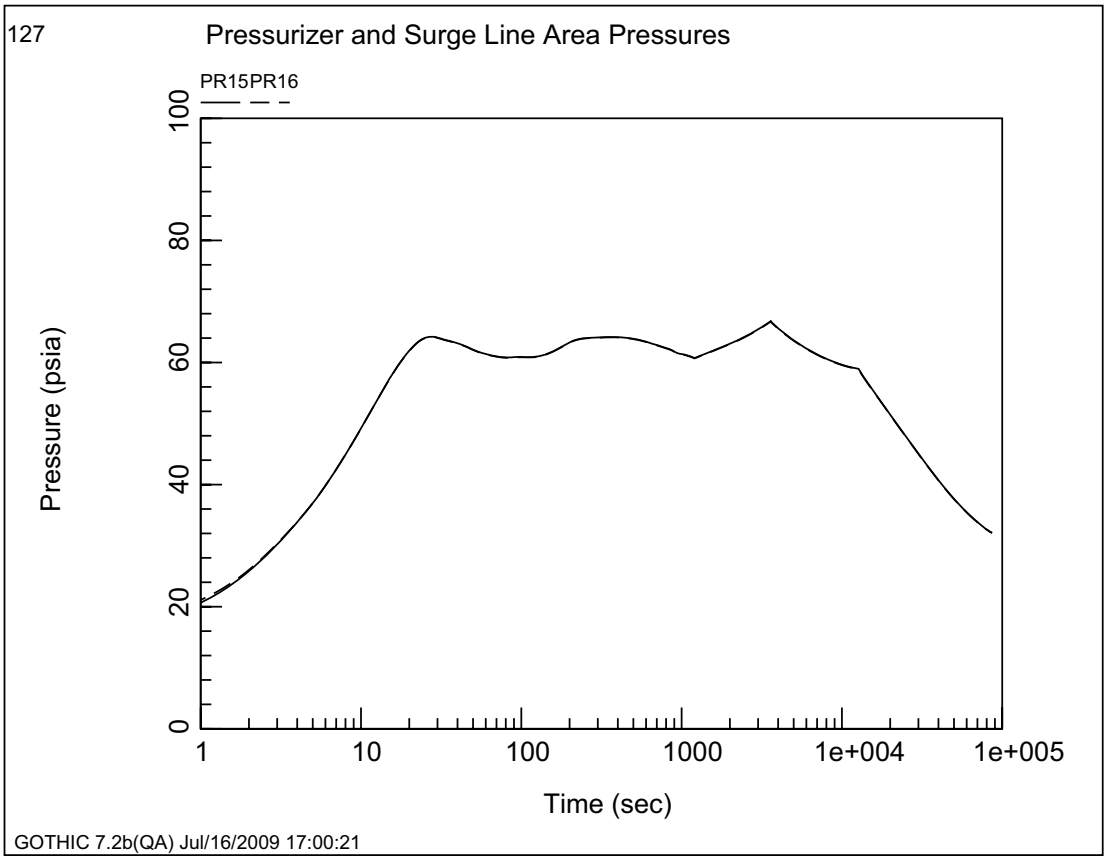


Figure 6.2.1-07d-69—Pressure in the Pressurizer and Surge Line Rooms



Accessible Area

Figure 6.2.1-07d-70—Pressure in the CVCS and Steam Generator Blowdown Heat Exchanger Rooms

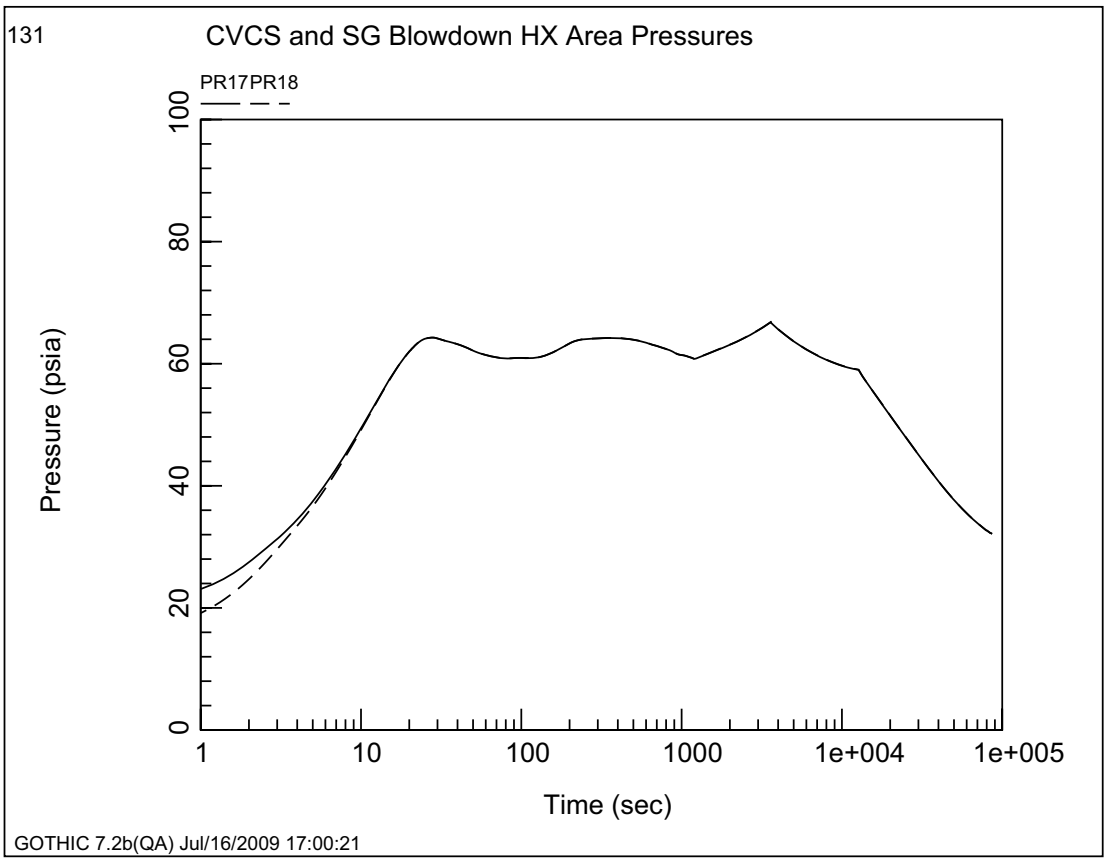


Figure 6.2.1-07d-71—Pressure in the Lower Annulus Rooms

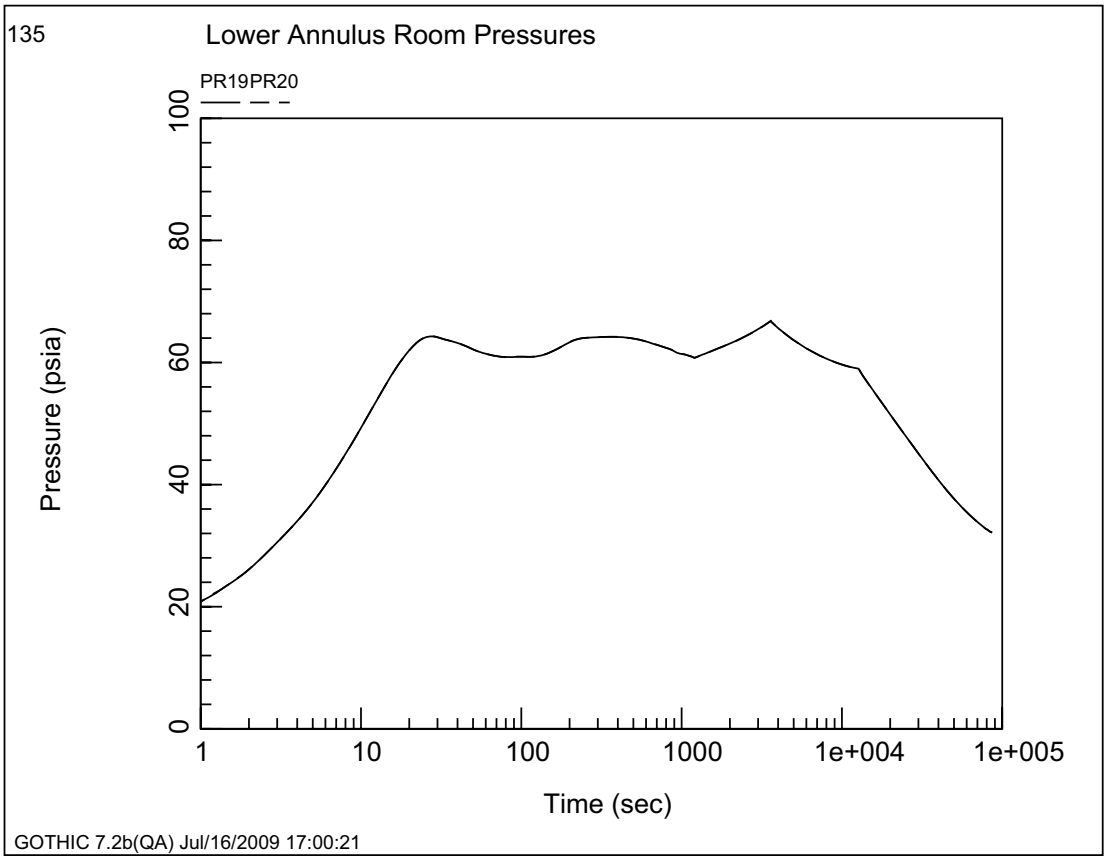


Figure 6.2.1-07d-72—Pressure in the Middle Annulus Rooms

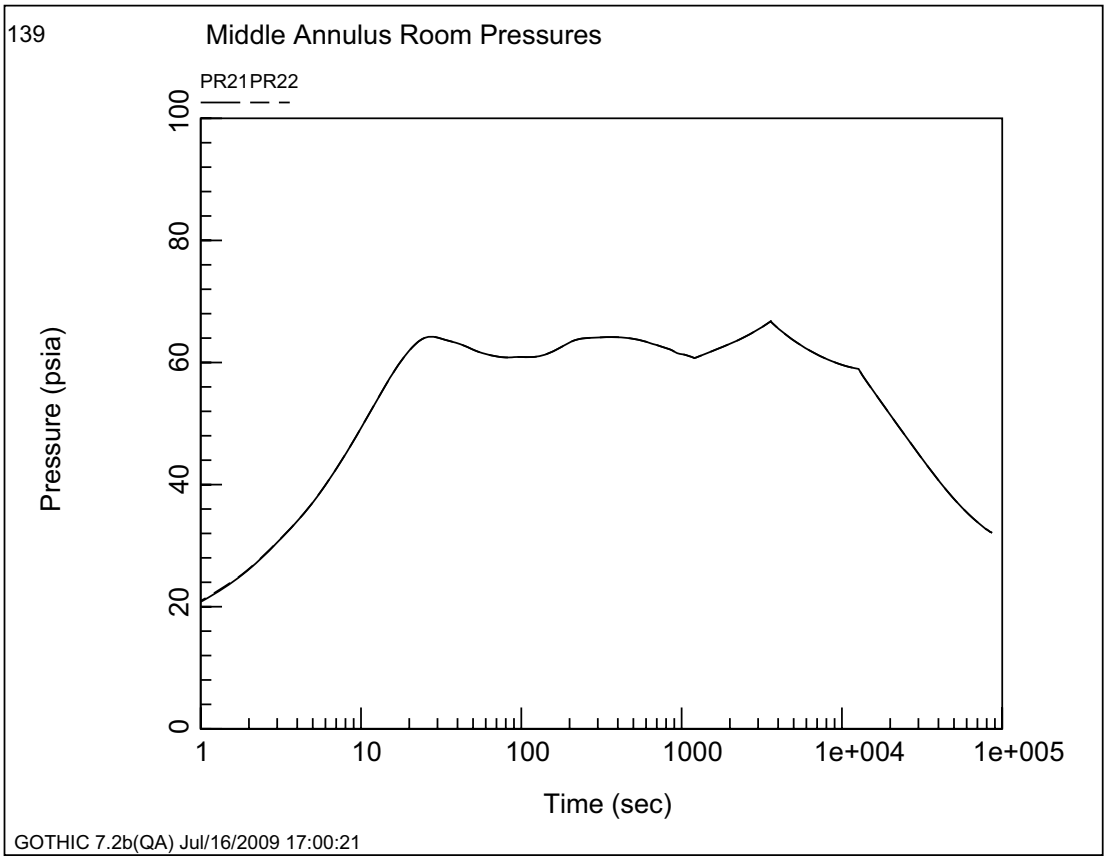


Figure 6.2.1-07d-73—Pressure in the Upper Annulus Rooms

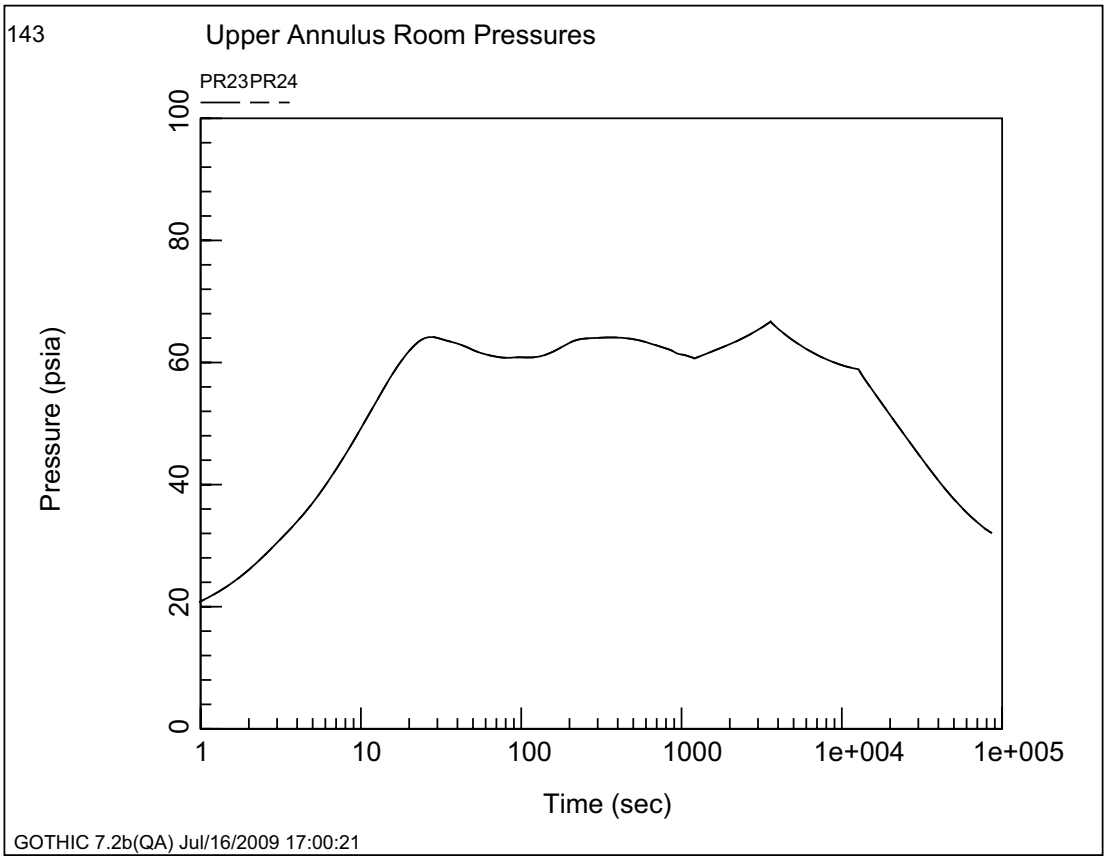


Figure 6.2.1-07d-74—Pressure in the Access and Hot Piping Penetration Rooms

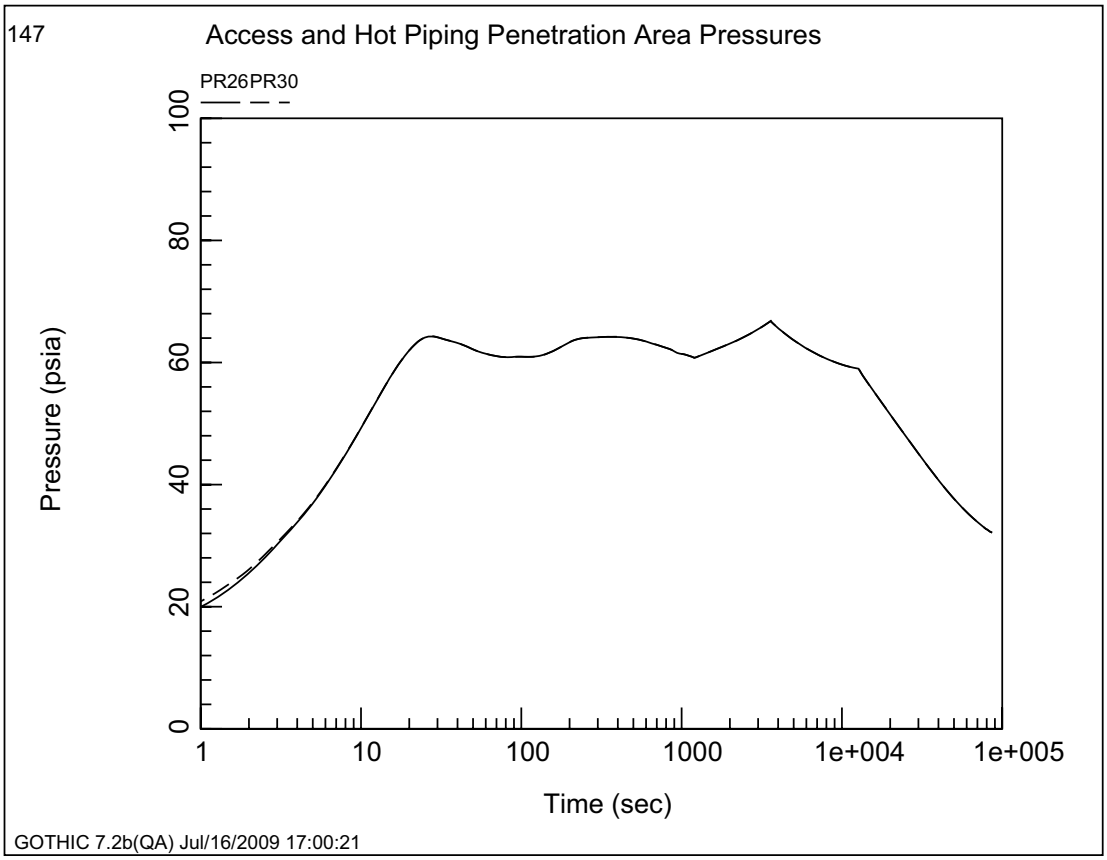
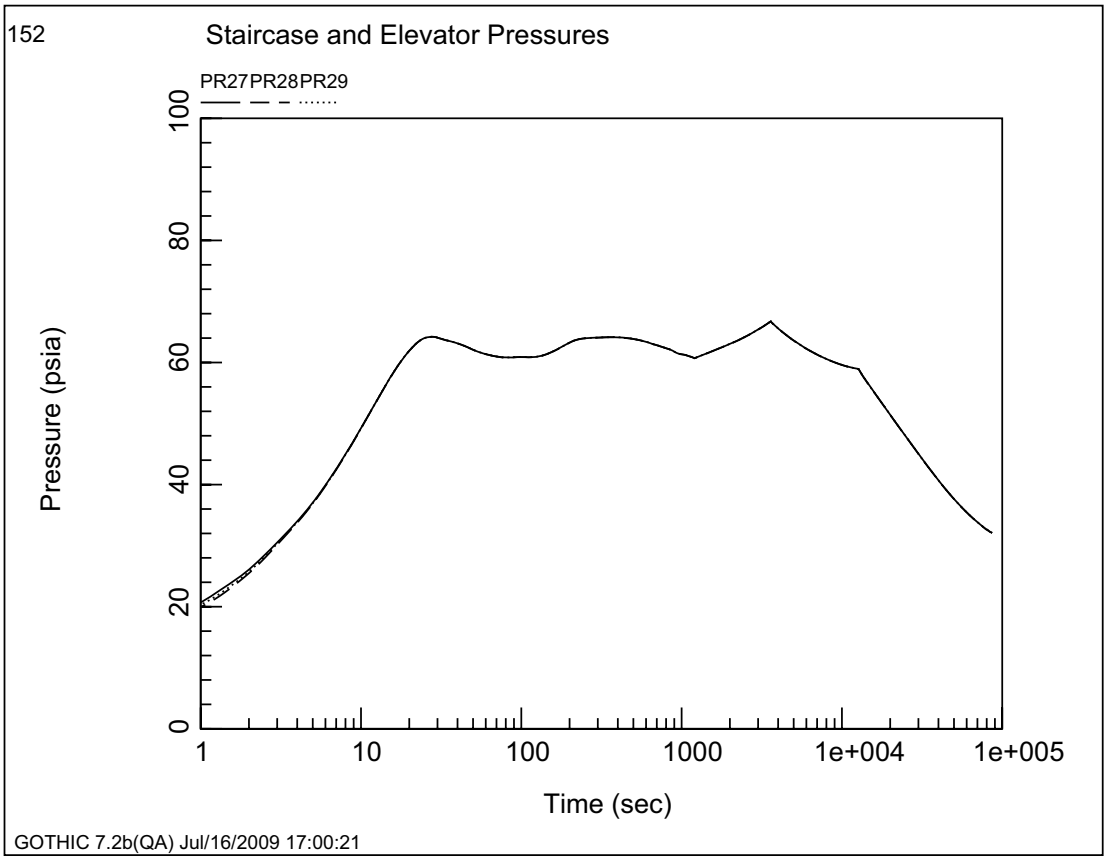


Figure 6.2.1-07d-75—Pressure in the North and South Staircases and in the Elevator



Containment Internal Liquid Volume Fractions

Equipment Area

Figure 6.2.1-07d-76—Liquid Volume Fraction in the Spreading Room and IRWST

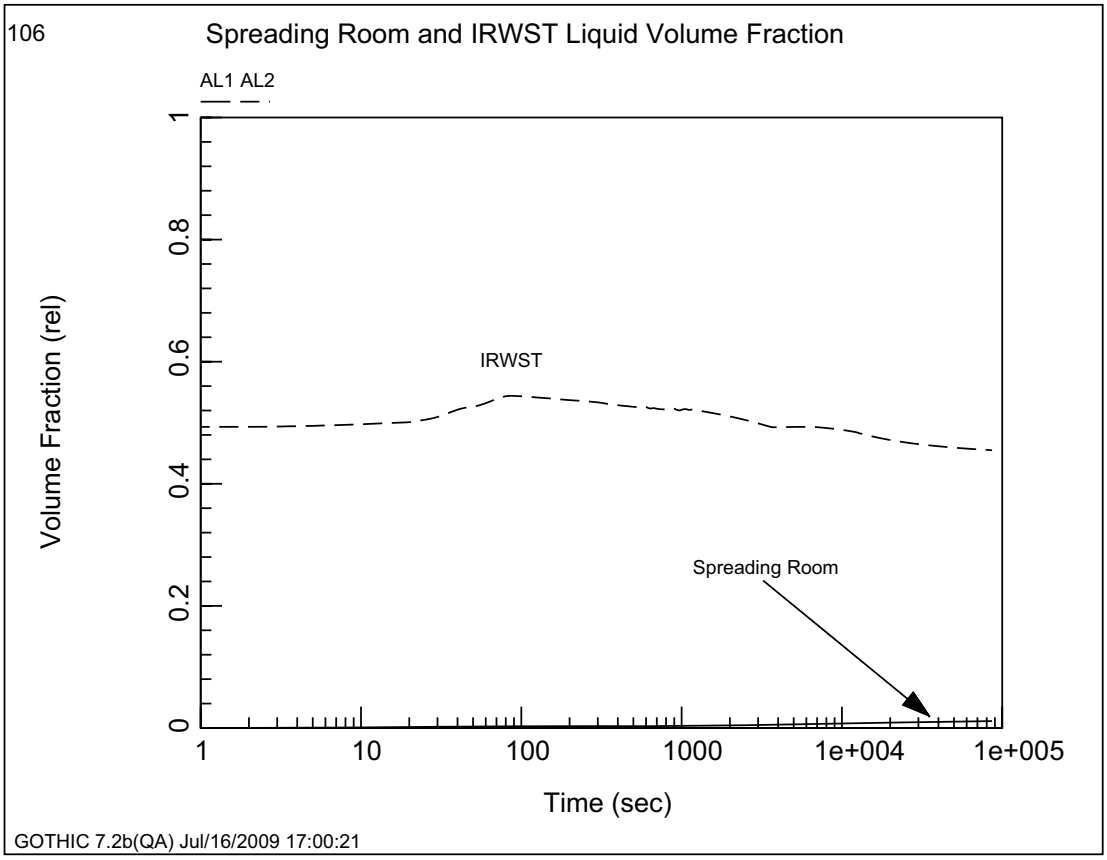


Figure 6.2.1-07d-77—Liquid Volume Fraction in the Lower Equipment Rooms

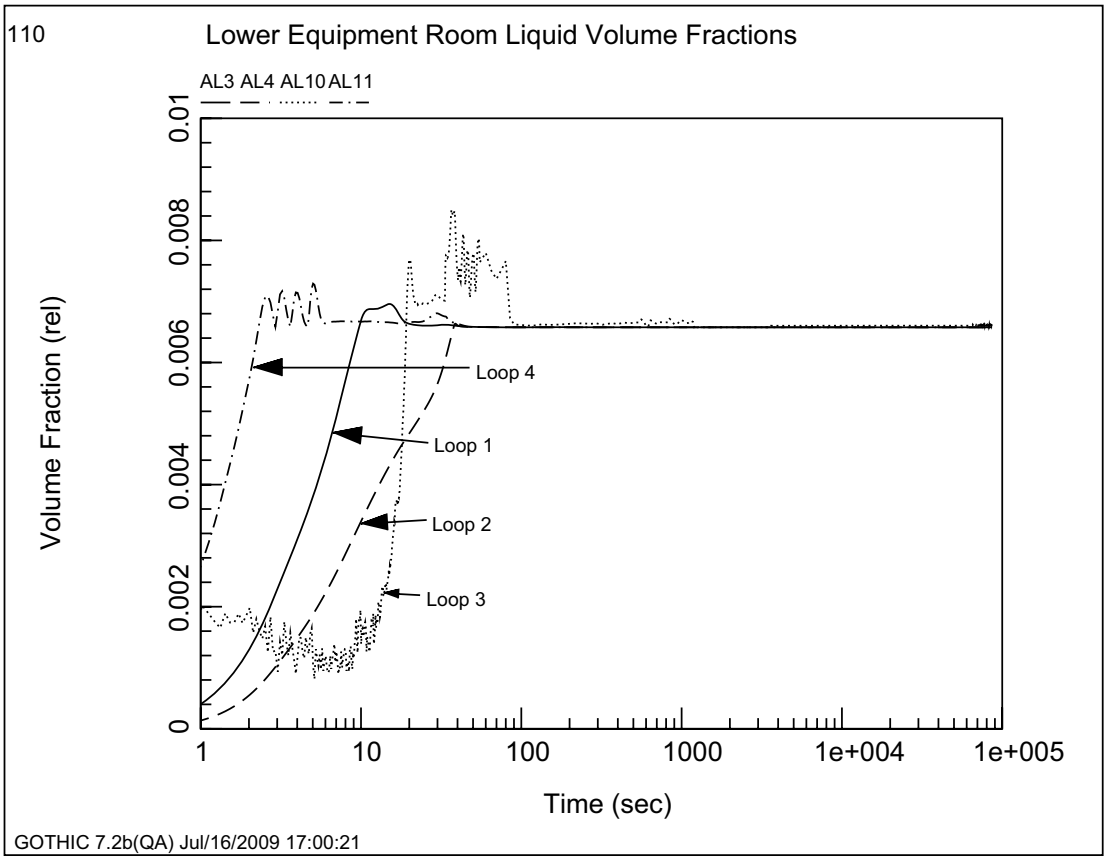


Figure 6.2.1-07d-78—Liquid Volume Fraction in the Middle Equipment Rooms

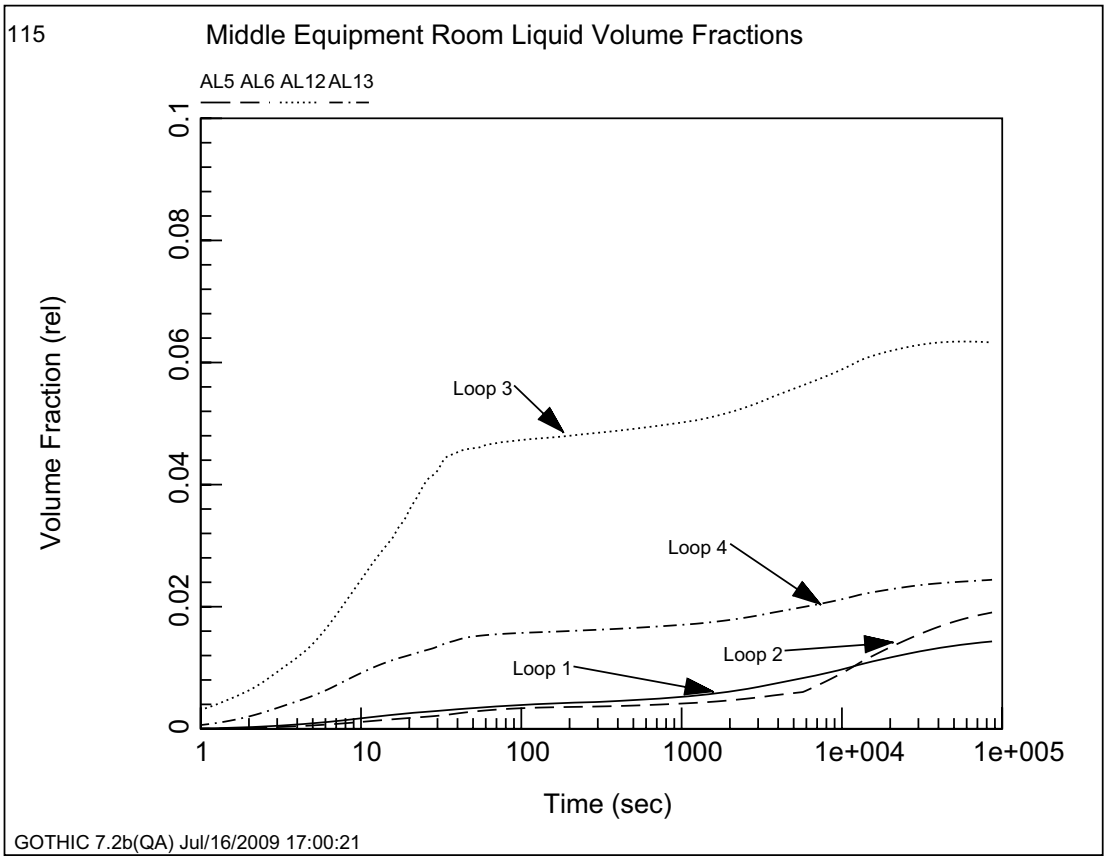


Figure 6.2.1-07d-79—Liquid Volume Fraction in the Upper Equipment Rooms

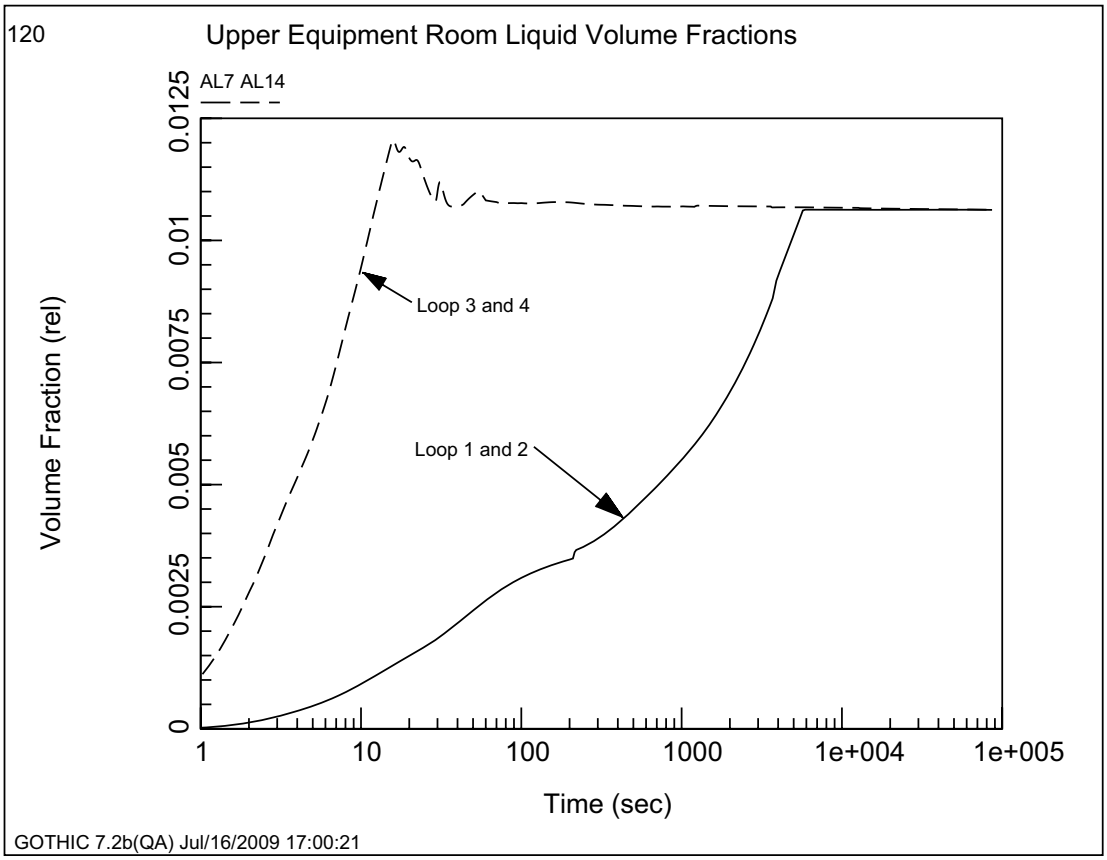


Figure 6.2.1-07d-80—Liquid Volume Fraction in the Reactor Pressure Vessel Pit and Reactor Cavity

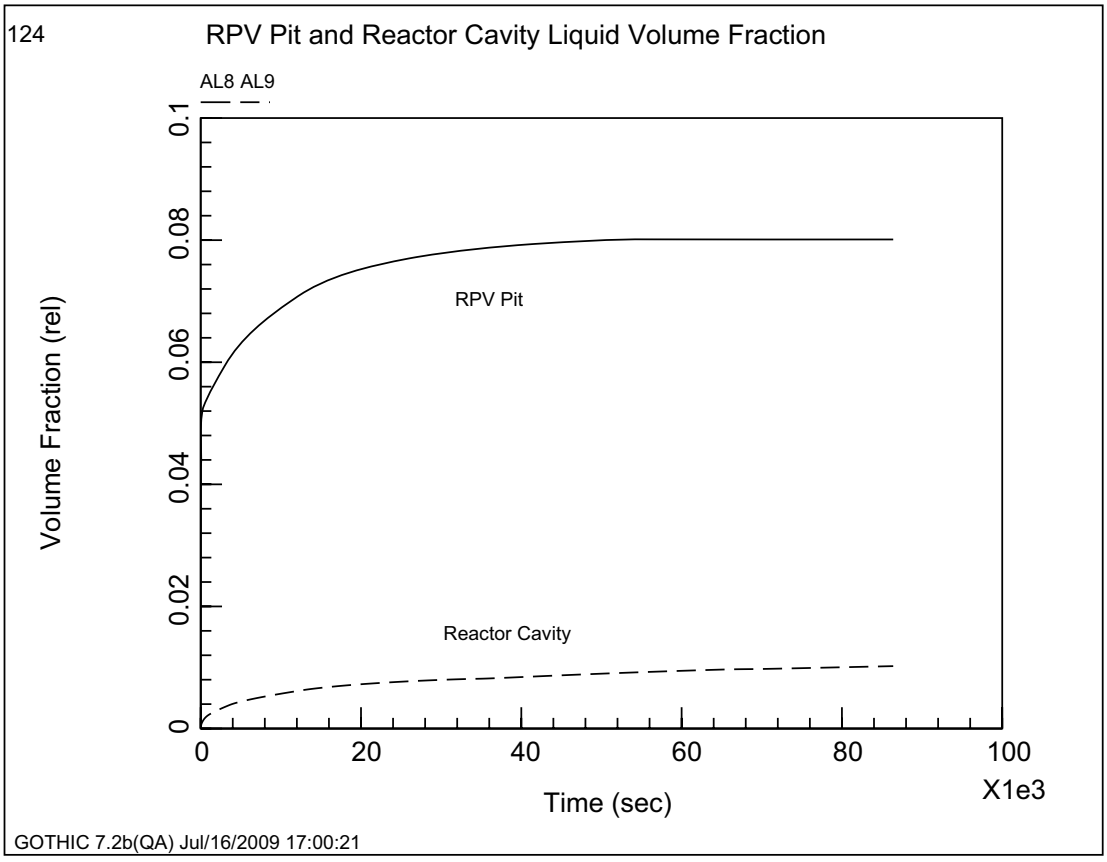
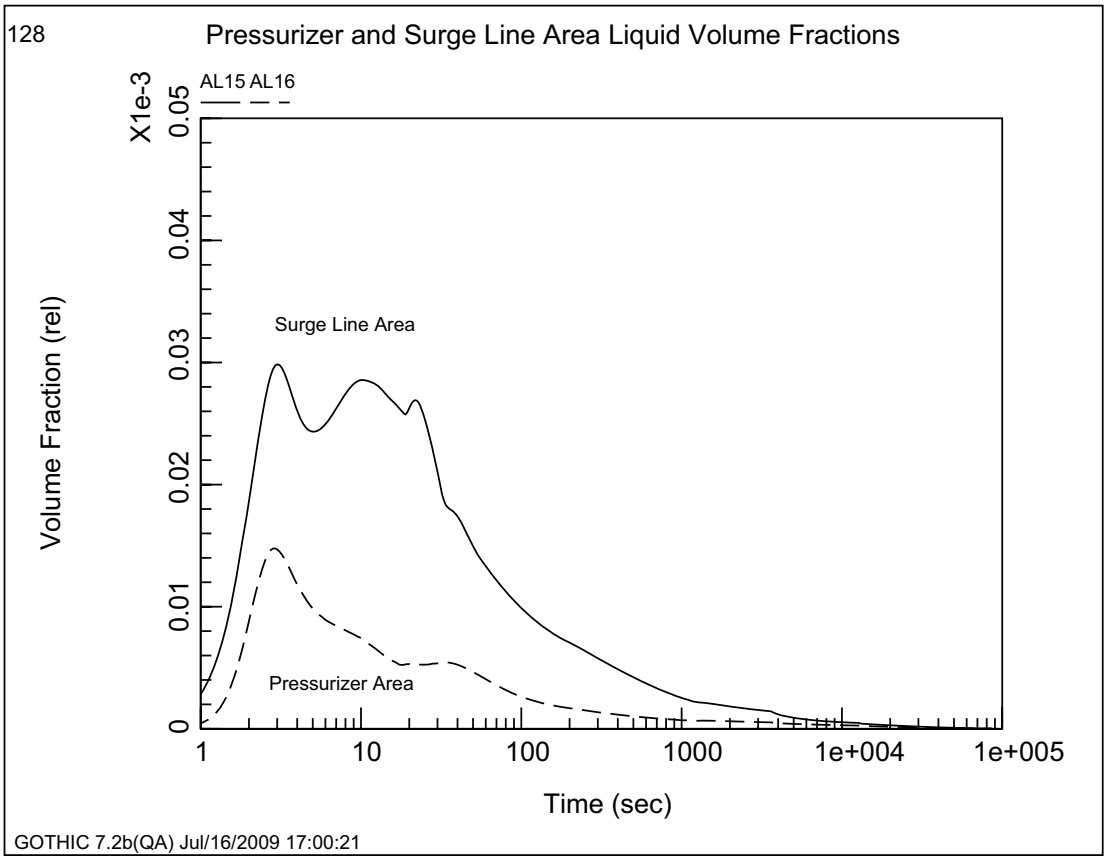


Figure 6.2.1-07d-81—Liquid Volume Fraction in the Pressurizer and Surge Line Rooms



Accessible Area

Figure 6.2.1-07d-82—Liquid Volume Fraction in the CVCS and Steam Generator Blowdown Heat Exchanger Rooms

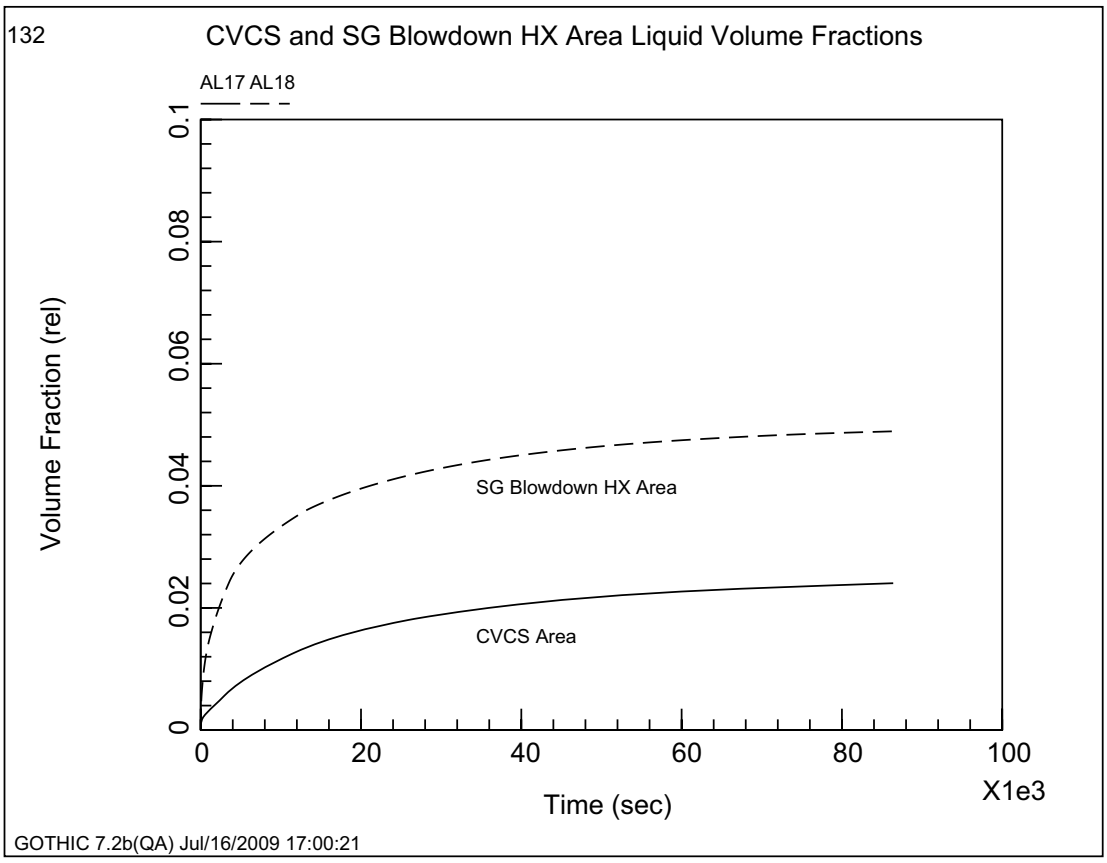


Figure 6.2.1-07d-83—Liquid Volume Fraction in the Lower Annulus Rooms

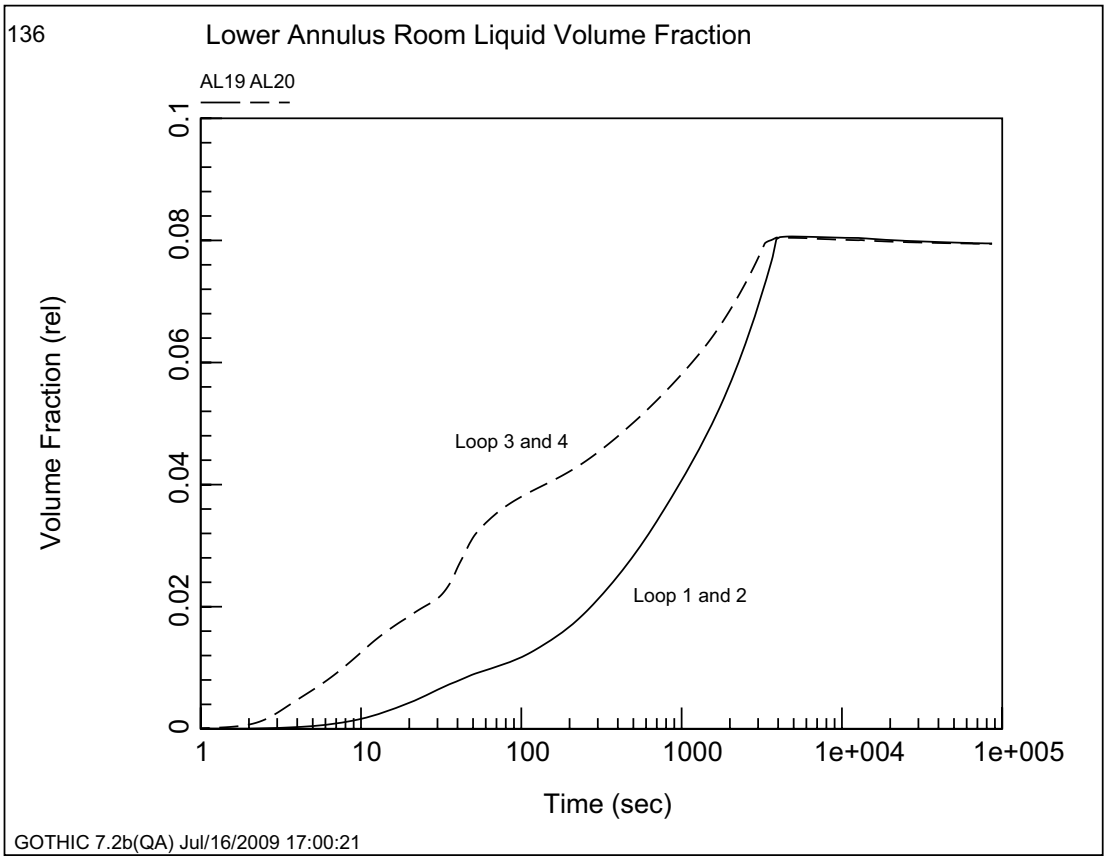


Figure 6.2.1-07d-84—Liquid Volume Fraction in the Middle Annulus Rooms

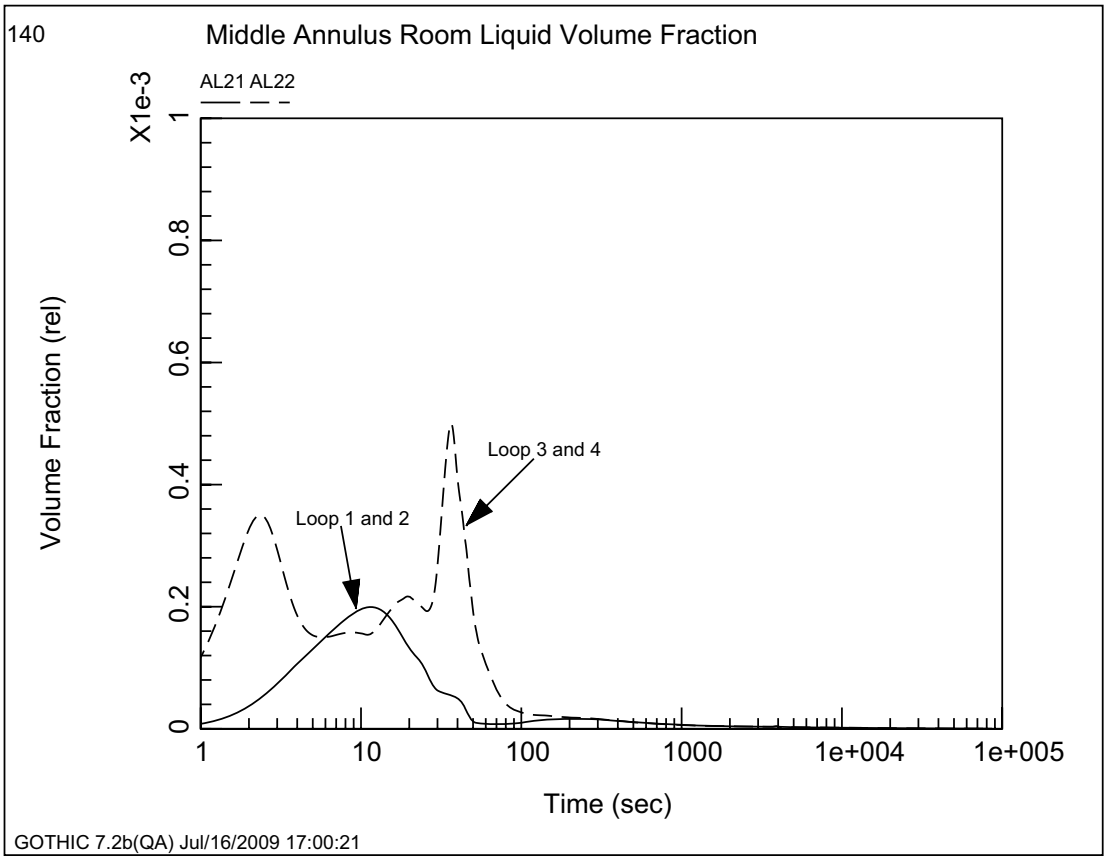


Figure 6.2.1-07d-85—Liquid Volume Fraction in the Upper Annulus Rooms

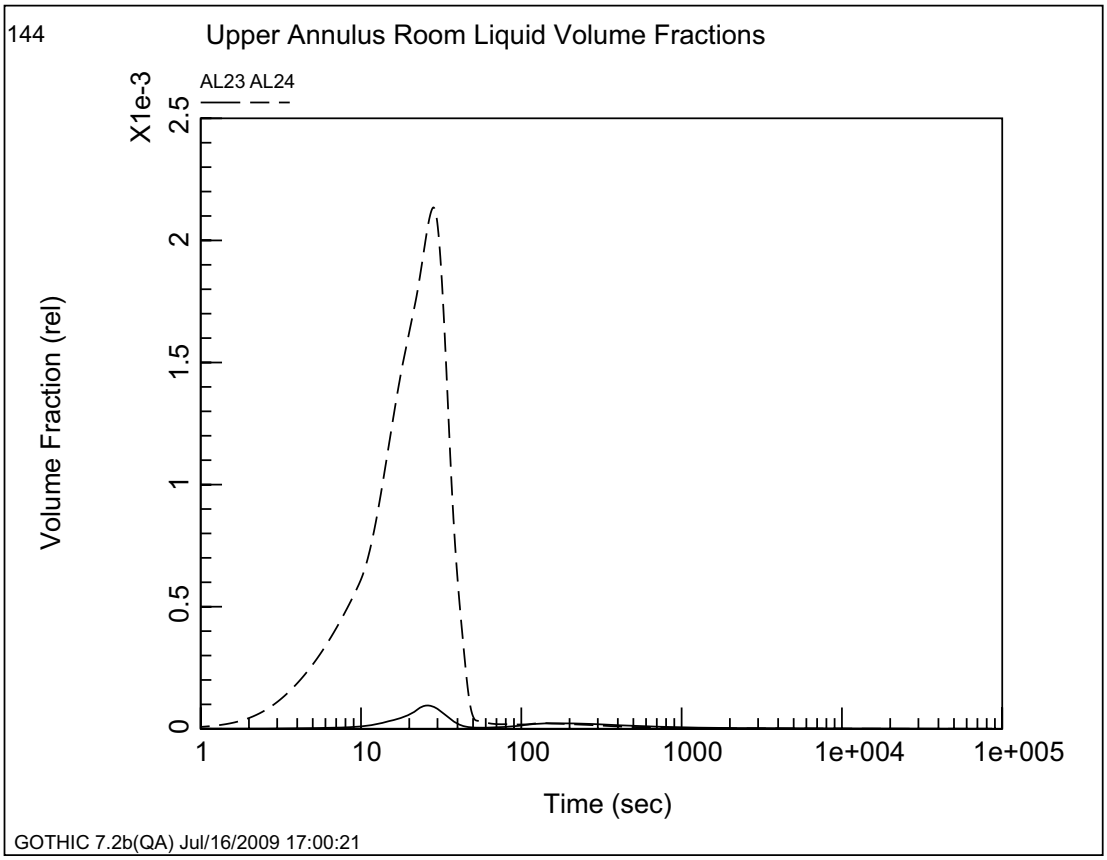


Figure 6.2.1-07d-86—Liquid Volume Fraction in the Access and Hot Piping Penetration Rooms

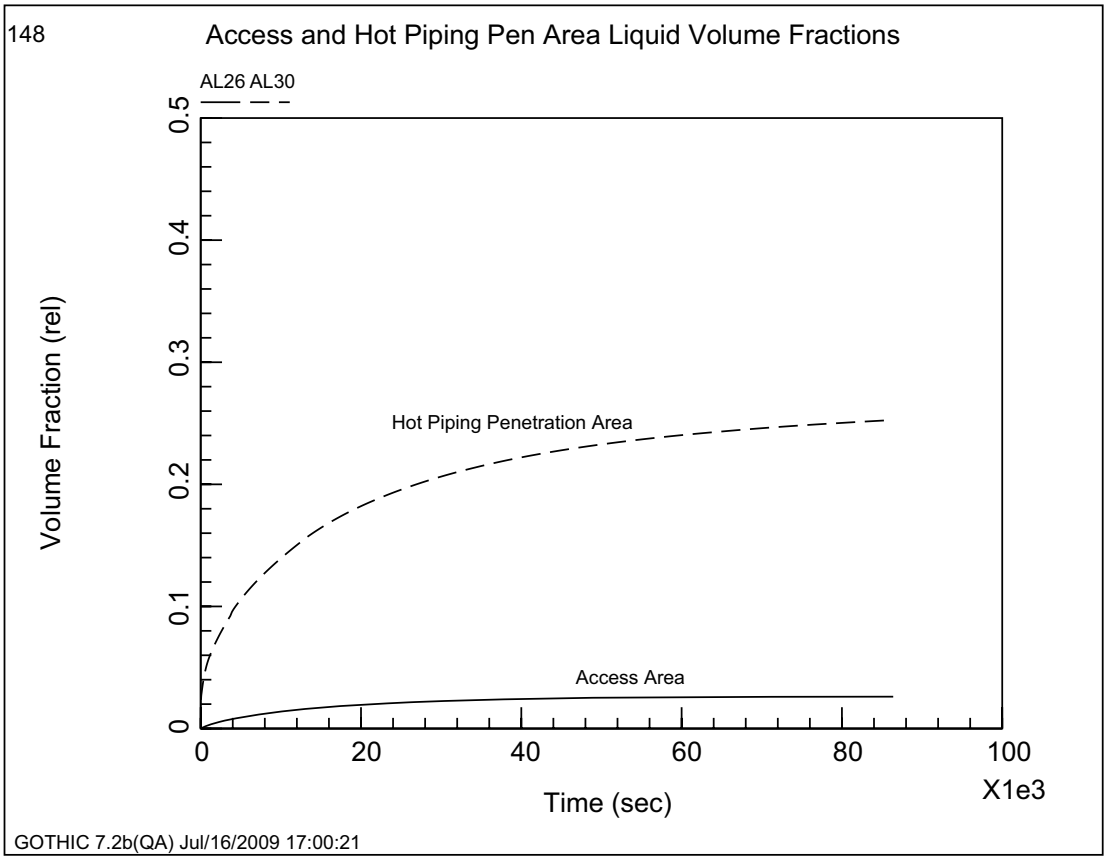
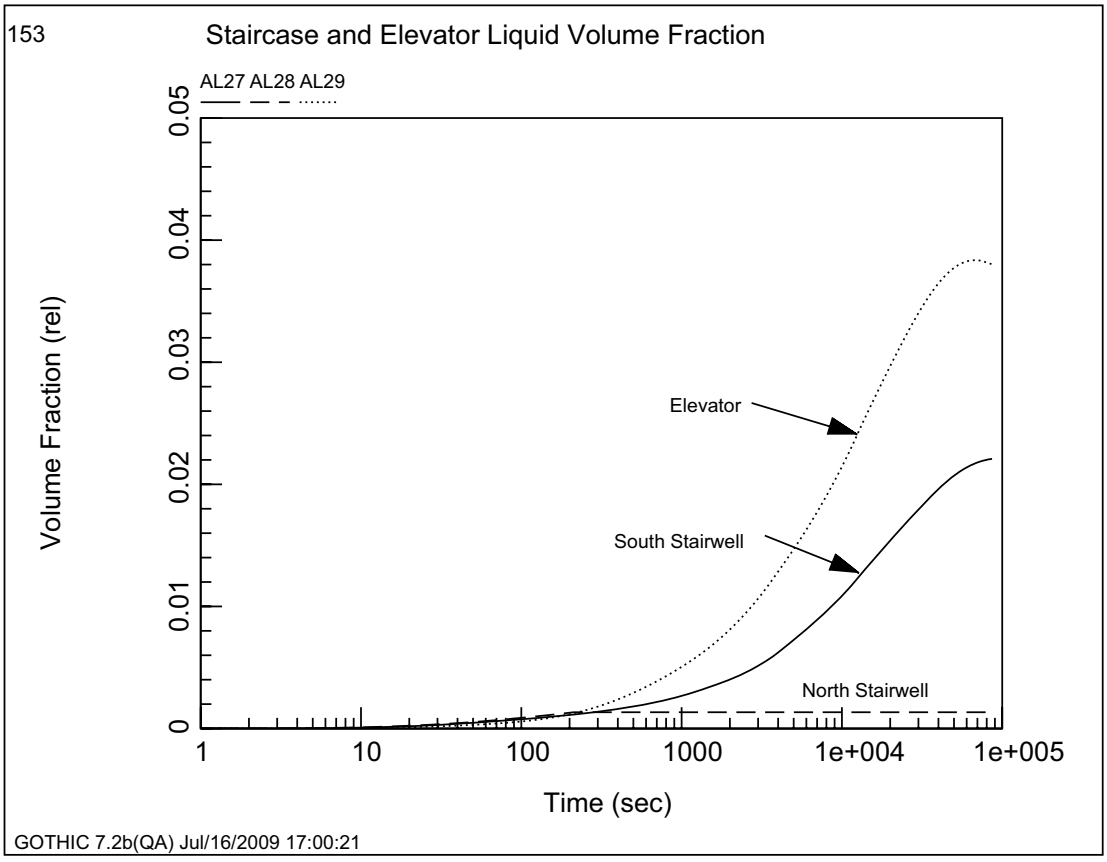


Figure 6.2.1-07d-87—Liquid Volume Fraction in the North and South Staircases and in the Elevator



Containment Internal Air and Steam Volume Fractions

Equipment Area

Figure 6.2.1-07d-88—Air and Steam Volume Fraction in the Spreading Room and IRWST

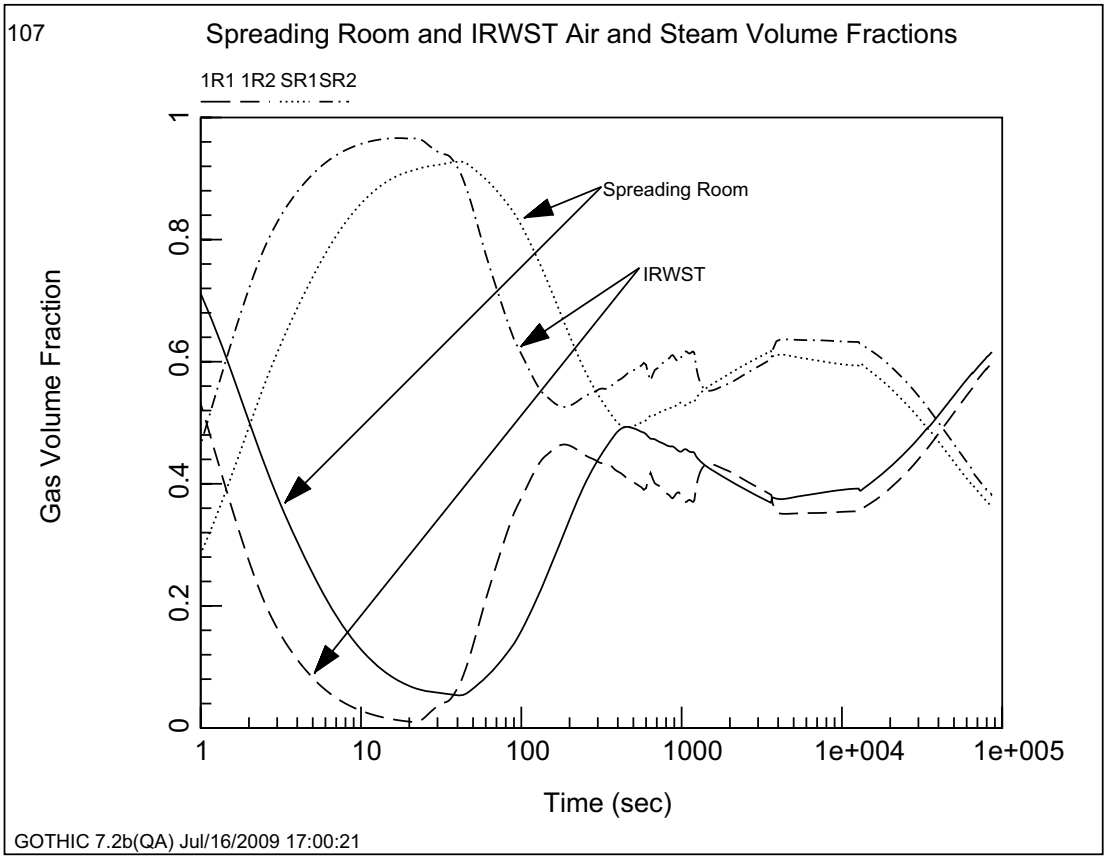


Figure 6.2.1-07d-89—Air Volume Fraction in the Lower Equipment Rooms

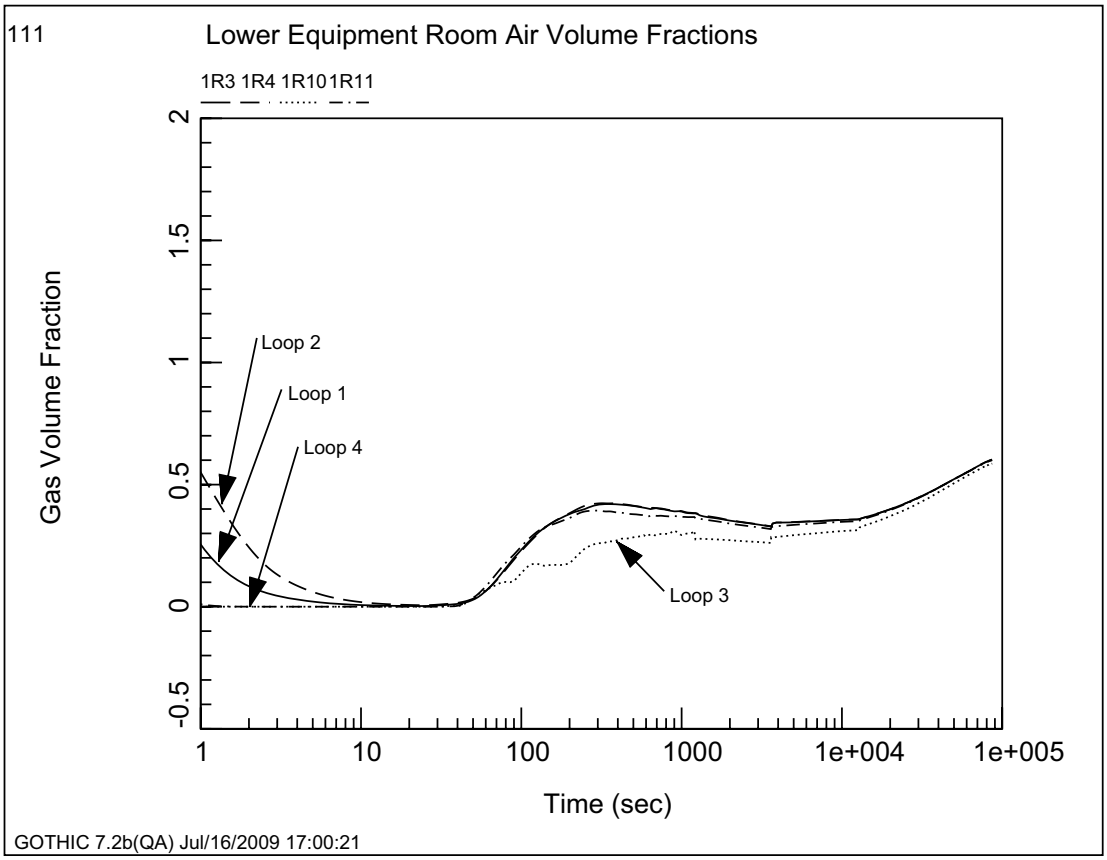


Figure 6.2.1-07d-90—Steam Volume Fraction in the Lower Equipment Rooms

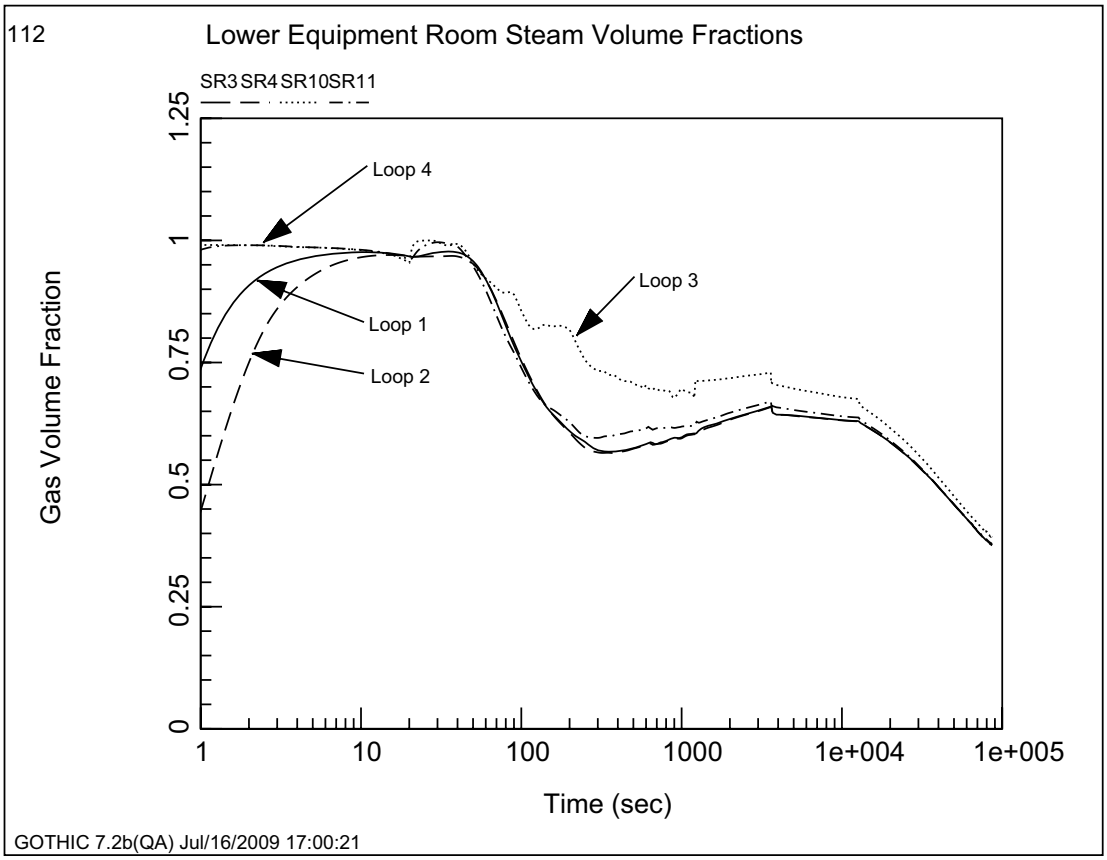


Figure 6.2.1-07d-91—Air Volume Fraction in the Middle Equipment Rooms

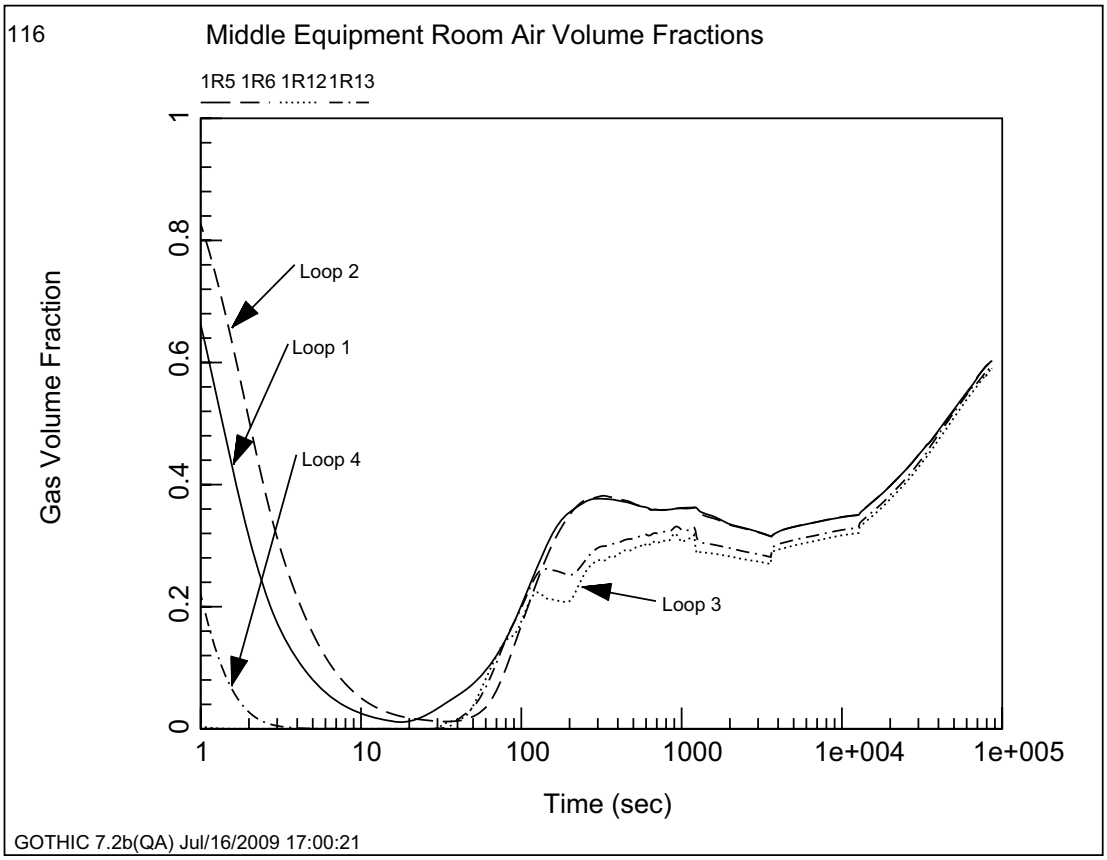


Figure 6.2.1-07d-92—Steam Volume Fraction in the Middle Equipment Rooms

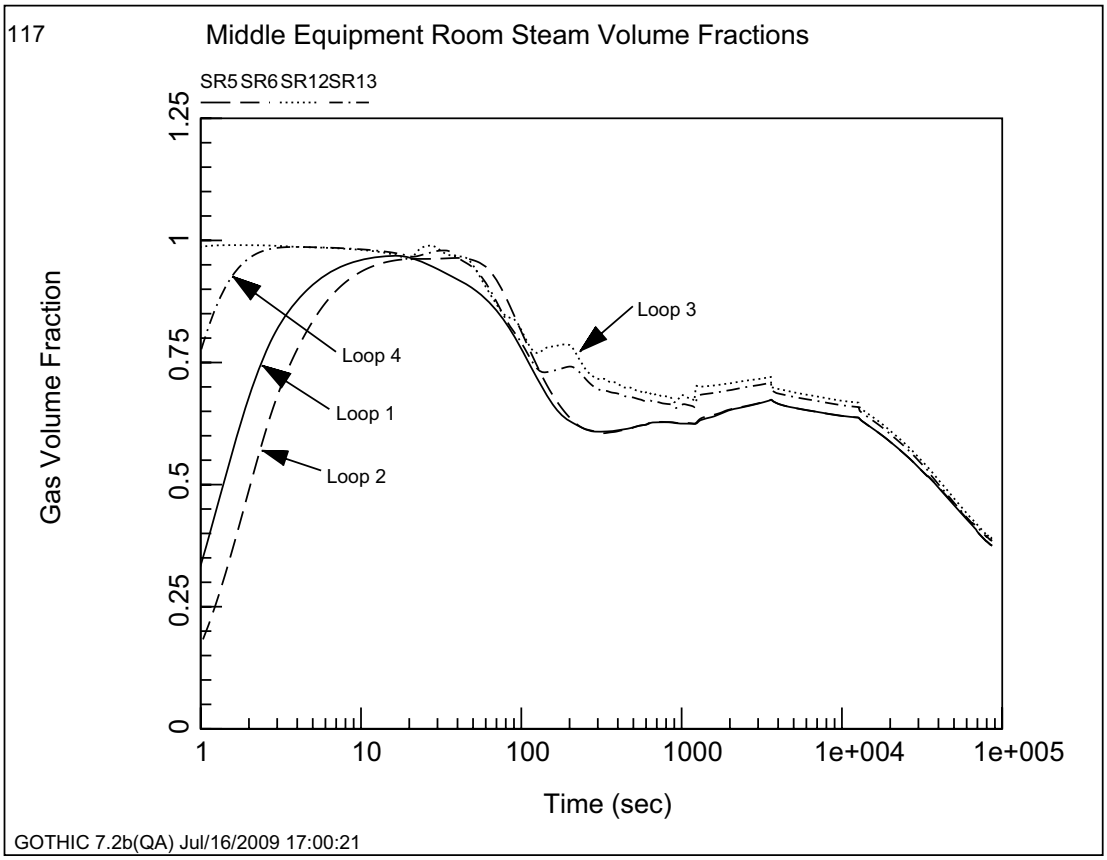


Figure 6.2.1-07d-93—Air and Steam Volume Fraction in the Upper Equipment Rooms

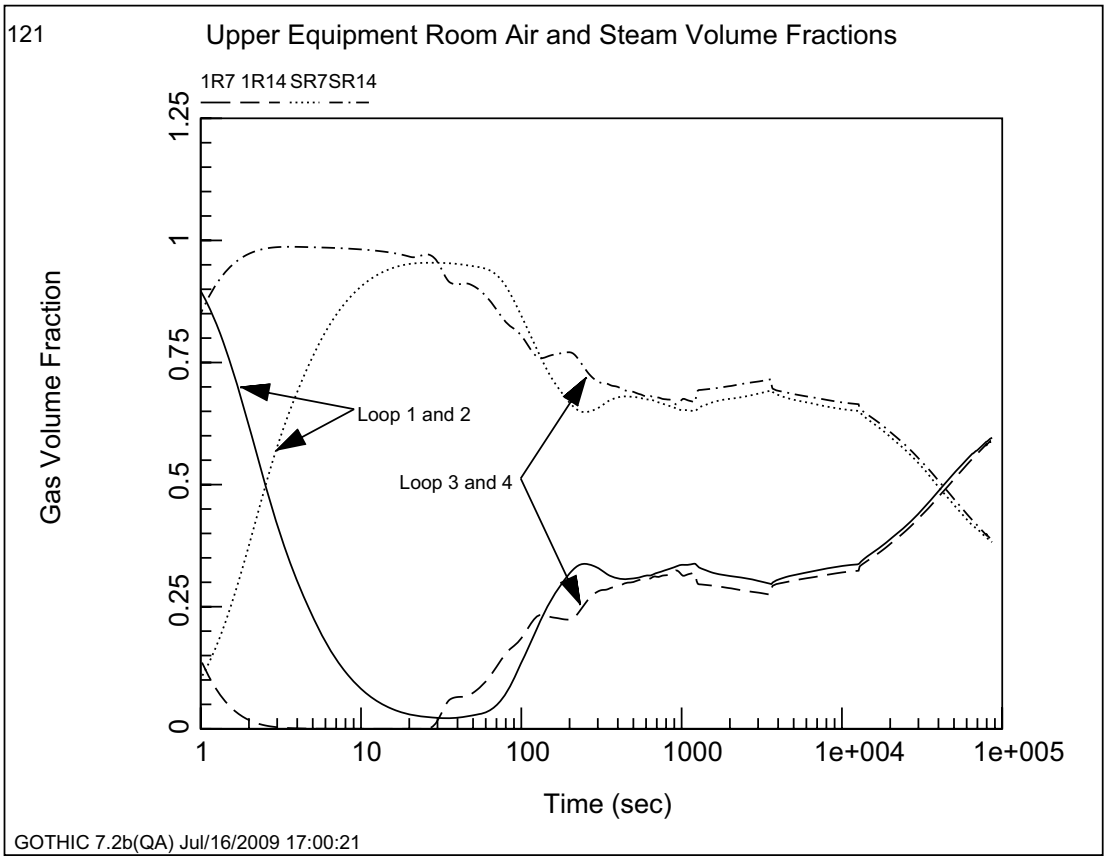


Figure 6.2.1-07d-94—Air and Steam Volume Fraction in the Reactor Pressure Vessel Pit and Reactor Cavity

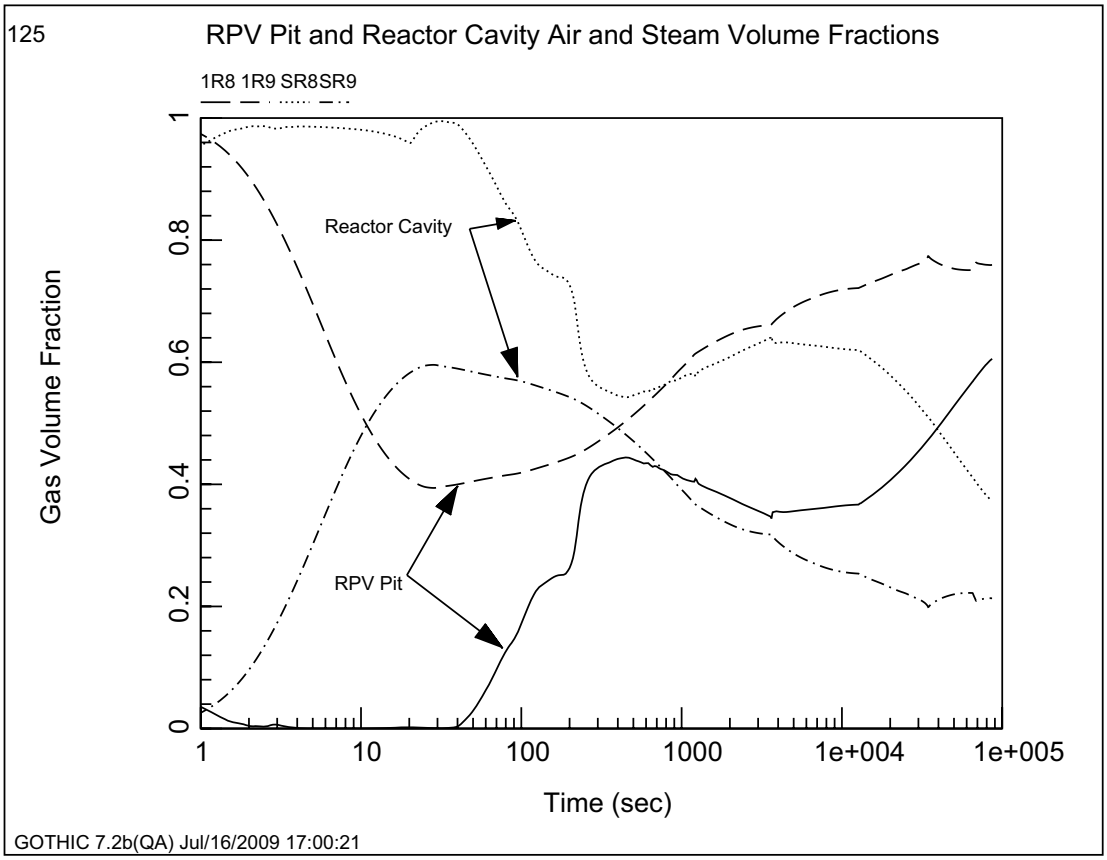
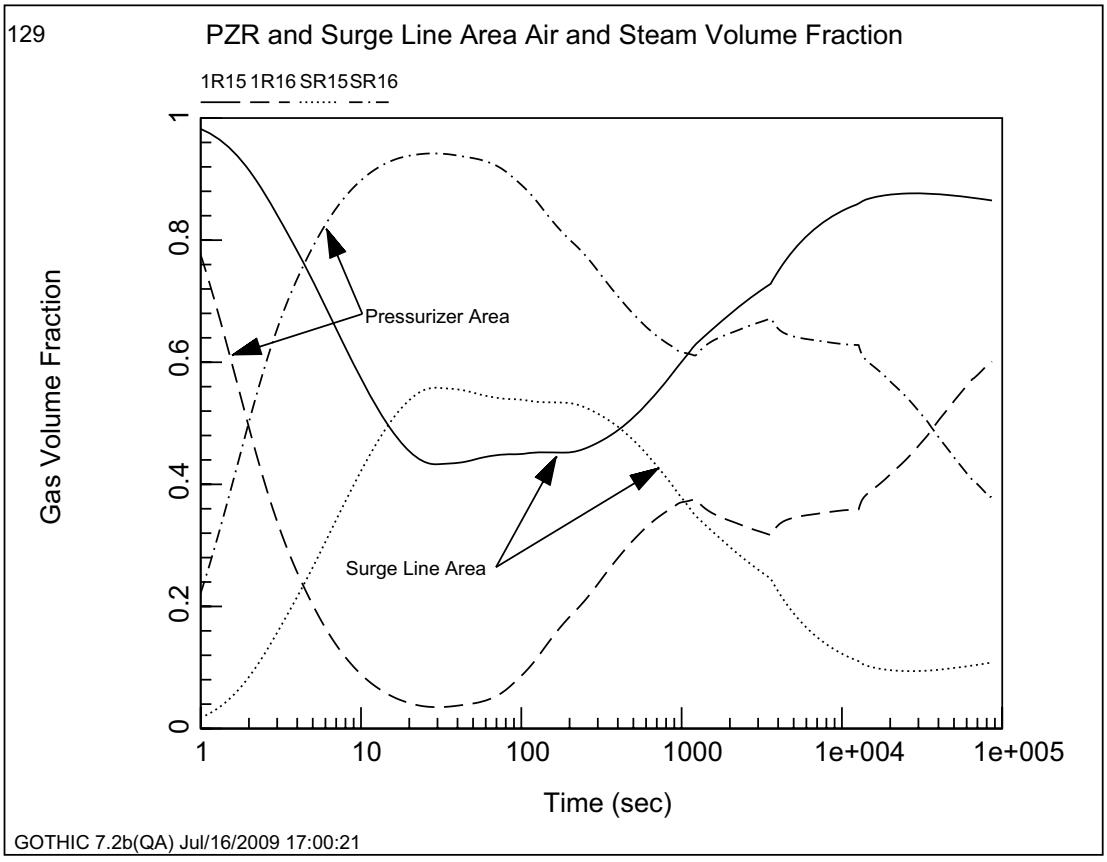


Figure 6.2.1-07d-95—Air and Steam Volume Fraction in the Pressurizer and Surge Line Rooms



Accessible Area

Figure 6.2.1-07d-96—Air and Steam Volume Fraction in the CVCS and Steam Generator Blowdown Heat Exchanger Rooms

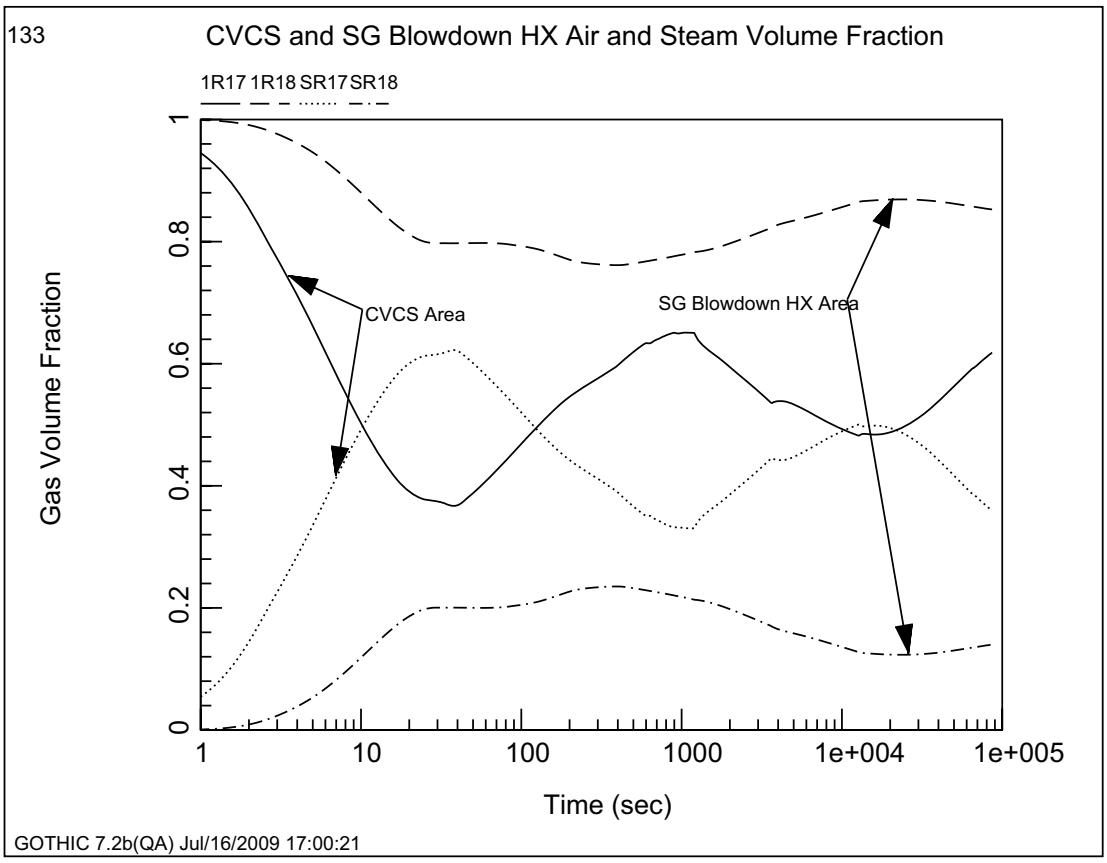


Figure 6.2.1-07d-97—Air and Steam Volume Fraction in the Lower Annulus Rooms

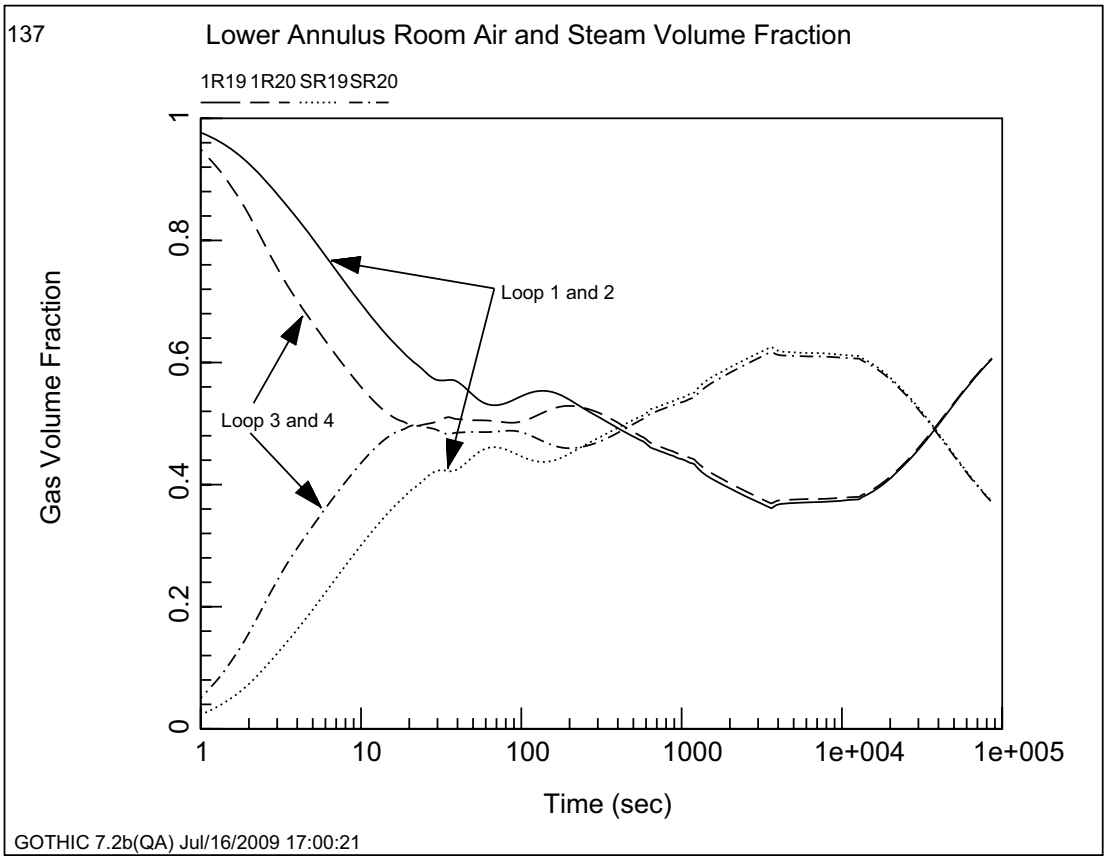


Figure 6.2.1-07d-98—Air and Steam Volume Fraction in the Middle Annulus Rooms

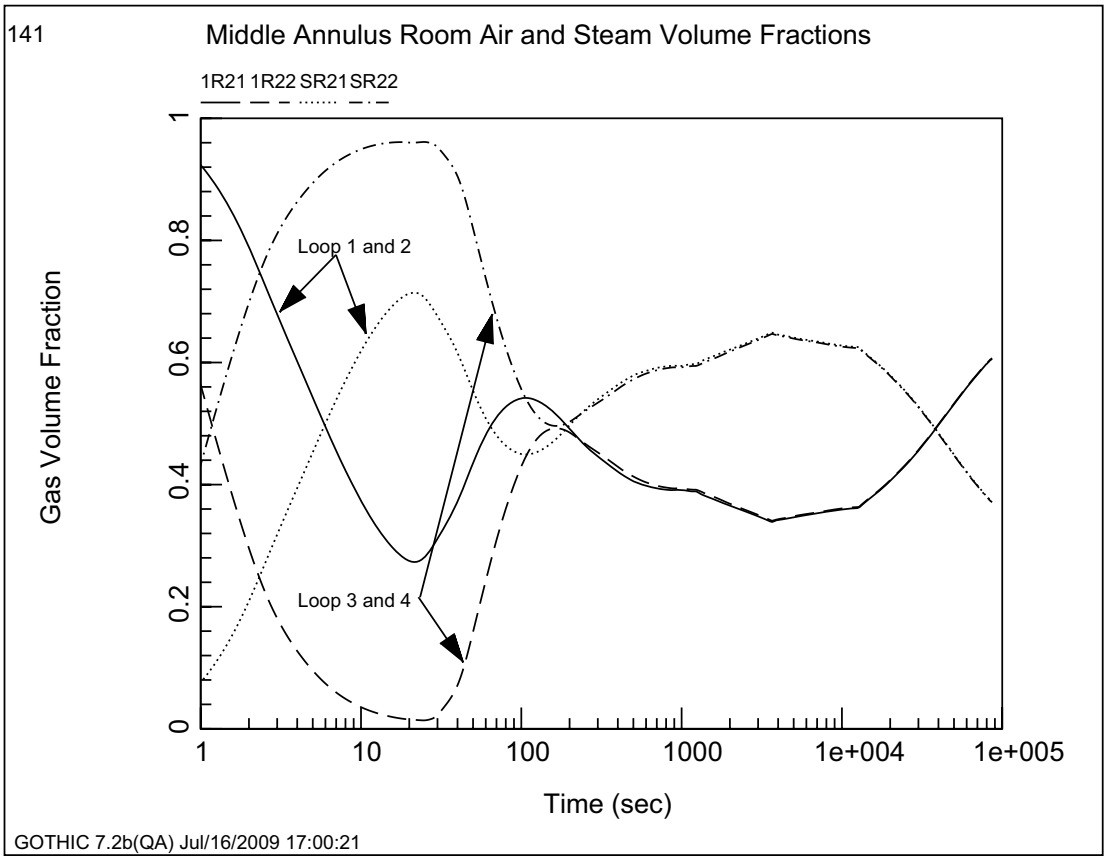


Figure 6.2.1-07d-99—Air and Steam Volume Fraction in the Upper Annulus Rooms

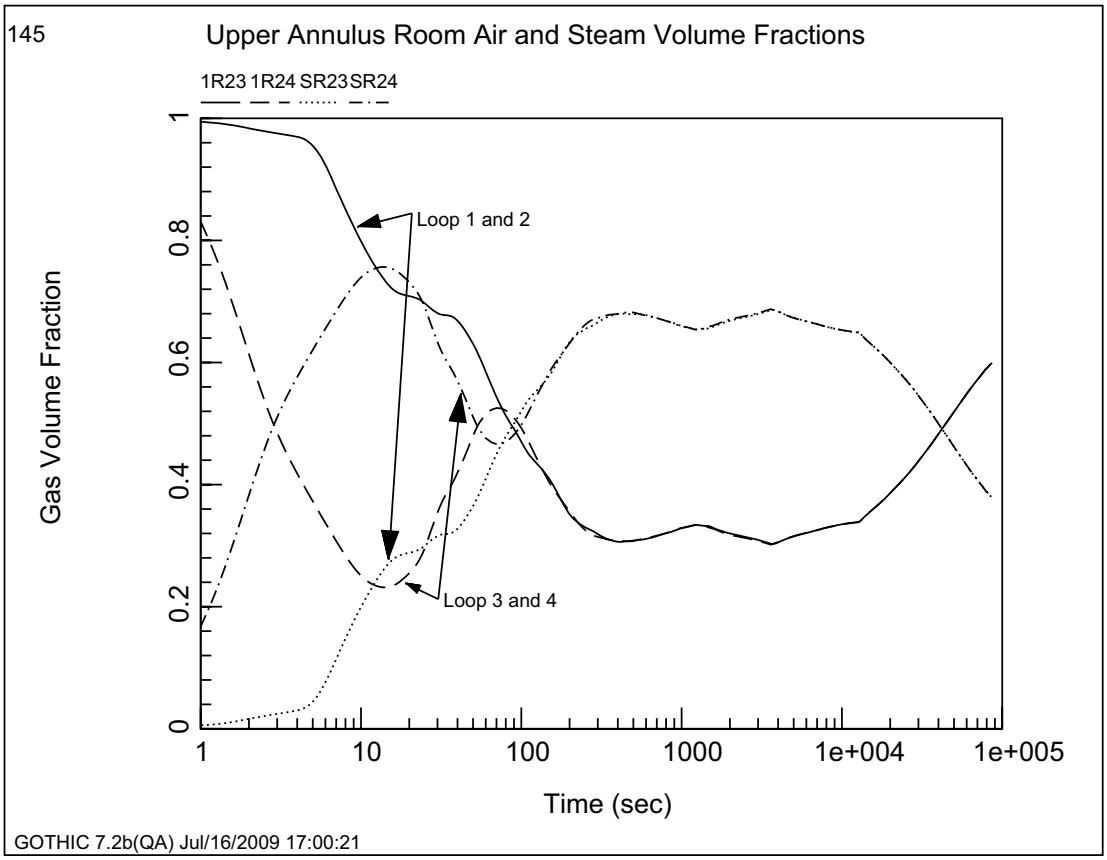


Figure 6.2.1-07d-100—Air Volume Fraction in the Access and Hot Piping Penetration Rooms

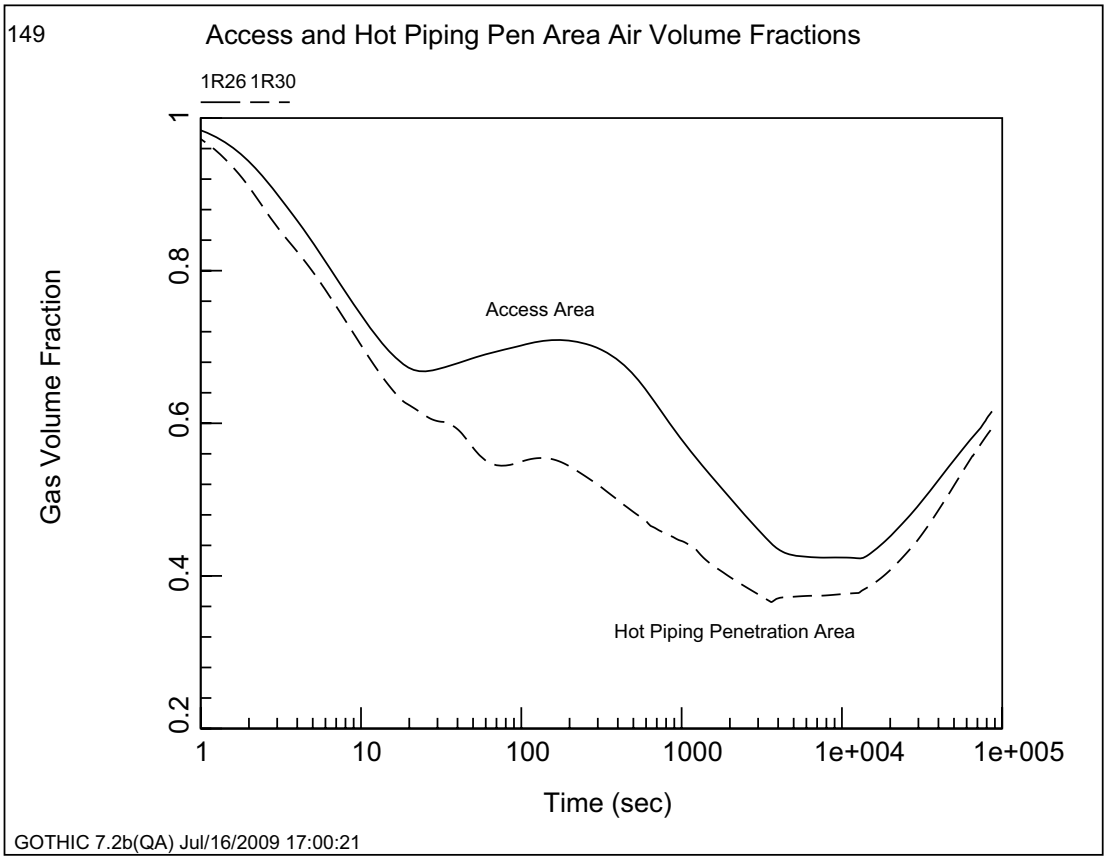


Figure 6.2.1-07d-101—Steam Volume Fraction in the Access and Hot Piping Penetration Rooms

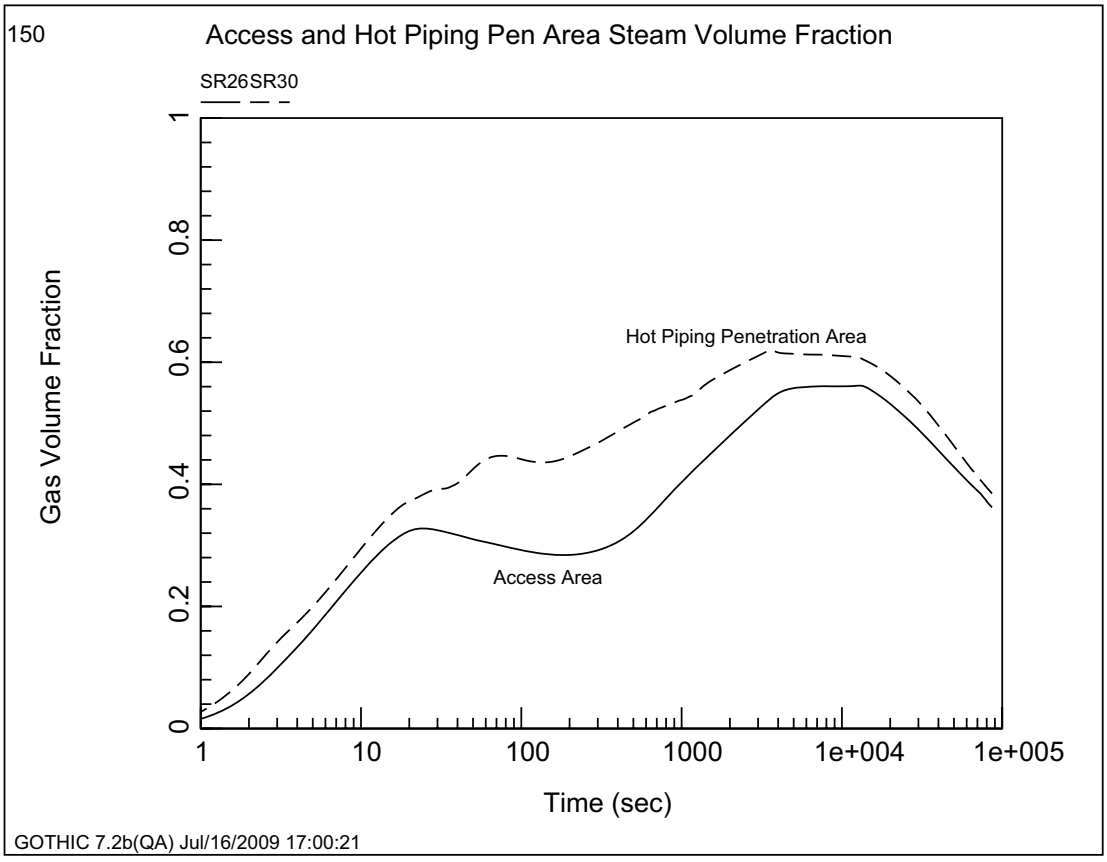


Figure 6.2.1-07d-102—Air Volume Fraction in the North and South Staircases and in the Elevator

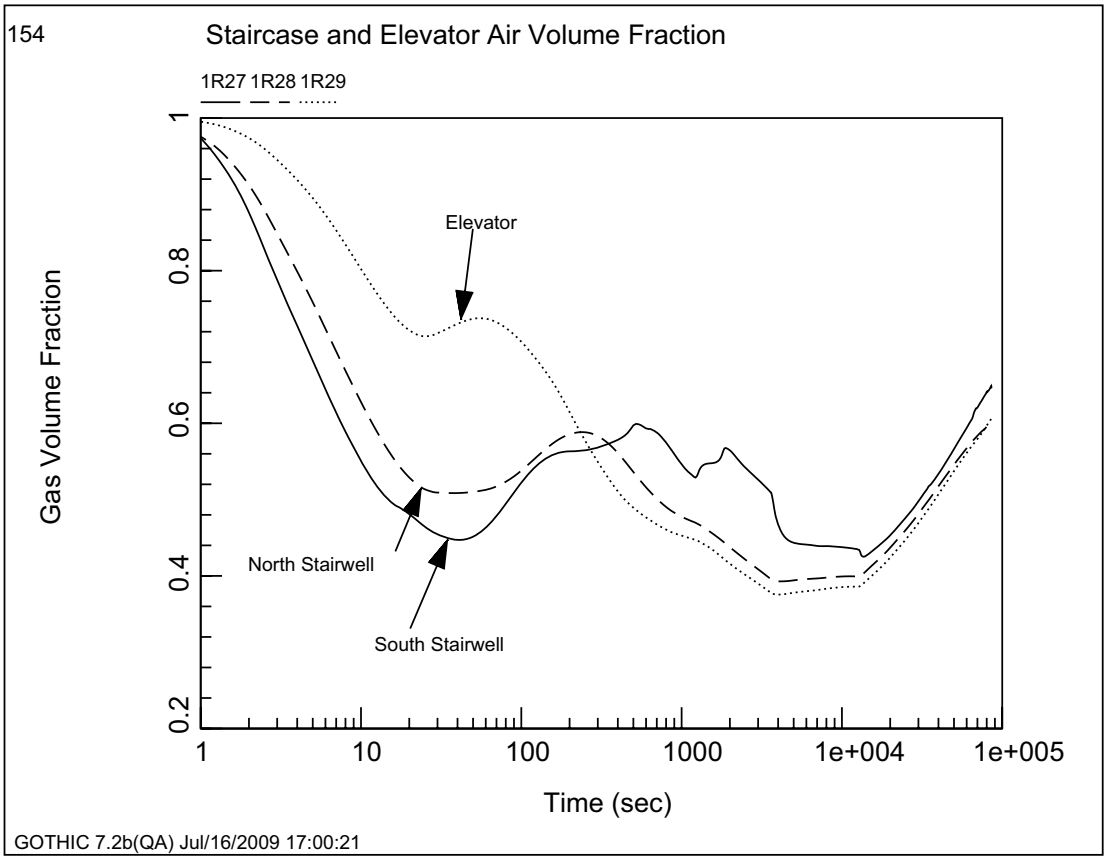


Figure 6.2.1-07d-103—Steam Volume Fraction in the North and South Staircases and in the Elevator

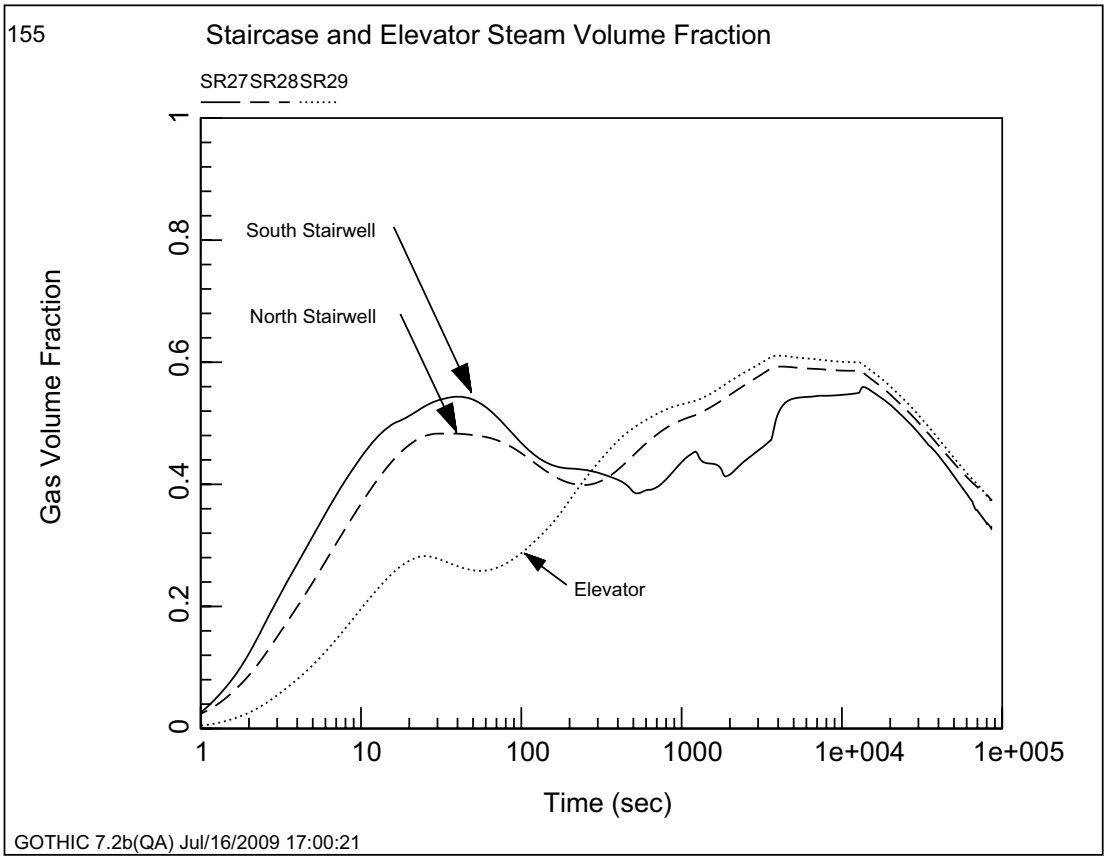


Figure 6.2.1-07d-104—Containment Dome Air Volume Fraction from the 63.98 ft to the 71.53 ft Elevation

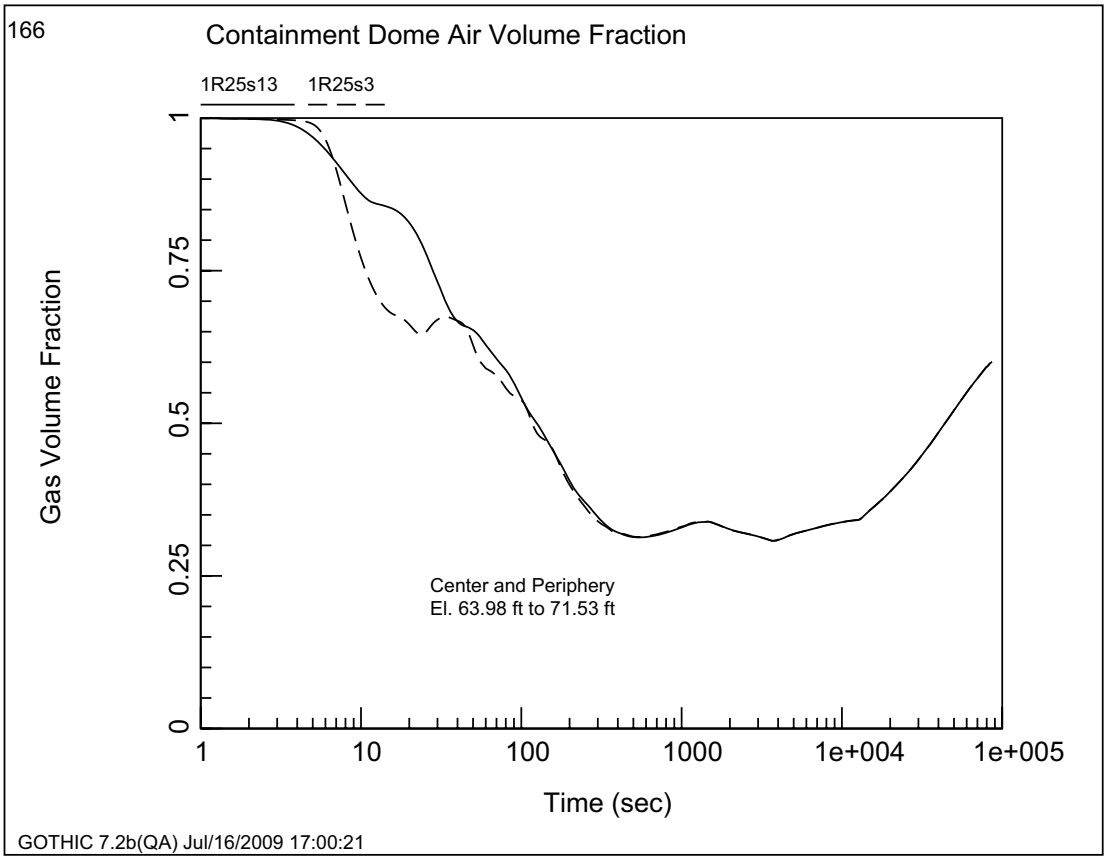


Figure 6.2.1-07d-105—Containment Dome Steam Volume Fraction from the 63.98 ft to the 71.53 ft Elevation

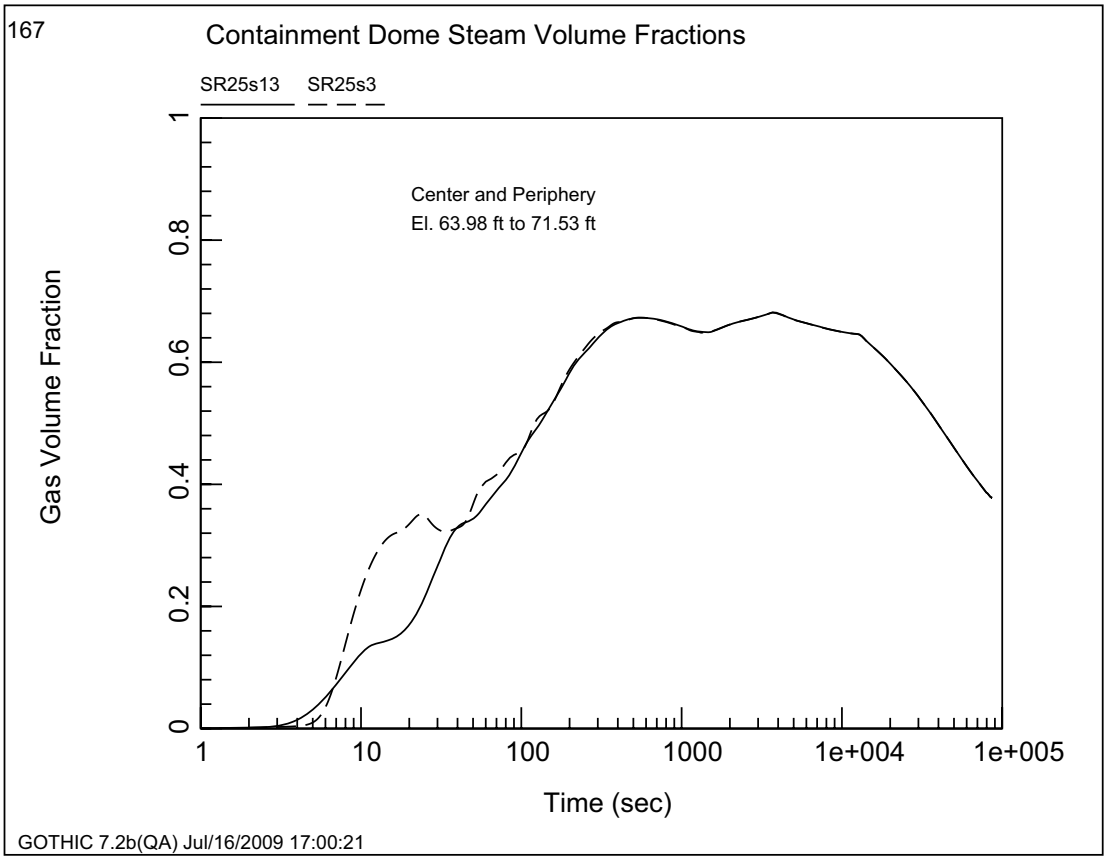


Figure 6.2.1-07d-106—Containment Dome Air Volume Fraction from the 79.07 ft to the 86.29 ft Elevation

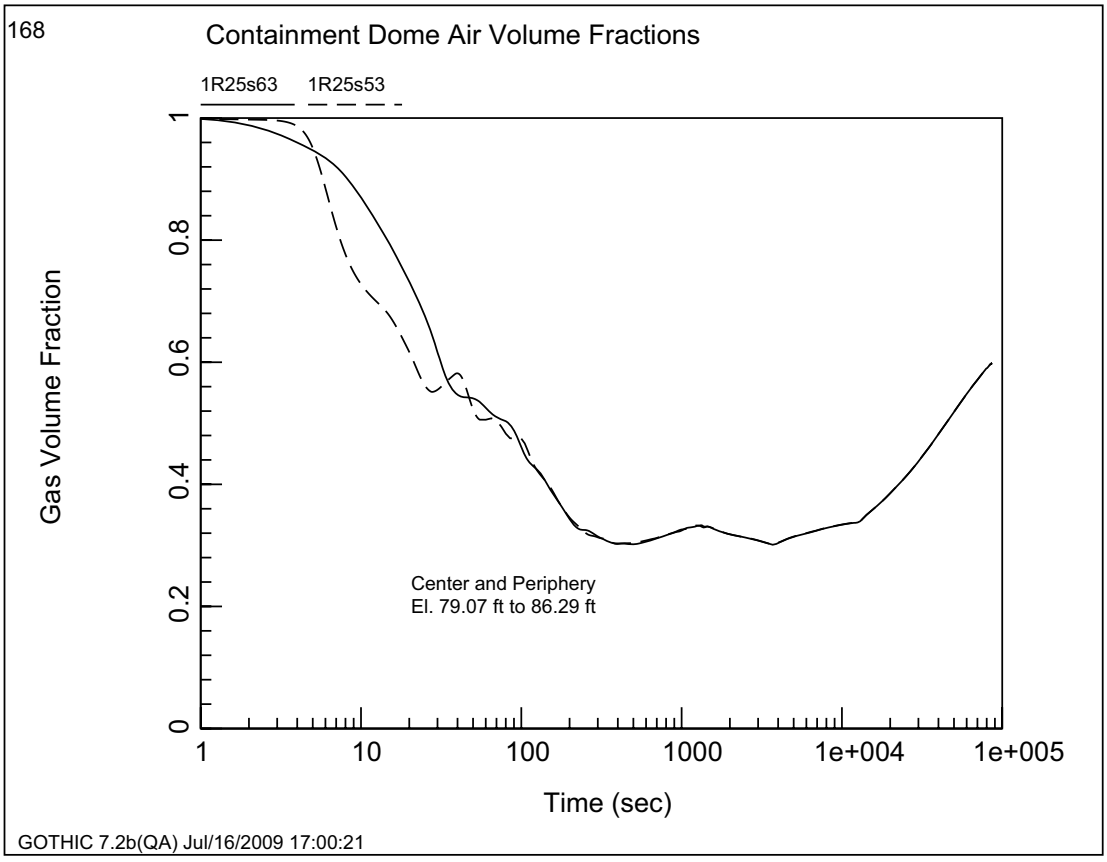


Figure 6.2.1-07d-107—Containment Dome Steam Volume Fraction from the 79.07 ft to the 86.29 ft Elevation

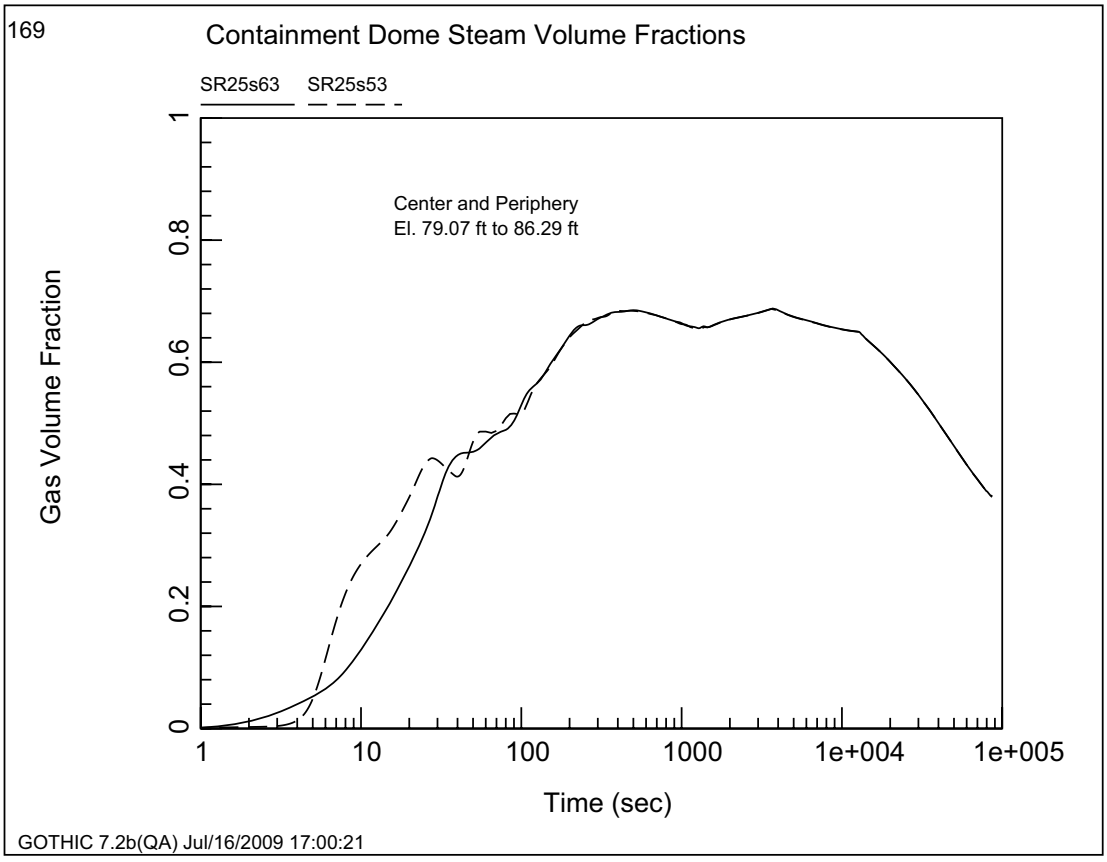


Figure 6.2.1-07d-108—Containment Dome Air Volume Fraction from the 93.51 ft to the 98.59 ft Elevation

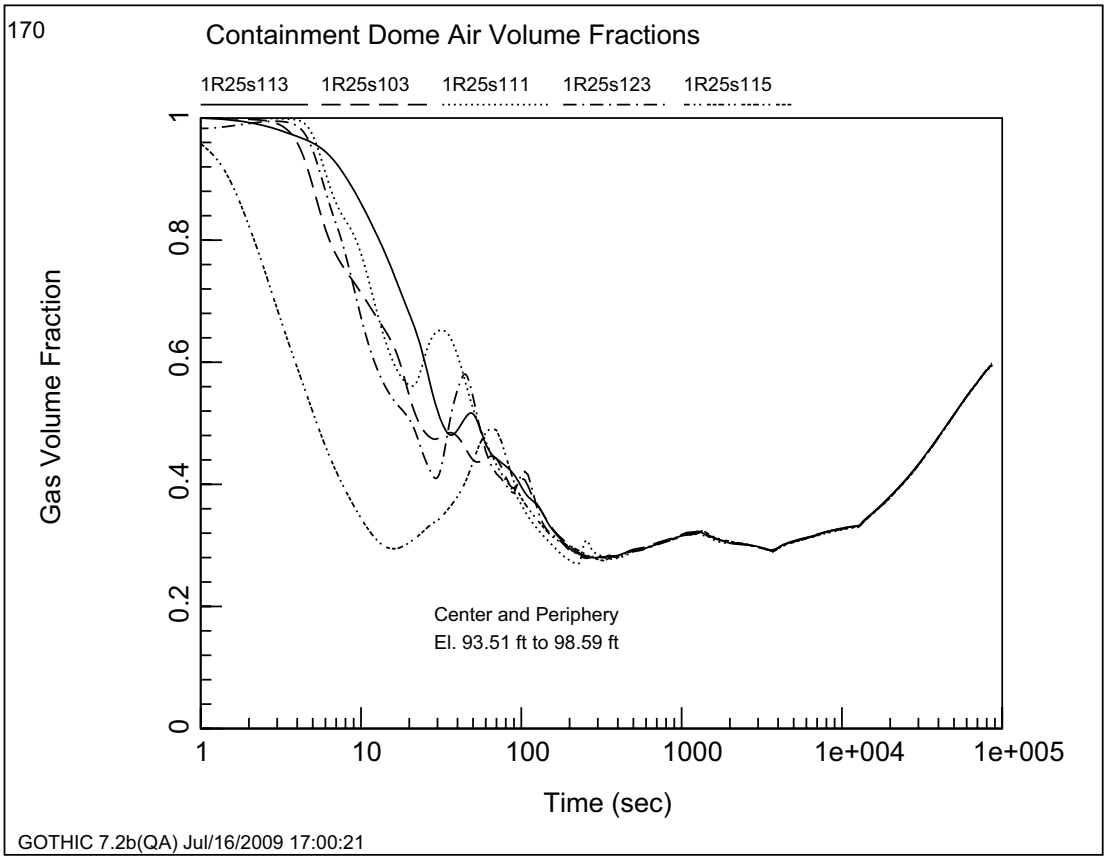


Figure 6.2.1-07d-109—Containment Dome Steam Volume Fraction from the 93.51 ft to the 98.59 ft Elevation

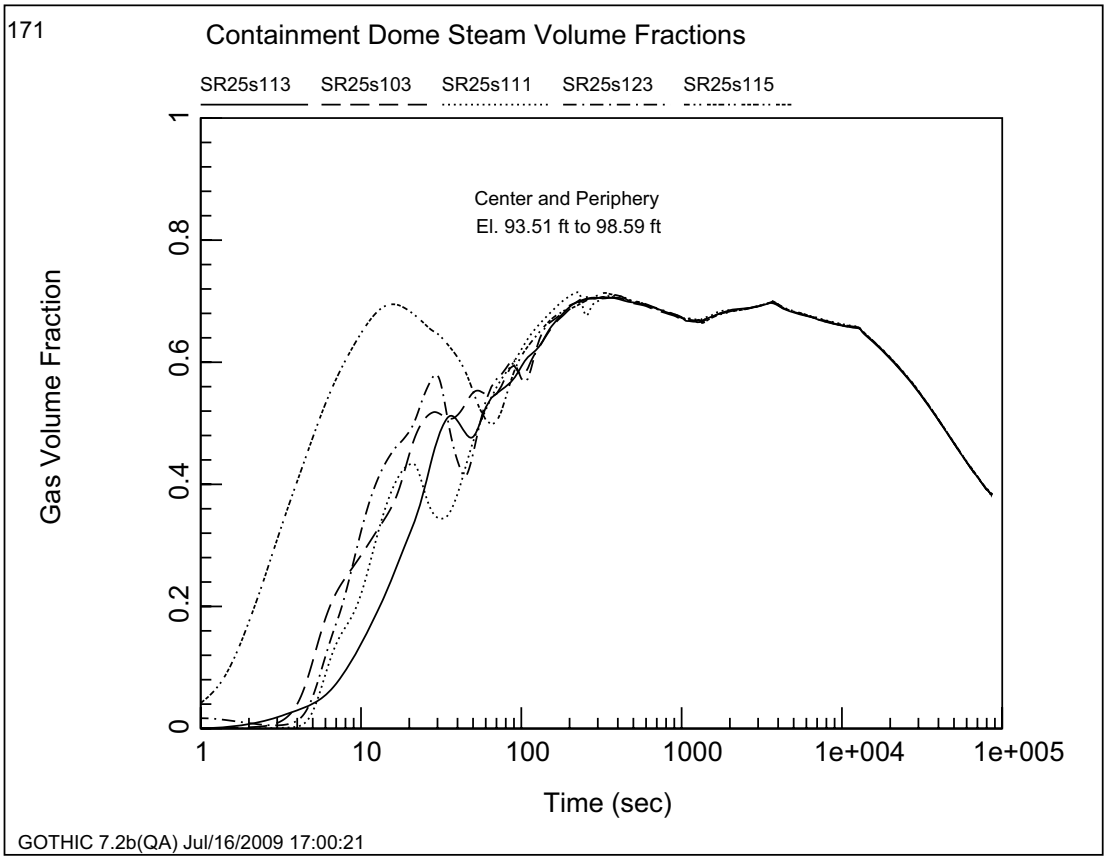


Figure 6.2.1-07d-110—Containment Dome Air Volume Fraction from the 103.68 ft to the 108.76 ft Elevation

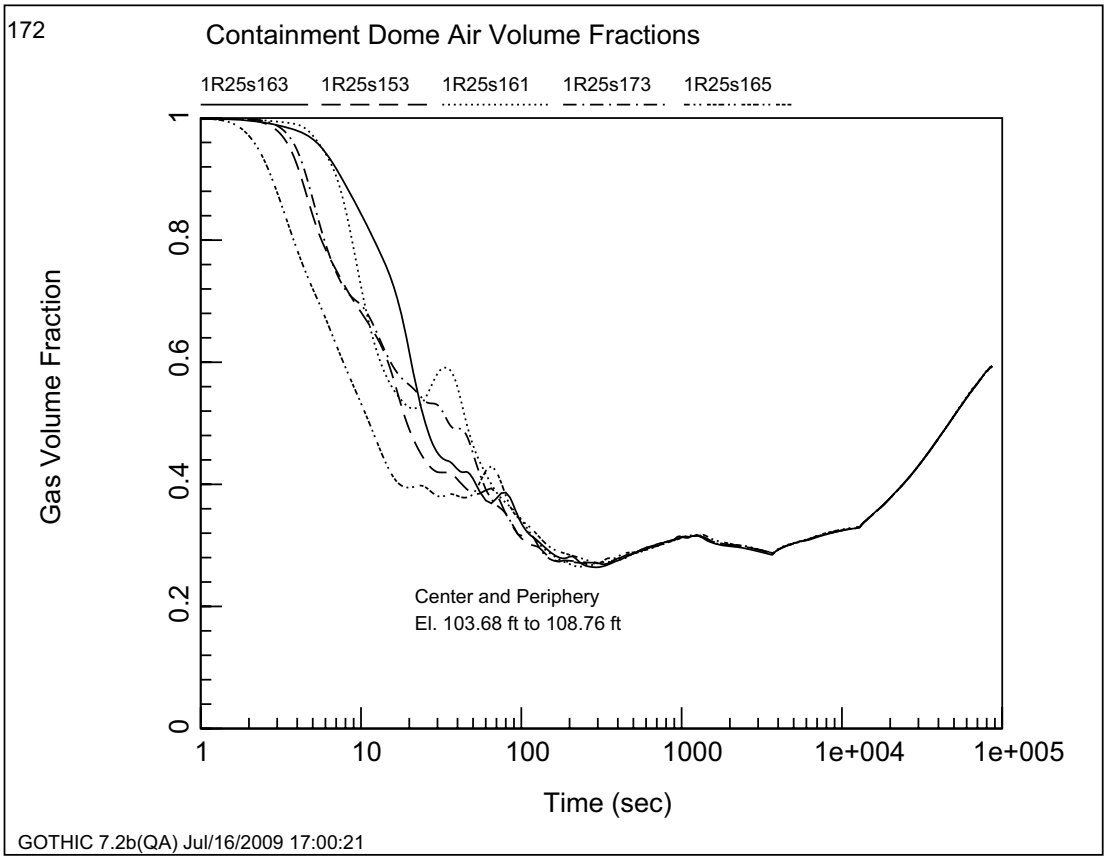


Figure 6.2.1-07d-111—Containment Dome Steam Volume Fraction from the 103.68 ft to the 108.76 ft Elevation

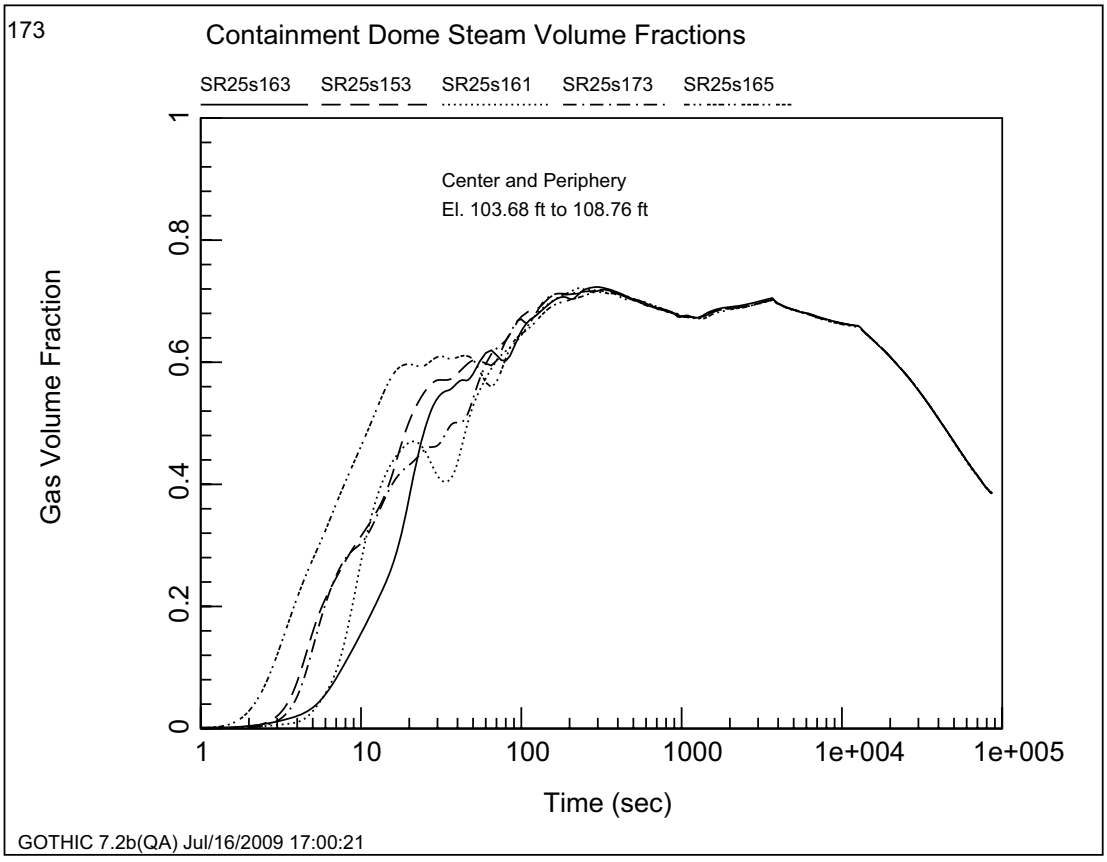


Figure 6.2.1-07d-112—Containment Dome Air Volume Fraction from the 113.85 ft to the 119.90 ft Elevation

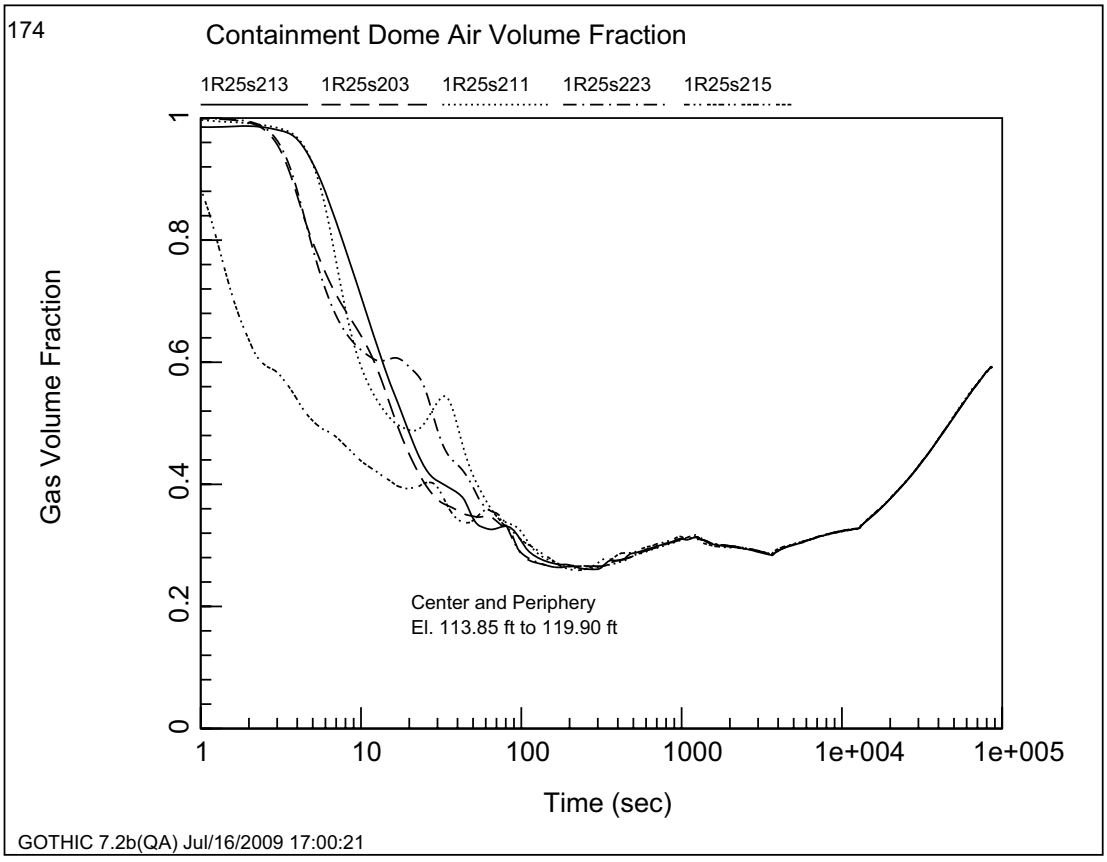


Figure 6.2.1-07d-113—Containment Dome Steam Volume Fraction from the 113.85 ft to the 119.90 ft Elevation

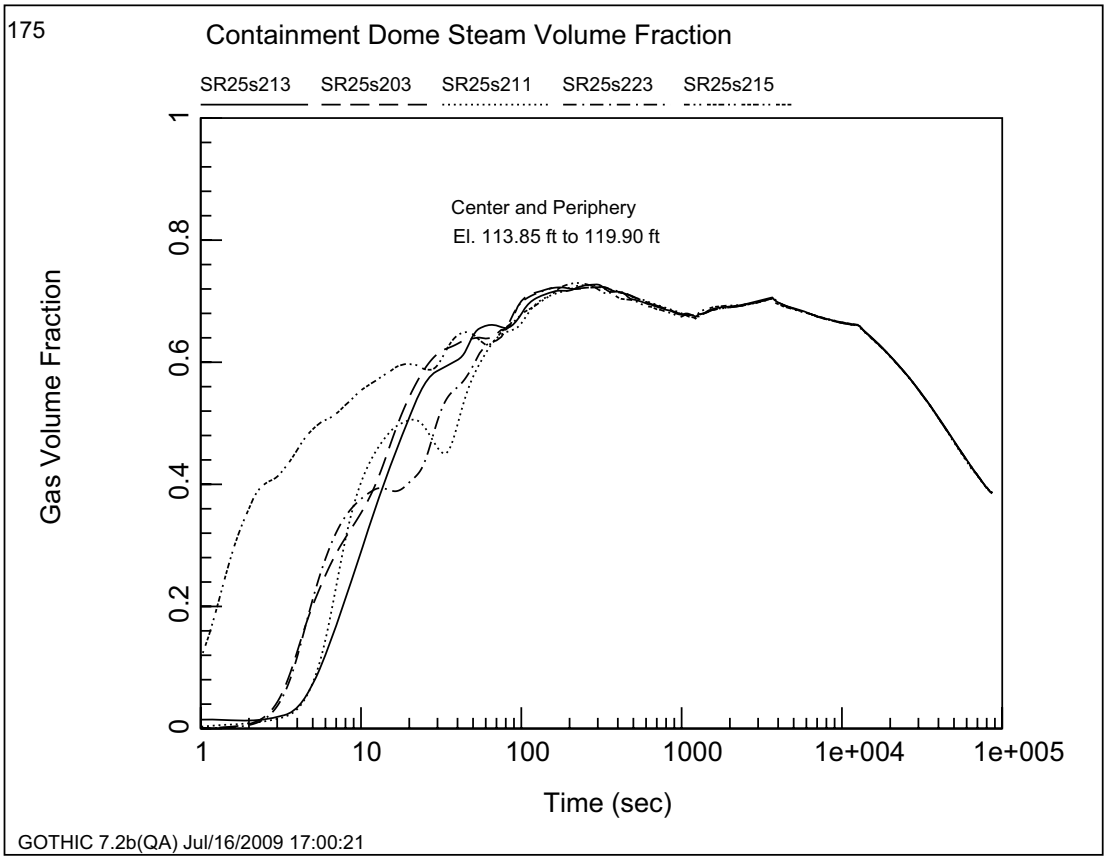


Figure 6.2.1-07d-114—Containment Dome Air Volume Fraction from the 125.95 ft to the 132.00 ft Elevation

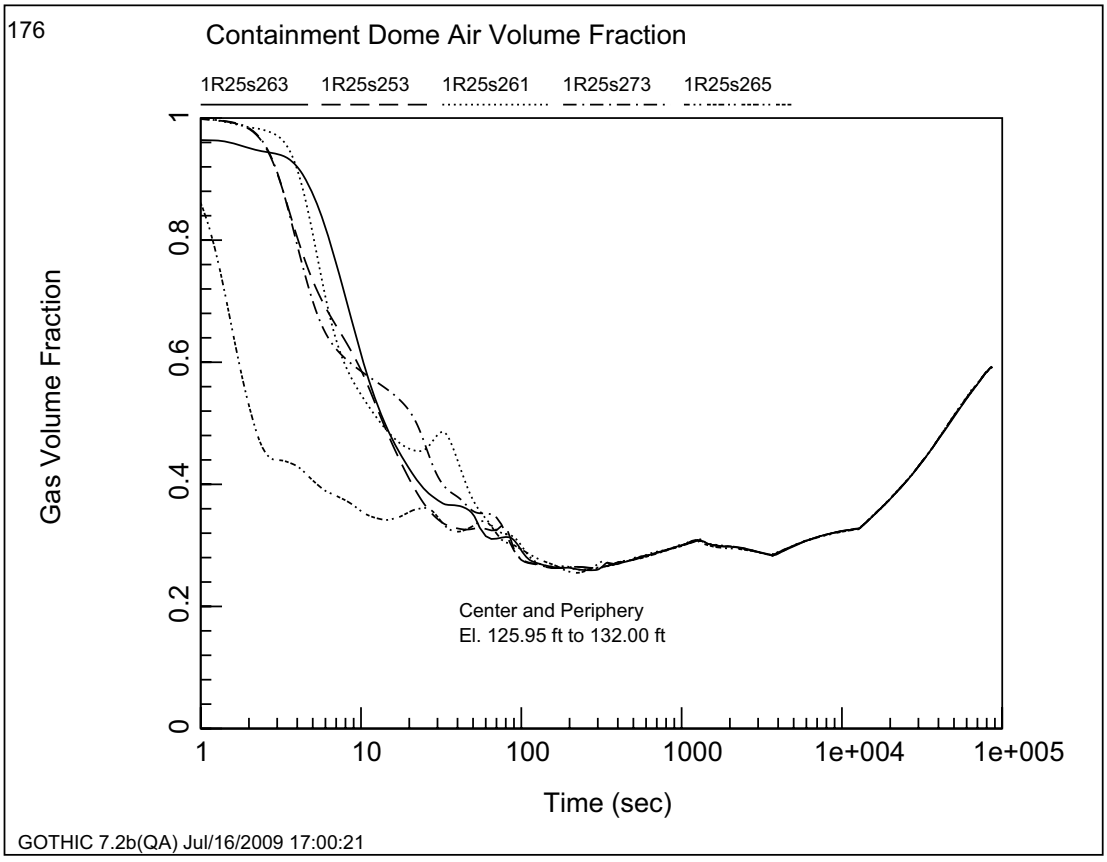


Figure 6.2.1-07d-115—Containment Dome Steam Volume Fraction from the 125.95 ft to the 132.00 ft Elevation

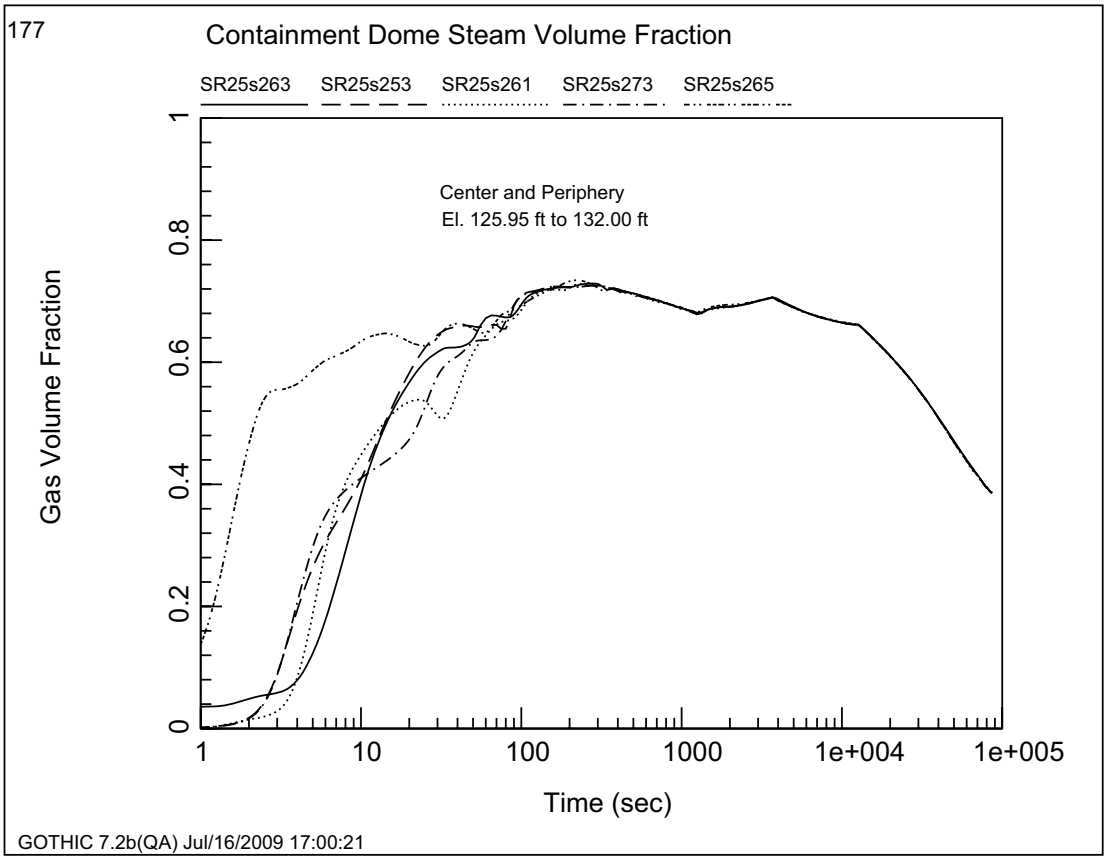


Figure 6.2.1-07d-116—Containment Dome Air Volume Fraction from the 138.05 ft to the 144.10 ft Elevation

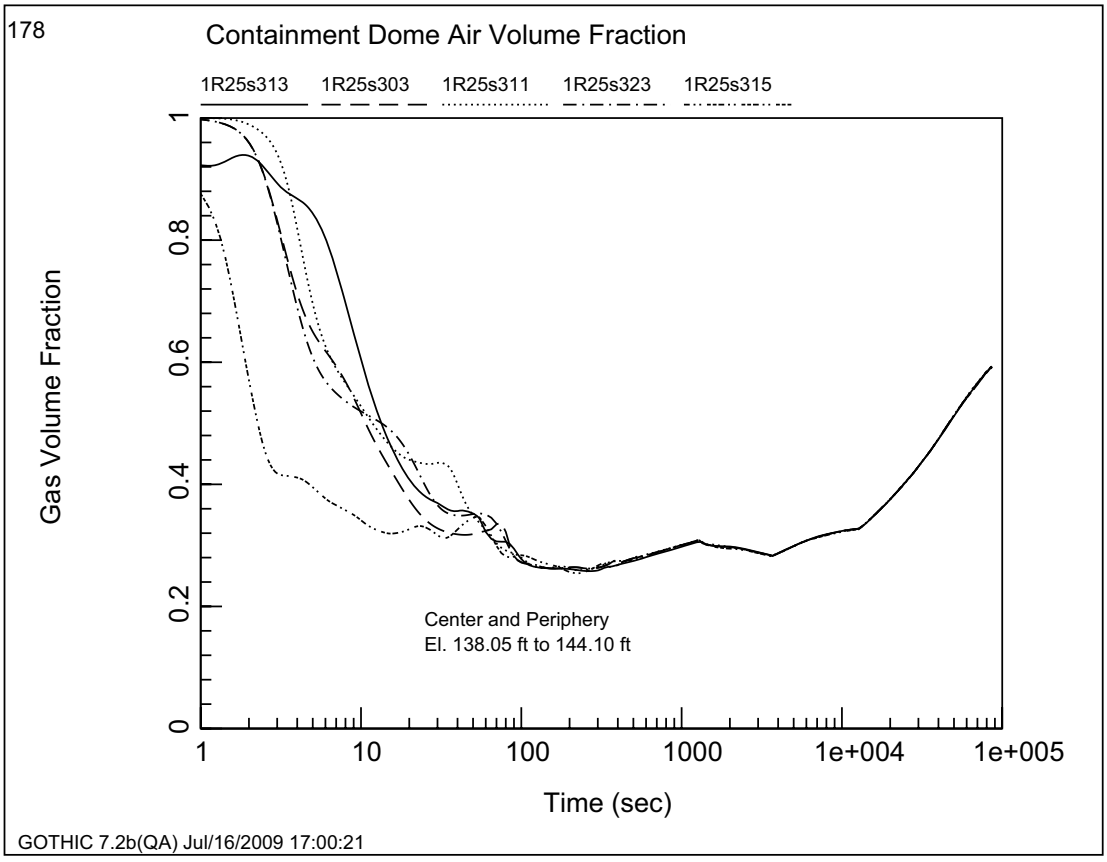


Figure 6.2.1-07d-117—Containment Dome Steam Volume Fraction from the 138.05 ft to the 144.10 ft Elevation

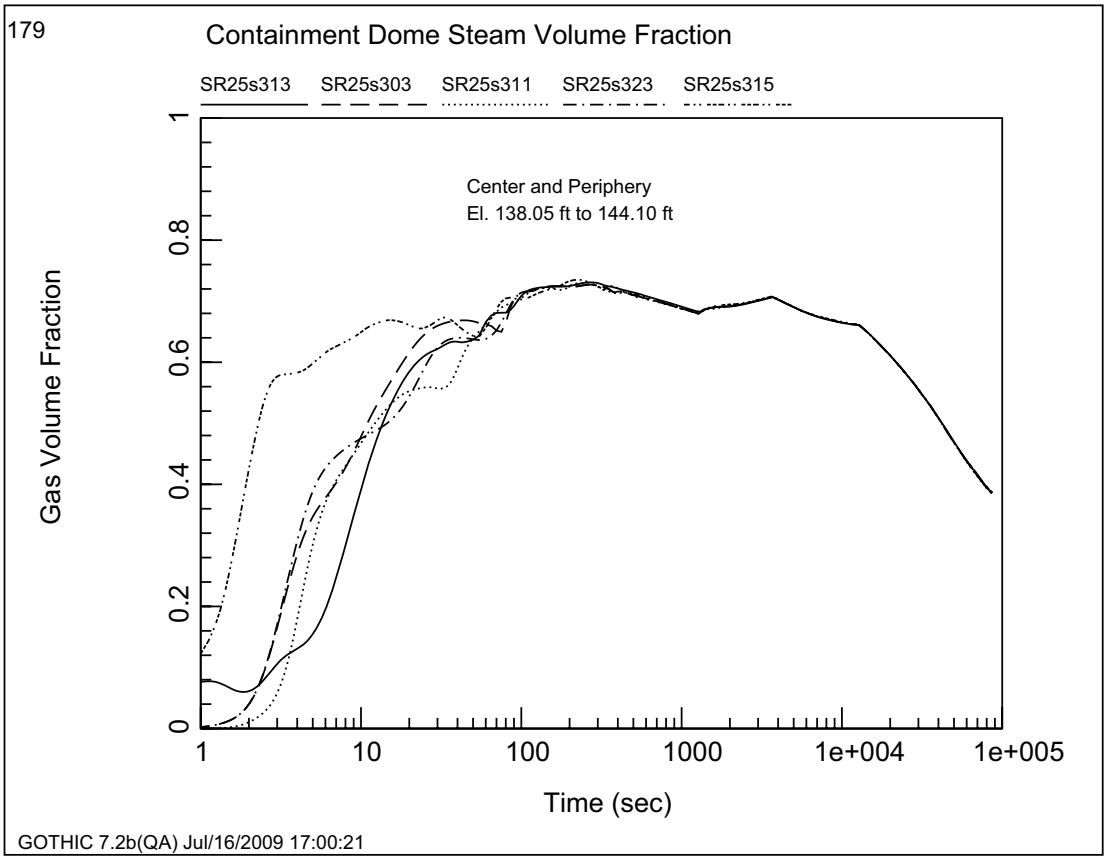


Figure 6.2.1-07d-118—Containment Dome Air Volume Fraction from the 151.12 ft to the 158.14 ft Elevation

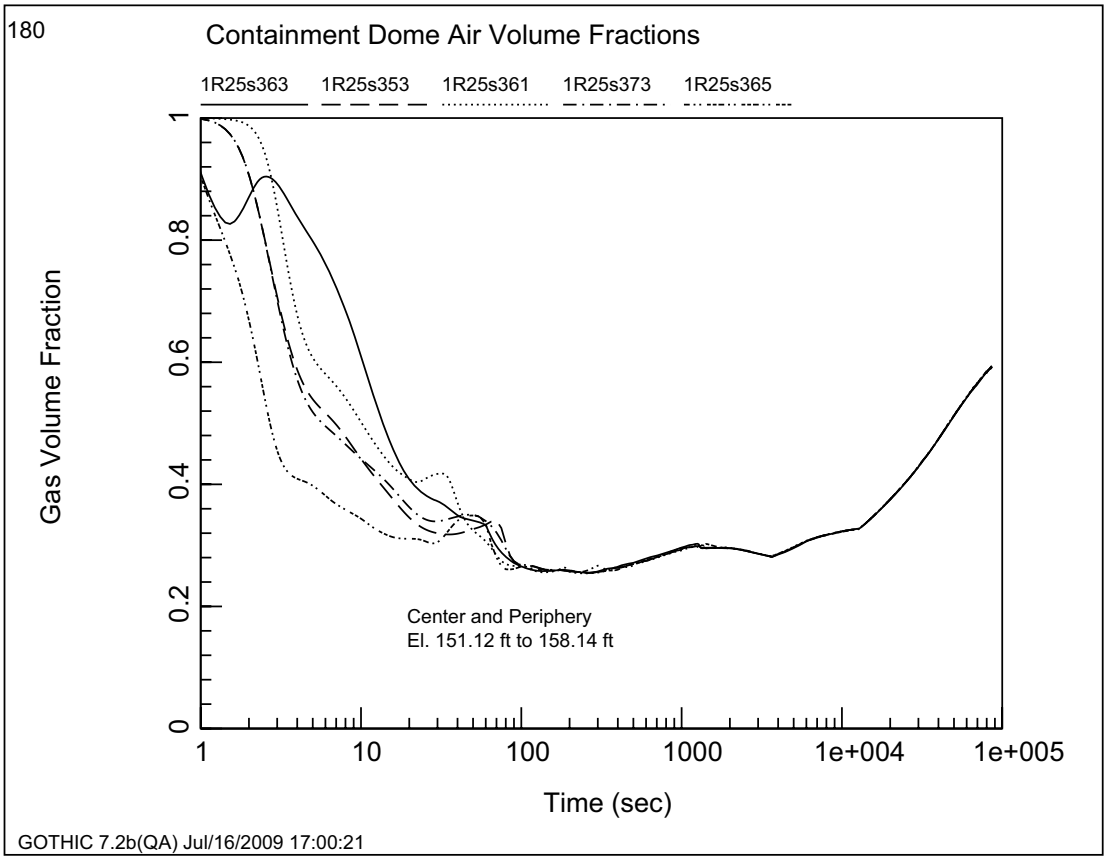


Figure 6.2.1-07d-119—Containment Dome Steam Volume Fraction from the 151.12 ft to the 158.14 ft Elevation

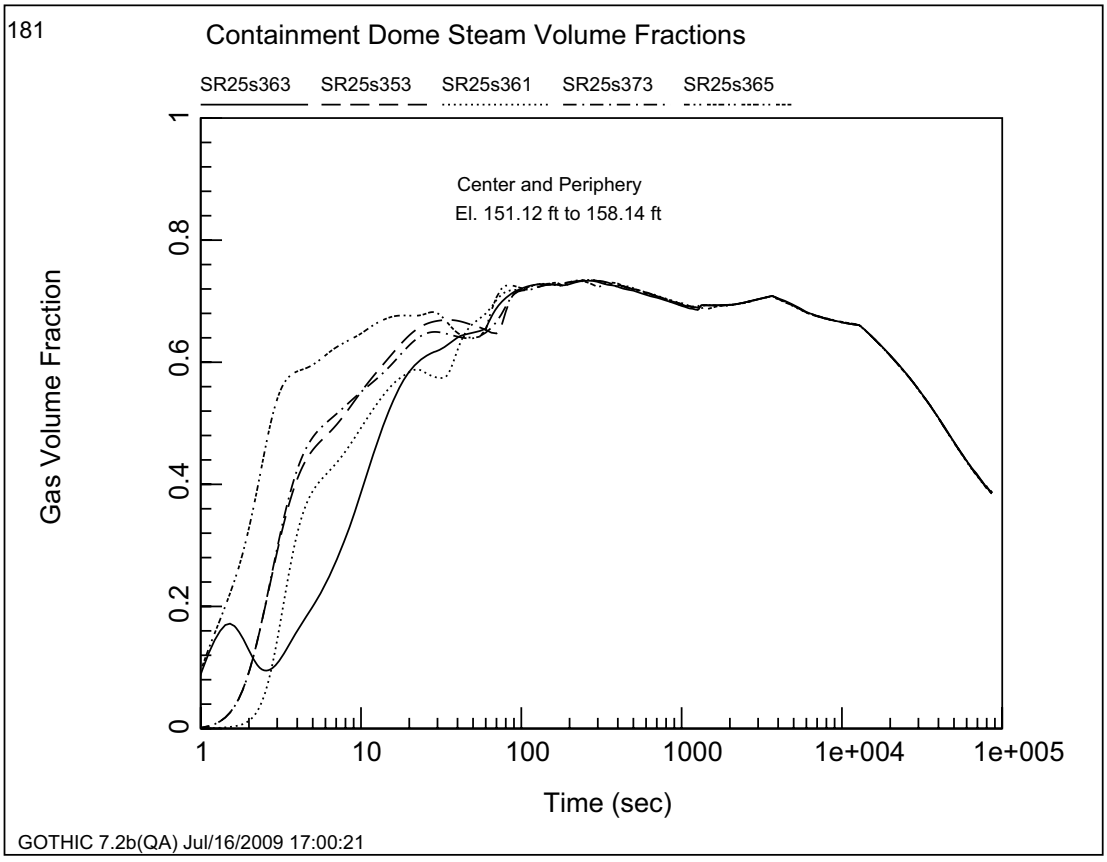


Figure 6.2.1-07d-120—Containment Dome Air Volume Fraction from the 165.16 ft to the 172.99 ft Elevation

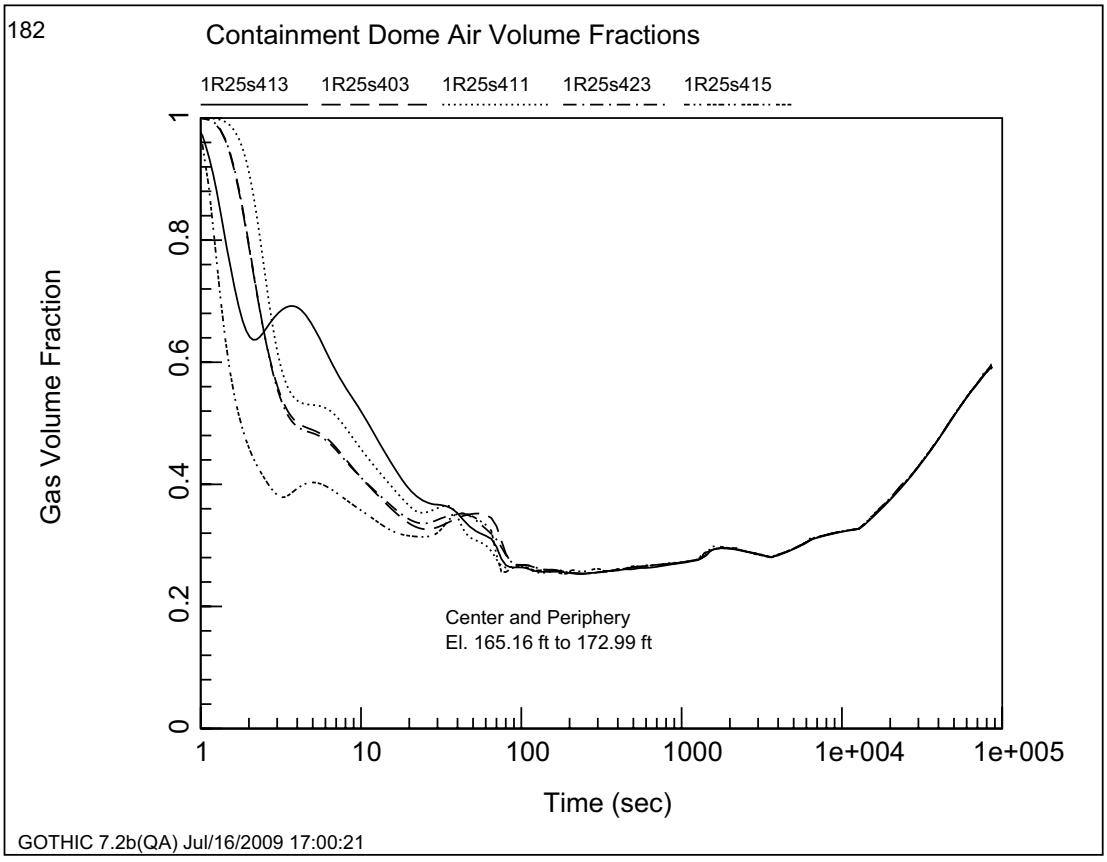


Figure 6.2.1-07d-121—Containment Dome Steam Volume Fraction from the 165.16 ft to the 172.99 ft Elevation

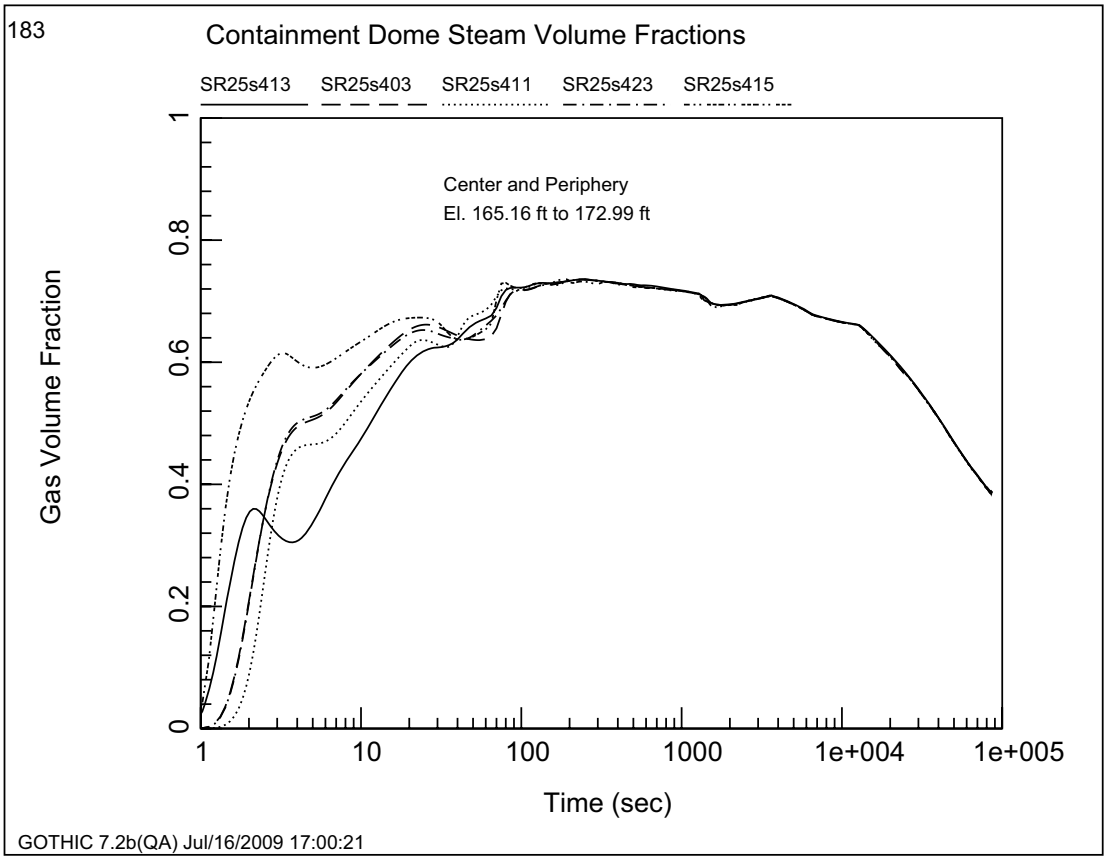


Figure 6.2.1-07d-122—Containment Dome Air Volume Fraction from the 180.82 ft to the 188.65 ft Elevation

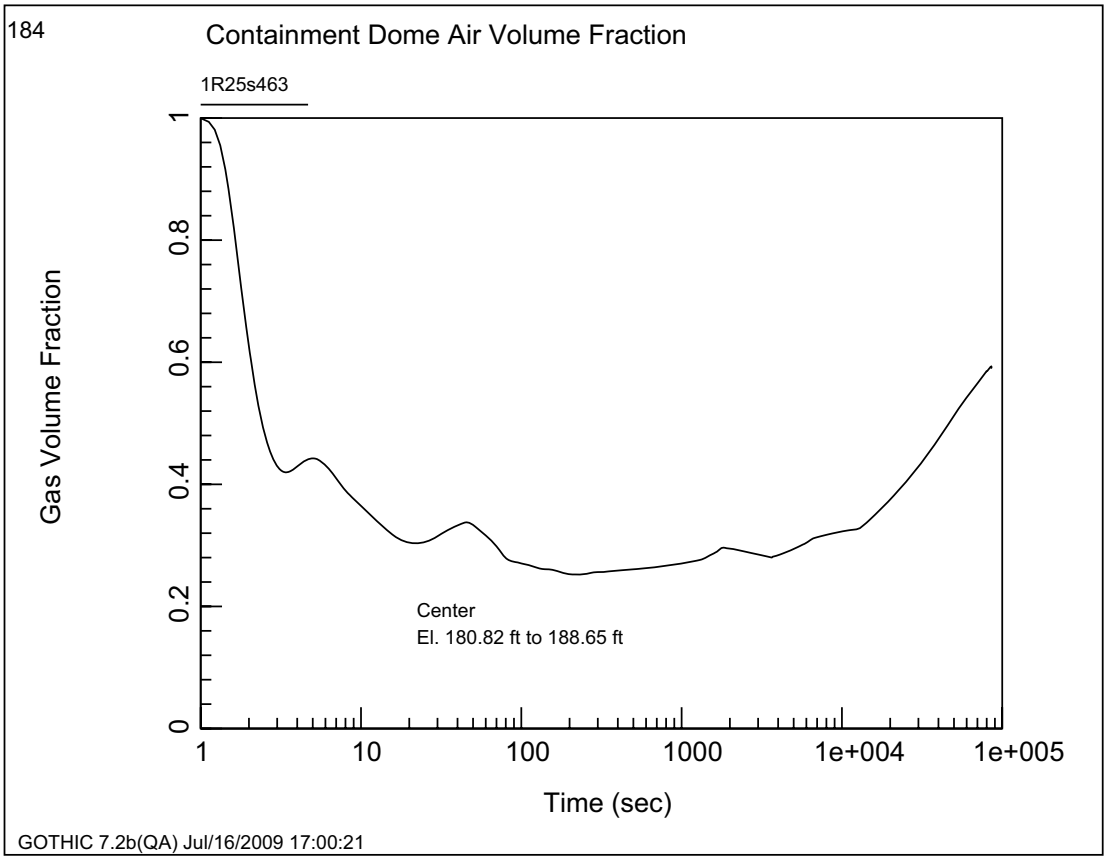
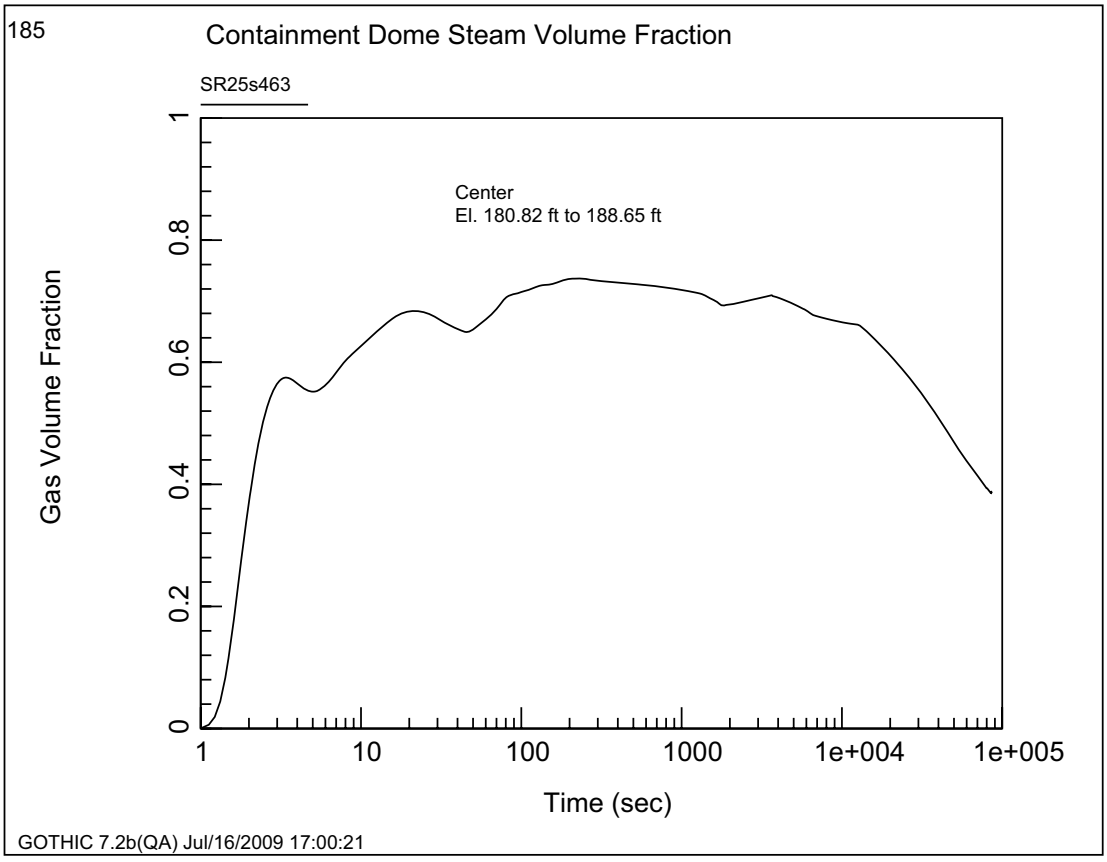


Figure 6.2.1-07d-123—Containment Dome Steam Volume Fraction from the 180.82 ft to the 188.65 ft Elevation



Containment Heat Conductor Surface Temperatures

Equipment Area

Figure 6.2.1-07d-124—IRWST Ceiling, Wall and Basemat Surface Temperature

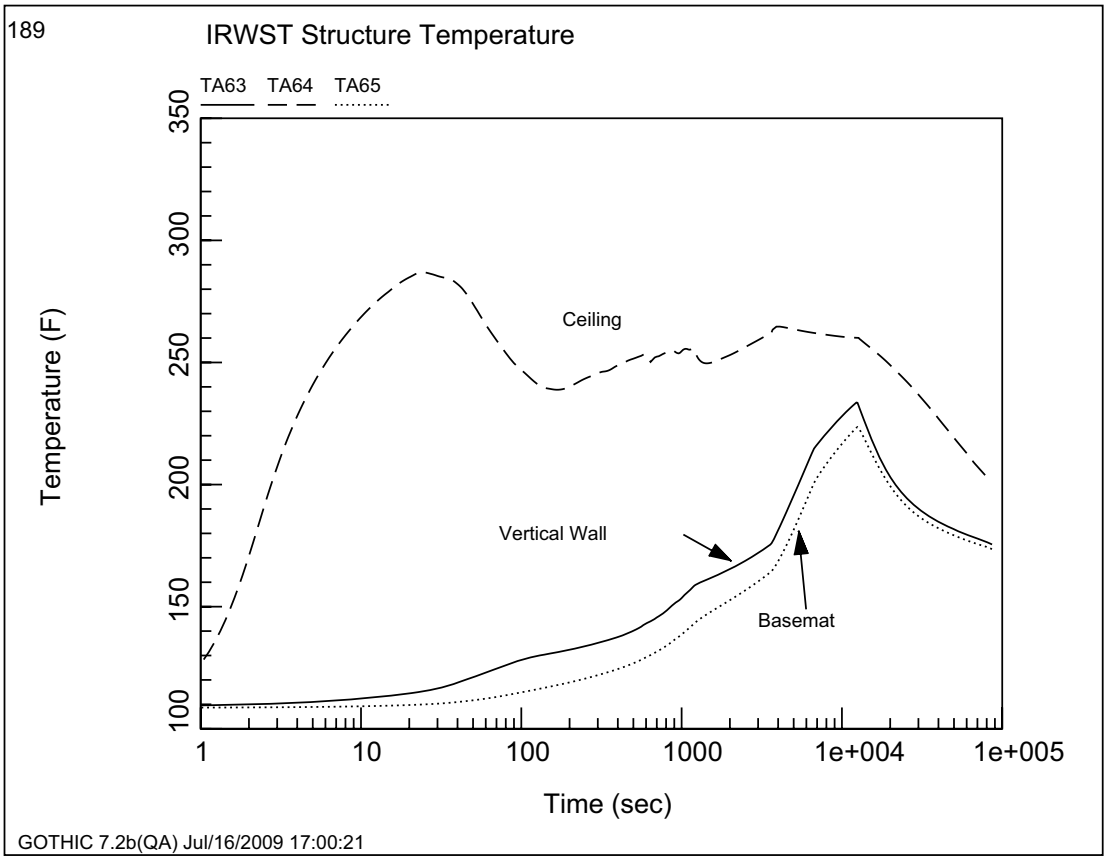


Figure 6.2.1-07d-125—IRWST Structural Steel Surface Temperature

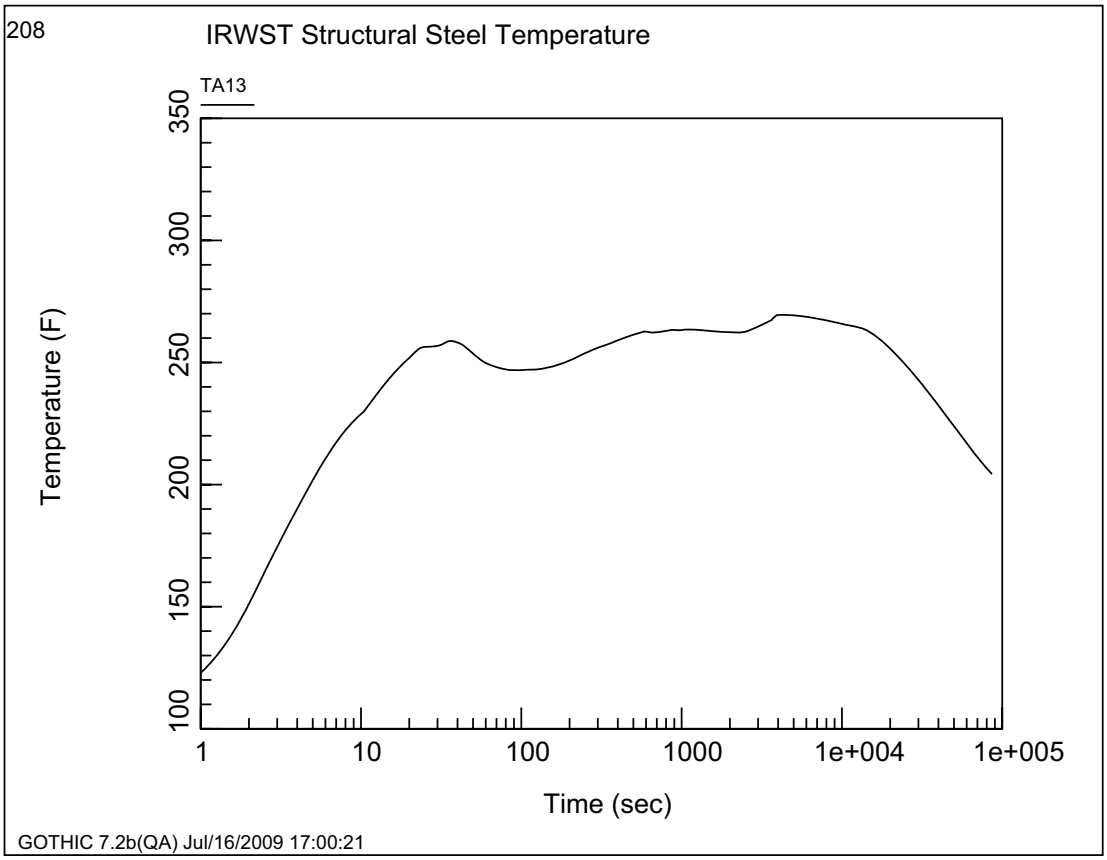


Figure 6.2.1-07d-126—Lower Equipment Room Wall Surface Temperatures

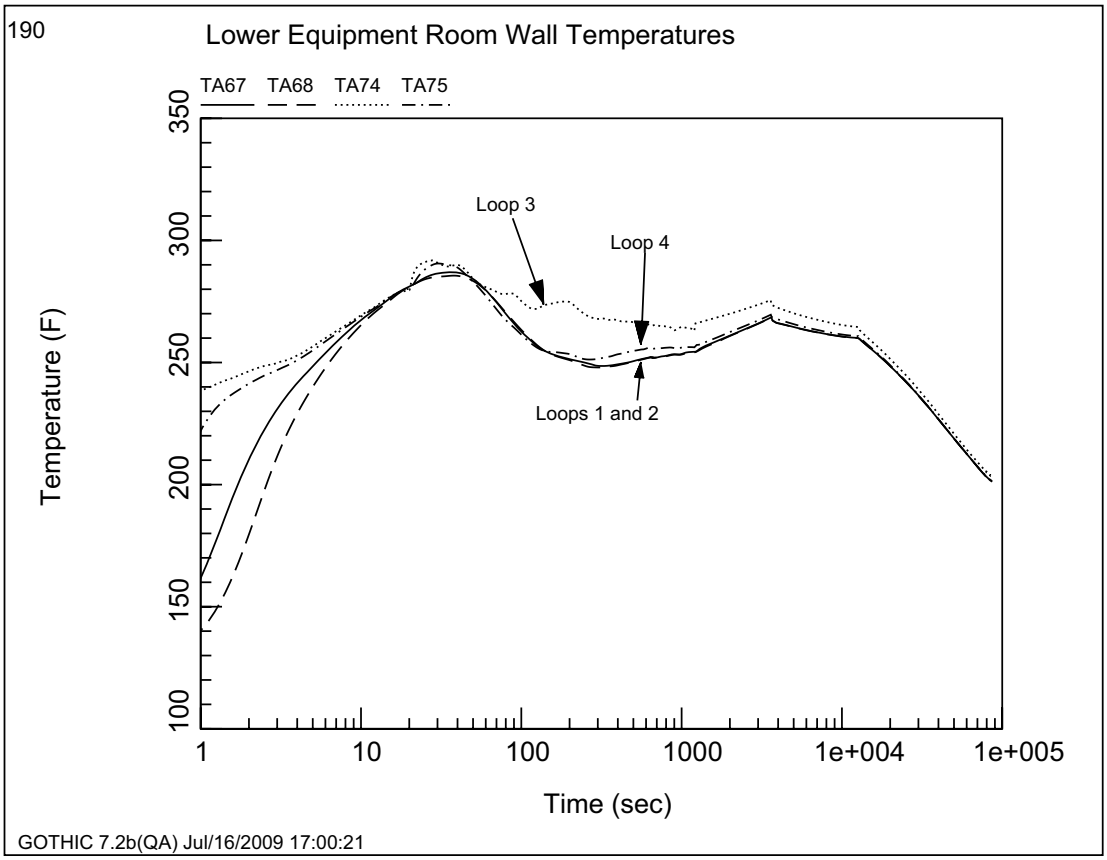


Figure 6.2.1-07d-127—Lower Equipment Room Structural Steel Surface Temperatures

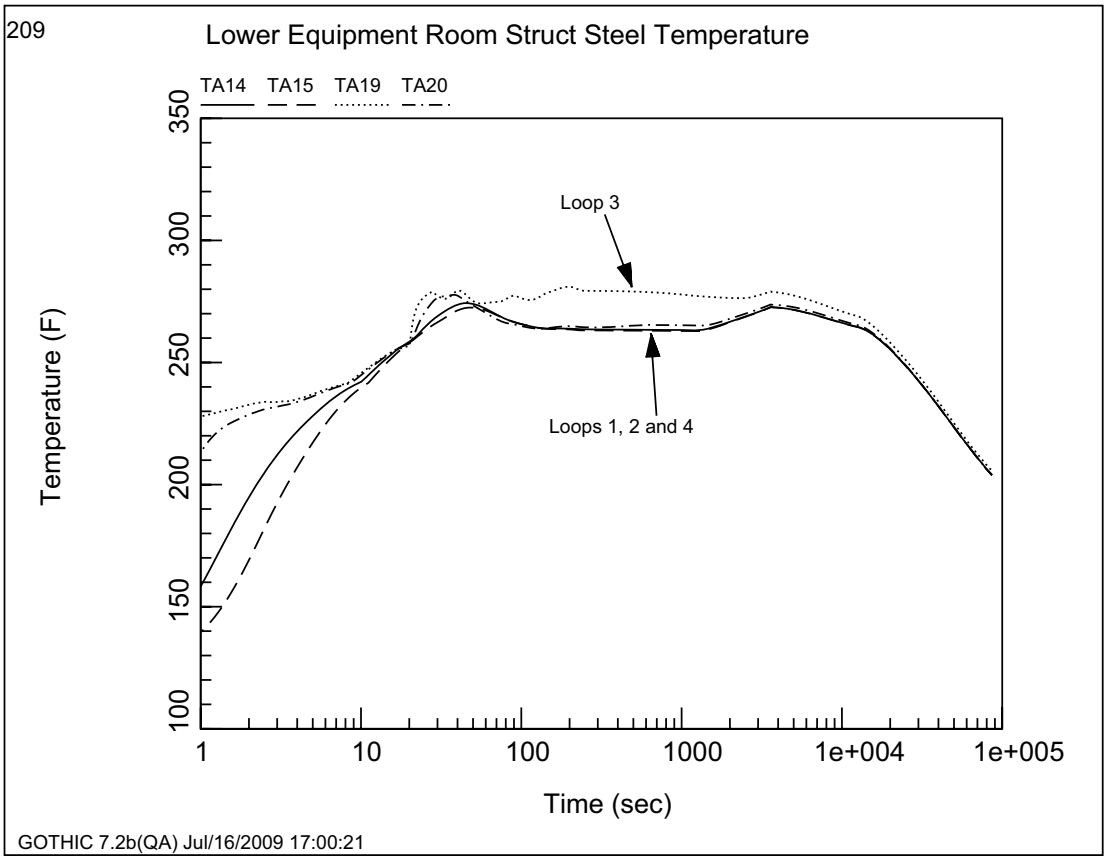


Figure 6.2.1-07d-128—Middle Equipment Room Wall Surface Temperatures

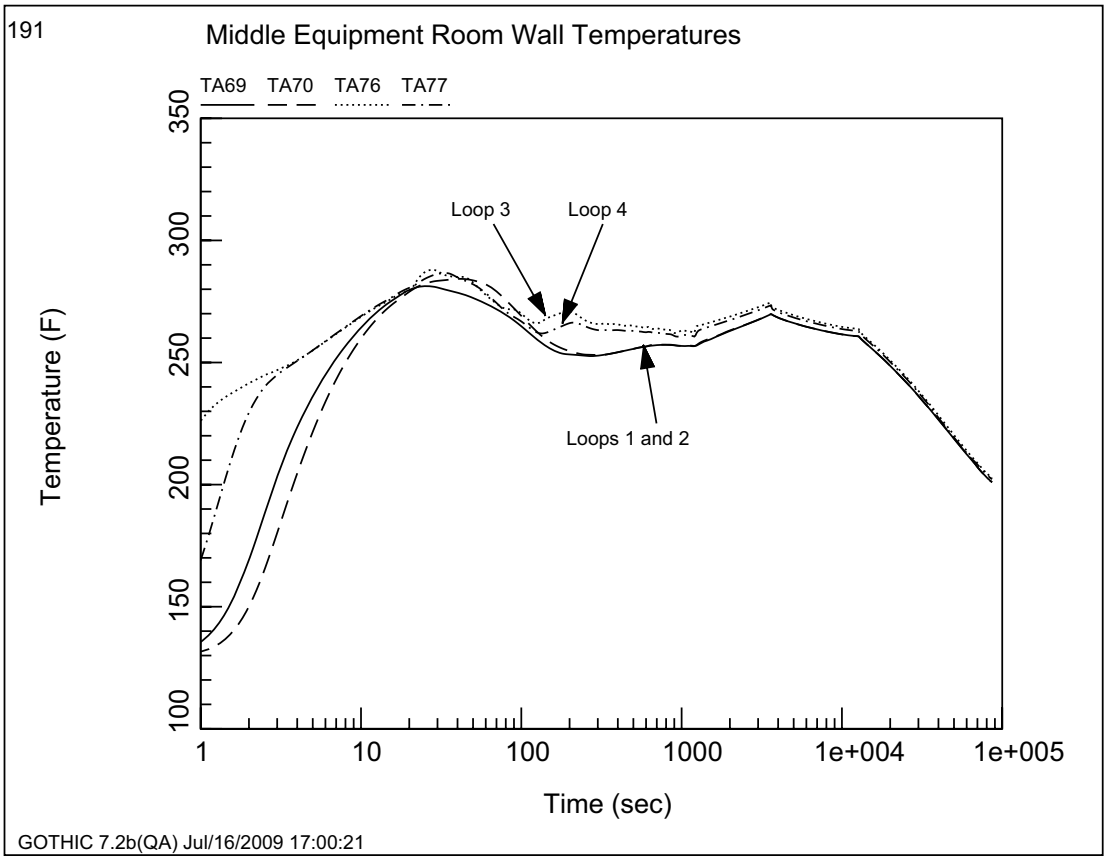


Figure 6.2.1-07d-129—Middle Equipment Room Structural Steel Surface Temperatures

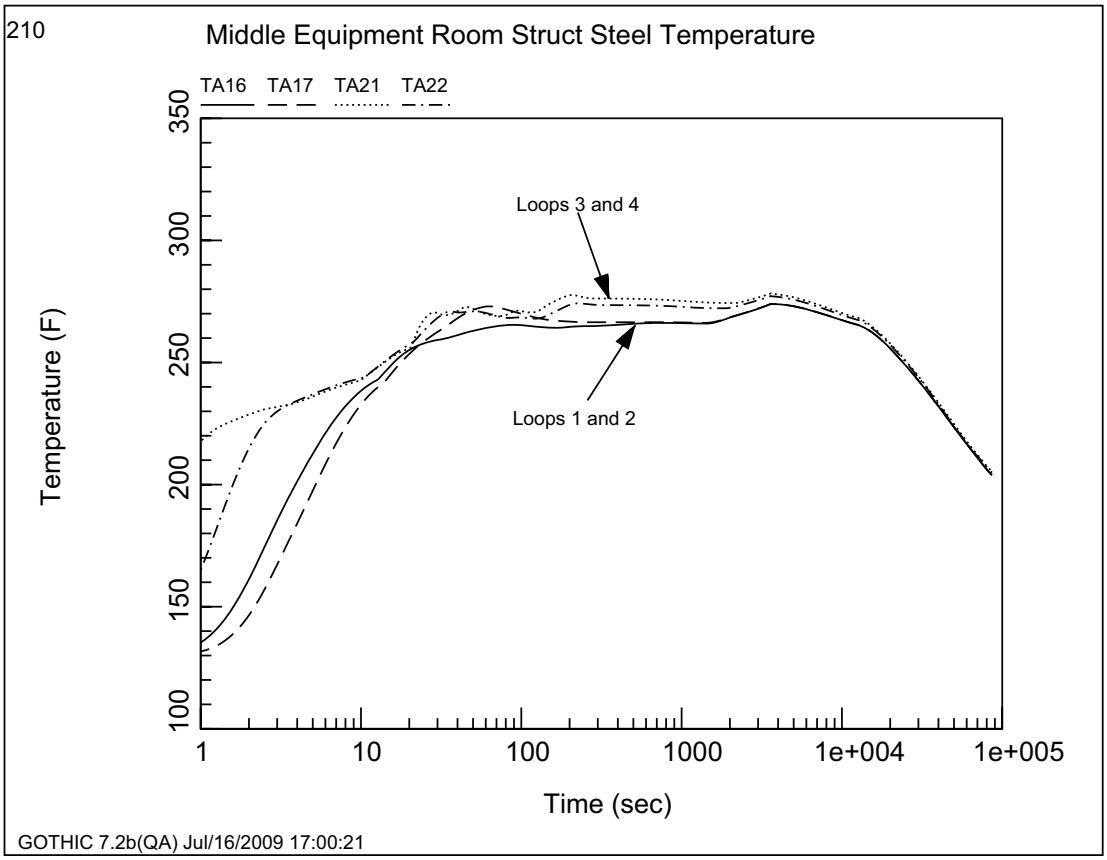


Figure 6.2.1-07d-130—Upper Equipment Room Wall Surface Temperatures

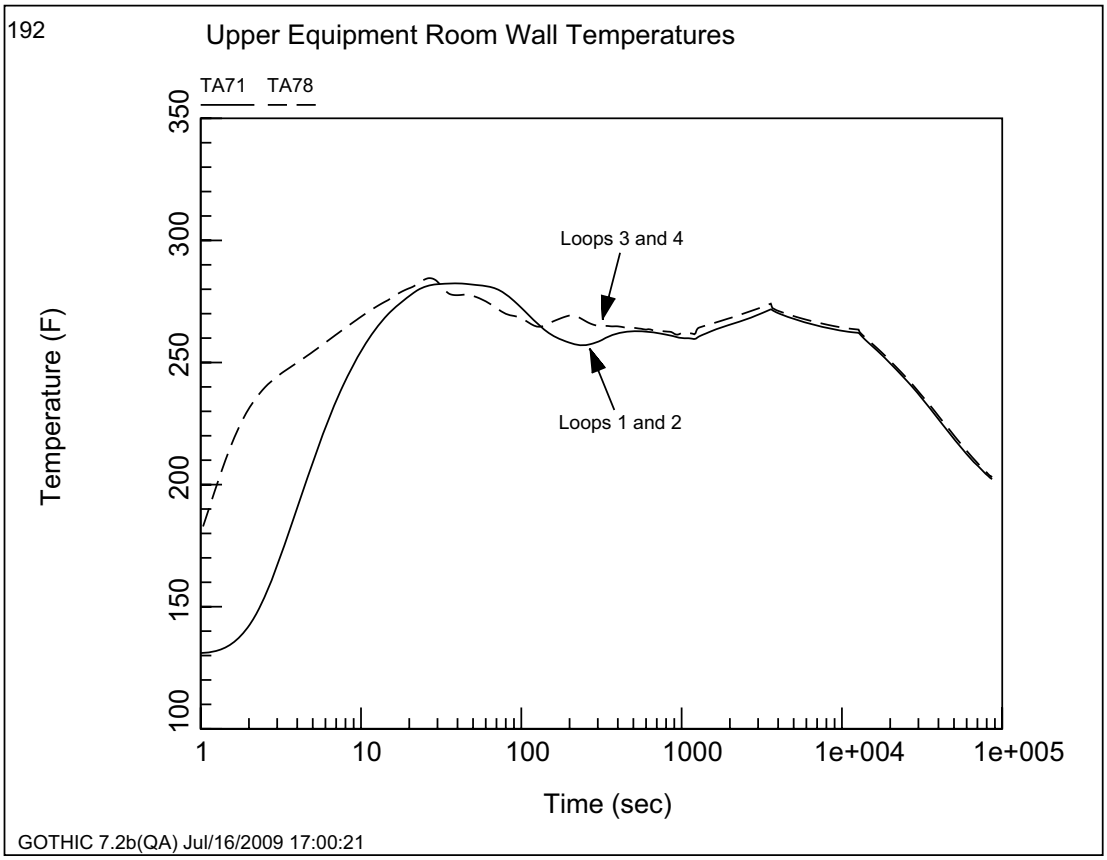


Figure 6.2.1-07d-131—Upper Equipment Room Structural Steel Surface Temperatures

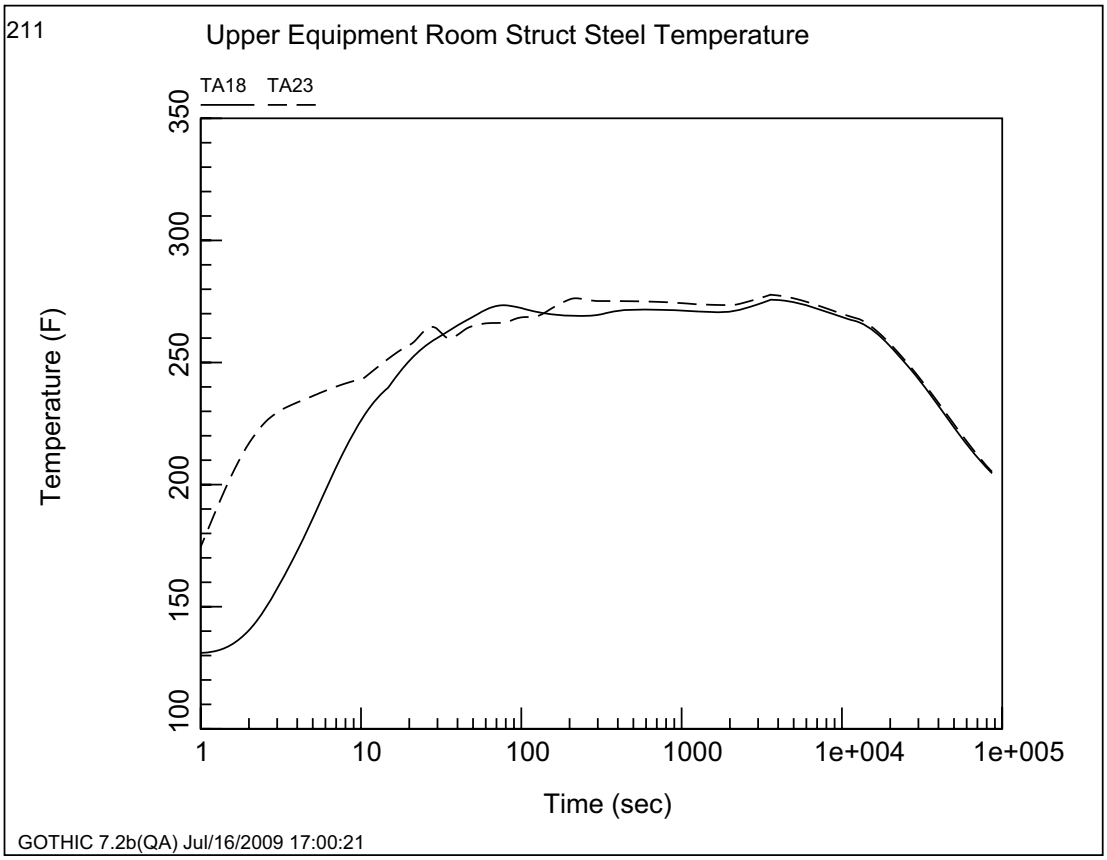


Figure 6.2.1-07d-132—Reactor Pressure Vessel Pit and Reactor Cavity Wall Surface Temperatures

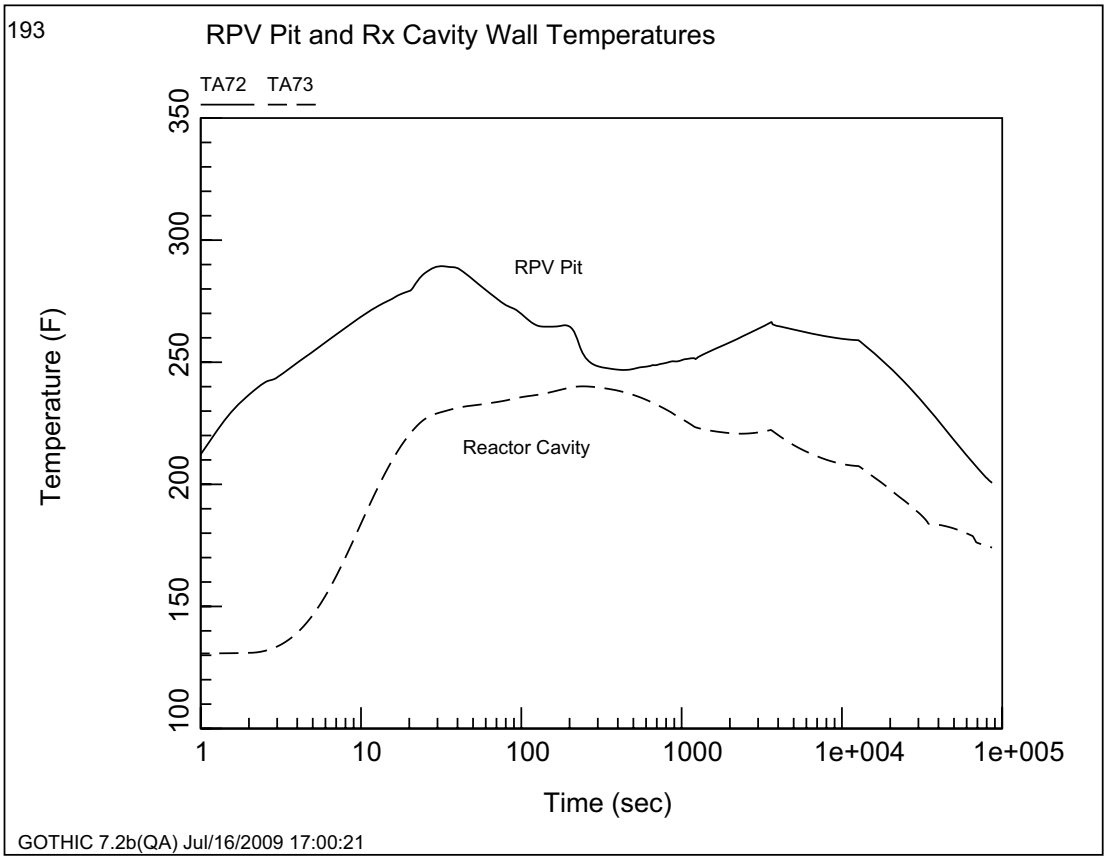


Figure 6.2.1-07d-133—Pressurizer Room and Surge Line Room Wall Surface Temperatures

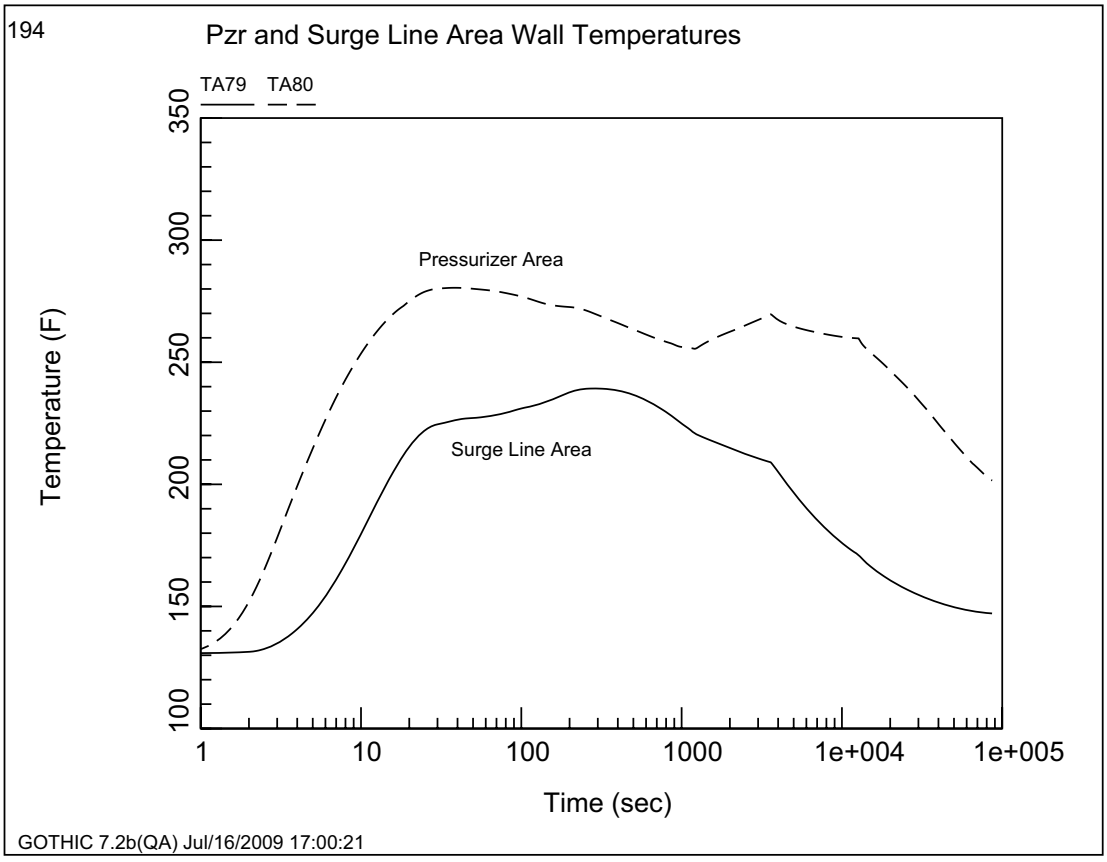
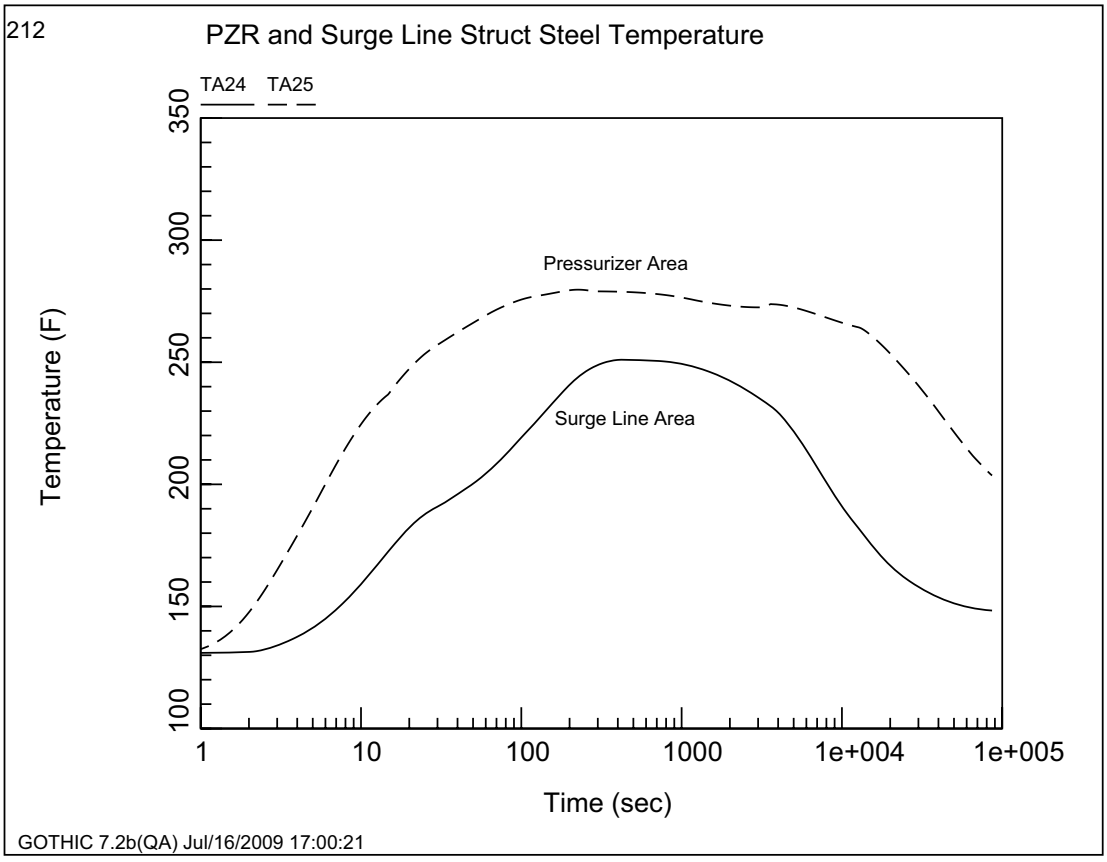


Figure 6.2.1-07d-134—Pressurizer Room and Surge Line Room Structural Steel Surface Temperatures



Accessible Area

Figure 6.2.1-07d-135—Containment Wall Surface Temperatures in the Loop 1 and 2 Lower, Middle and Upper Annulus Rooms

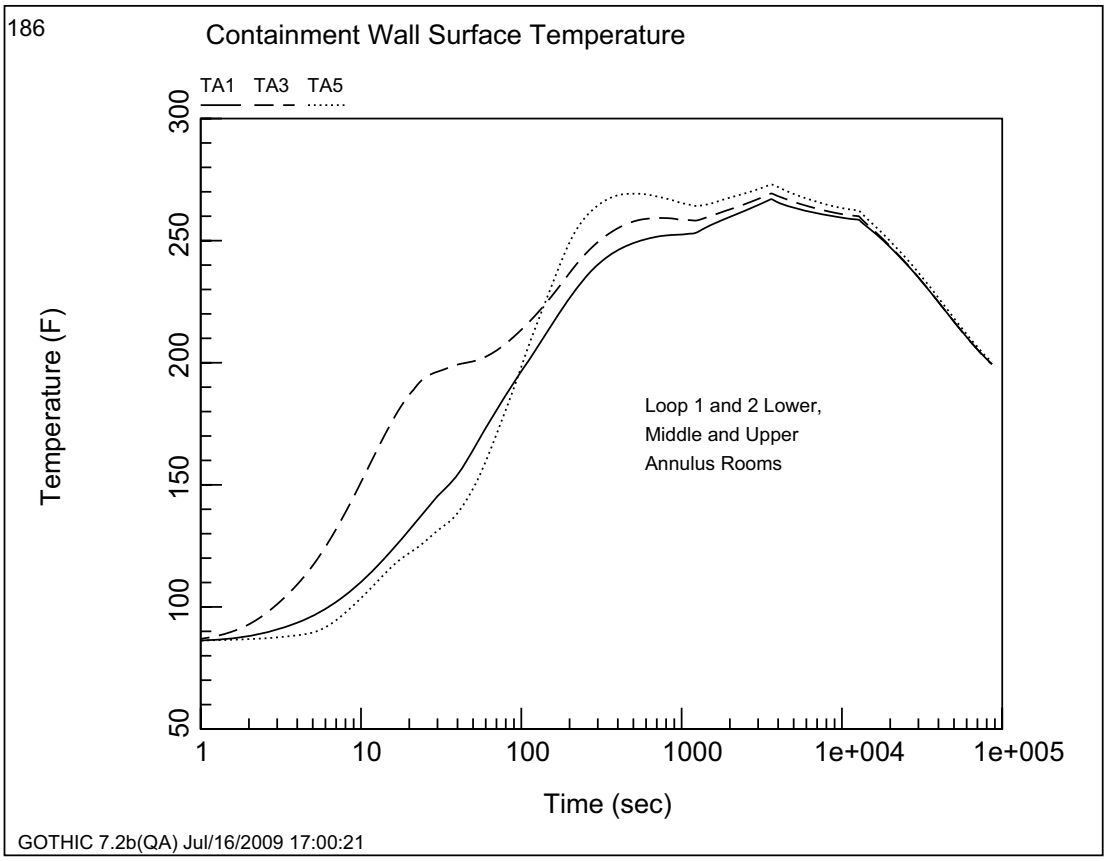


Figure 6.2.1-07d-136—Containment Wall Surface Temperatures in the Loop 3 and 4 Lower, Middle and Upper Annulus Rooms

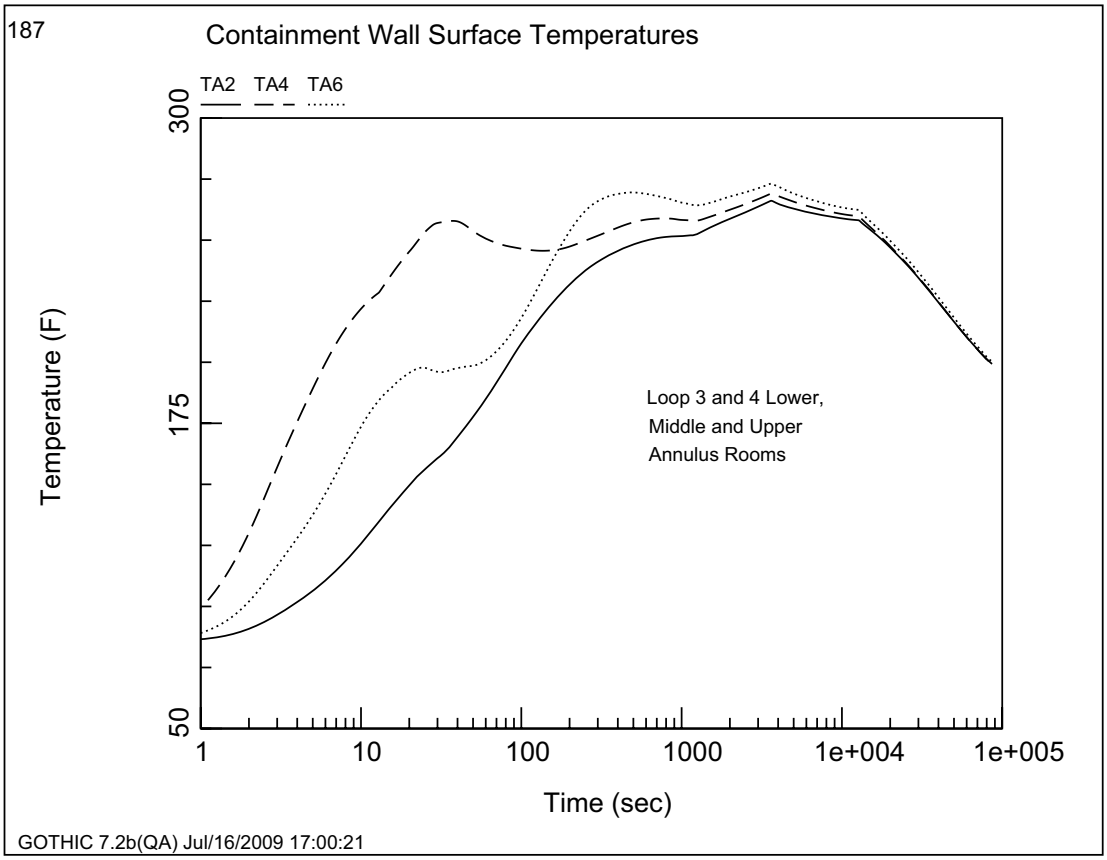


Figure 6.2.1-07d-137—Containment Wall Surface Temperature in the Access Room, South Staircase and Hot Piping Penetration Room

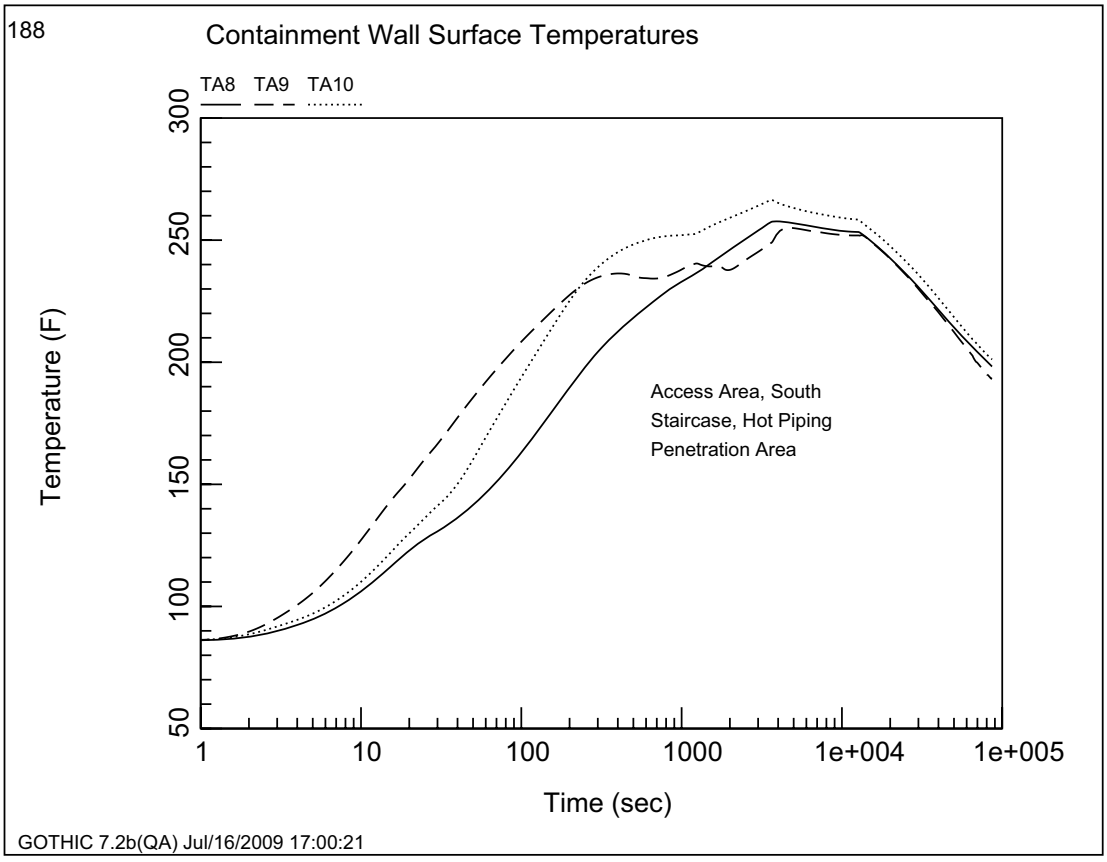


Figure 6.2.1-07d-138—CVCS Room and Steam Generator Blowdown Heat Exchanger Room Wall Surface Temperatures

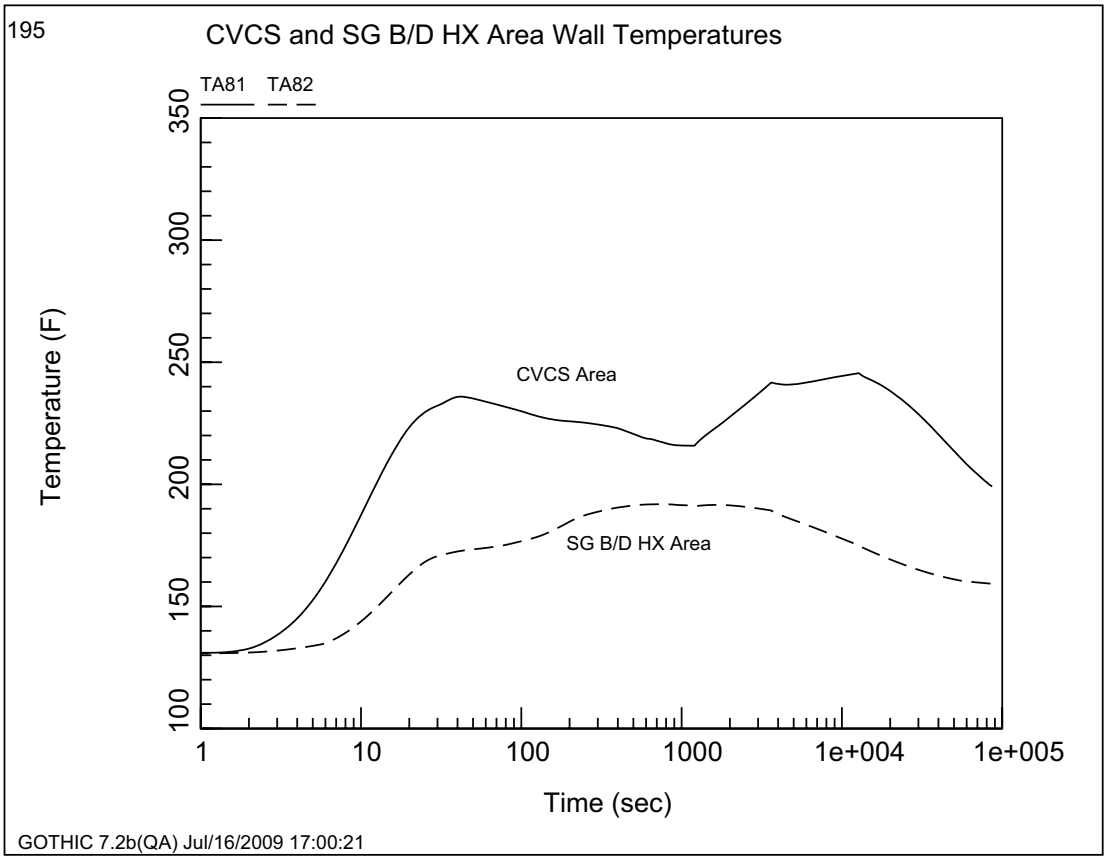


Figure 6.2.1-07d-139—CVCS Room and Steam Generator Blowdown Heat Exchanger Room Structural Steel Surface Temperatures

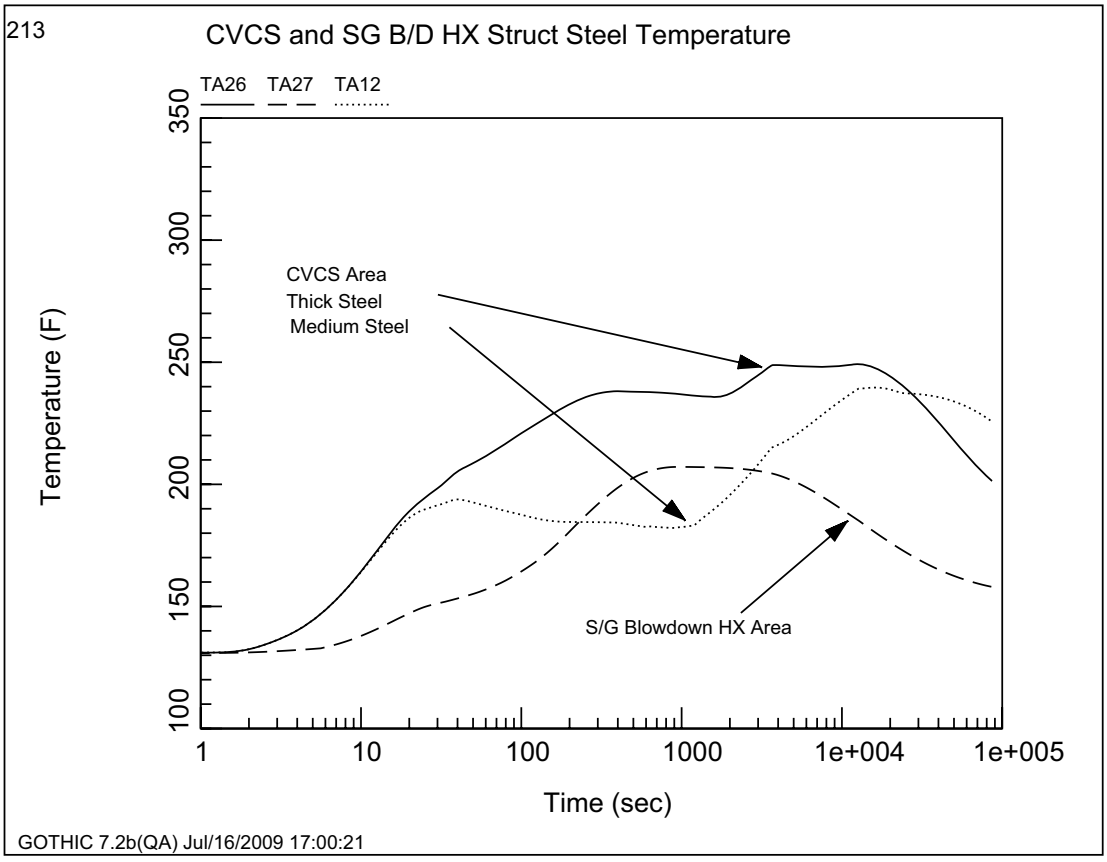


Figure 6.2.1-07d-140—Lower Annulus Room Wall Surface Temperatures

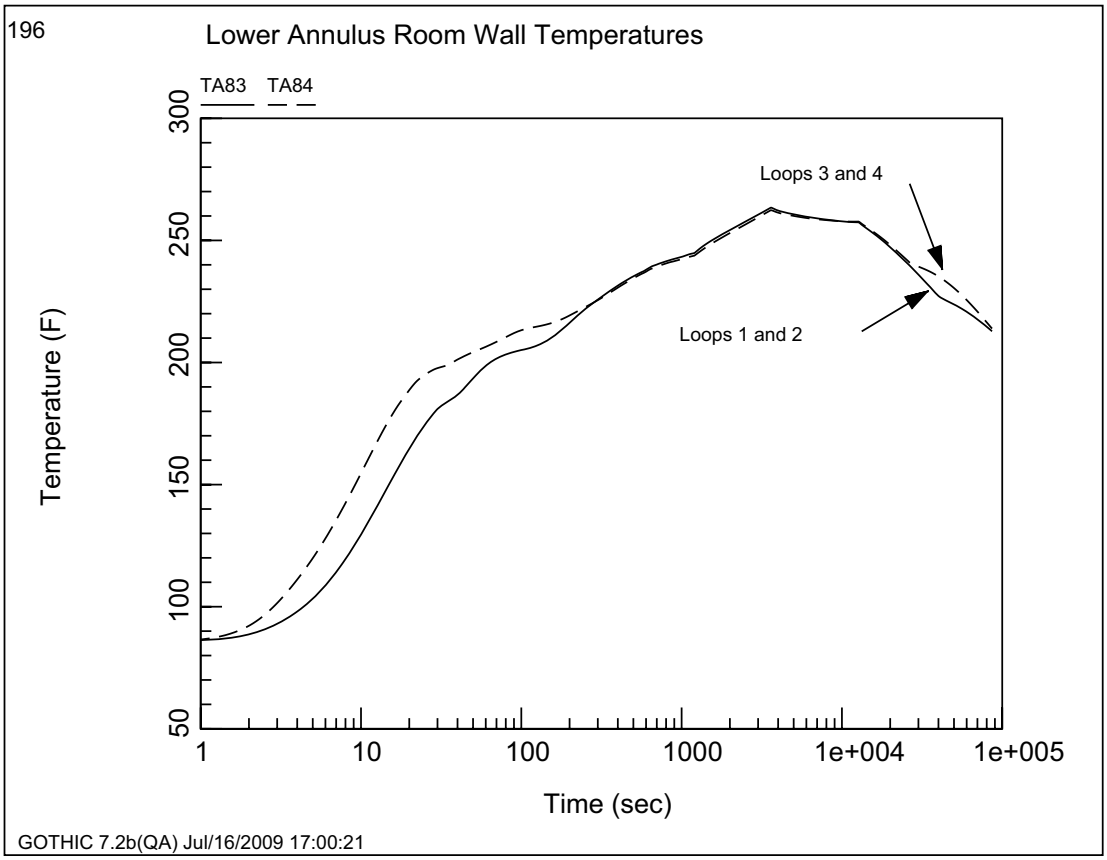


Figure 6.2.1-07d-141—Lower Annulus Room Structural Steel Surface Temperatures

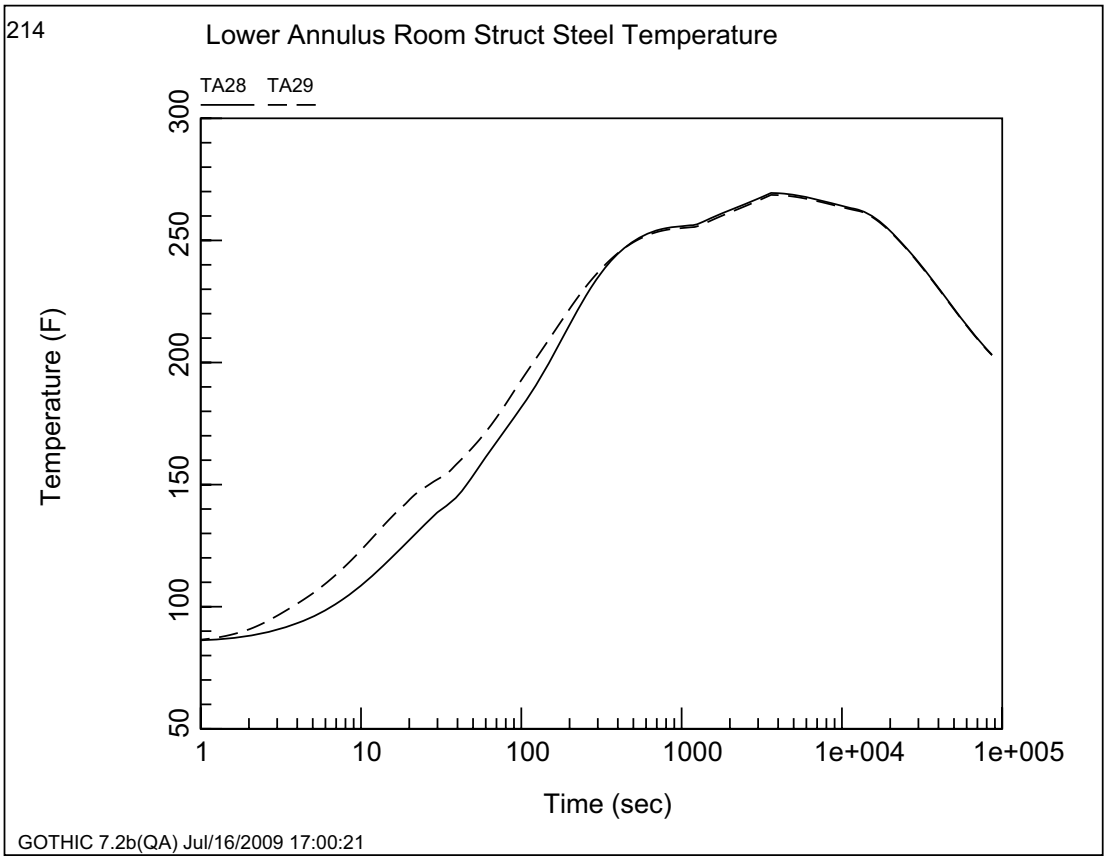


Figure 6.2.1-07d-142—Middle Annulus Room Wall Surface Temperatures

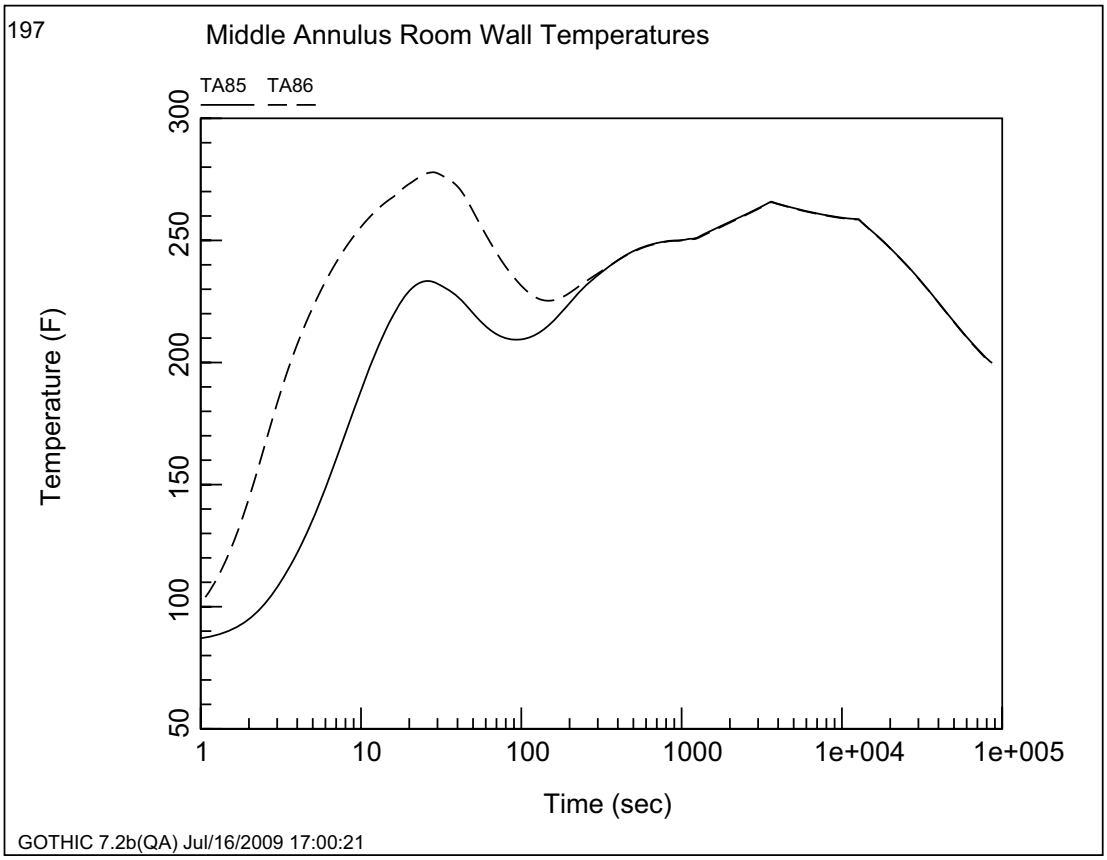


Figure 6.2.1-07d-143—Middle Annulus Room Structural Steel Surface Temperatures

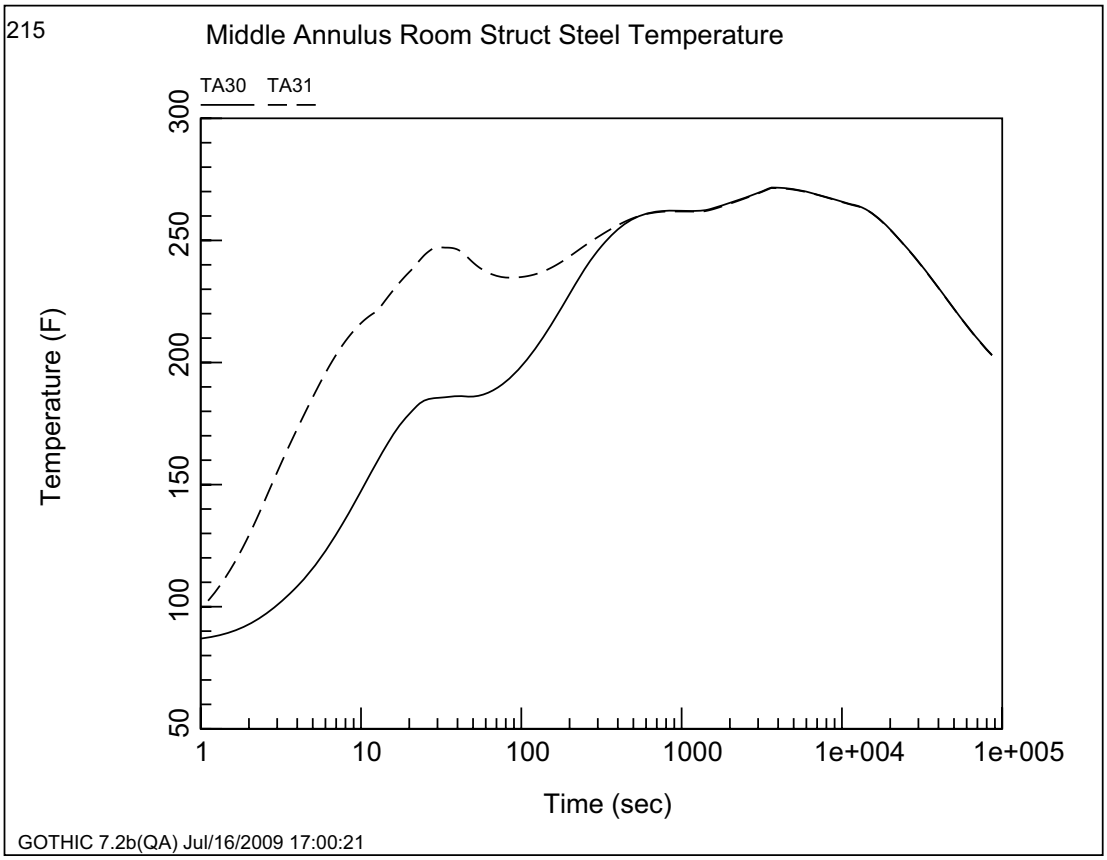


Figure 6.2.1-07d-144—Upper Annulus Room Wall Surface Temperatures

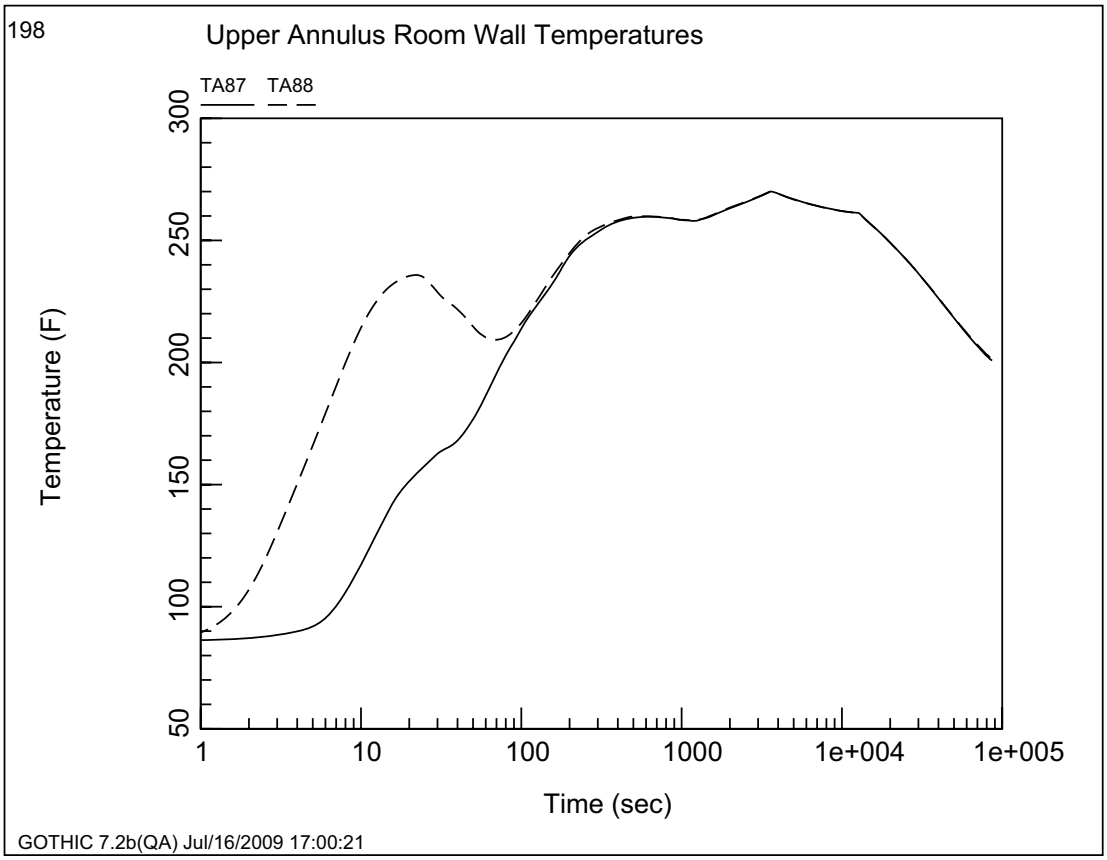


Figure 6.2.1-07d-145—Upper Annulus Room Structural Steel Surface Temperatures

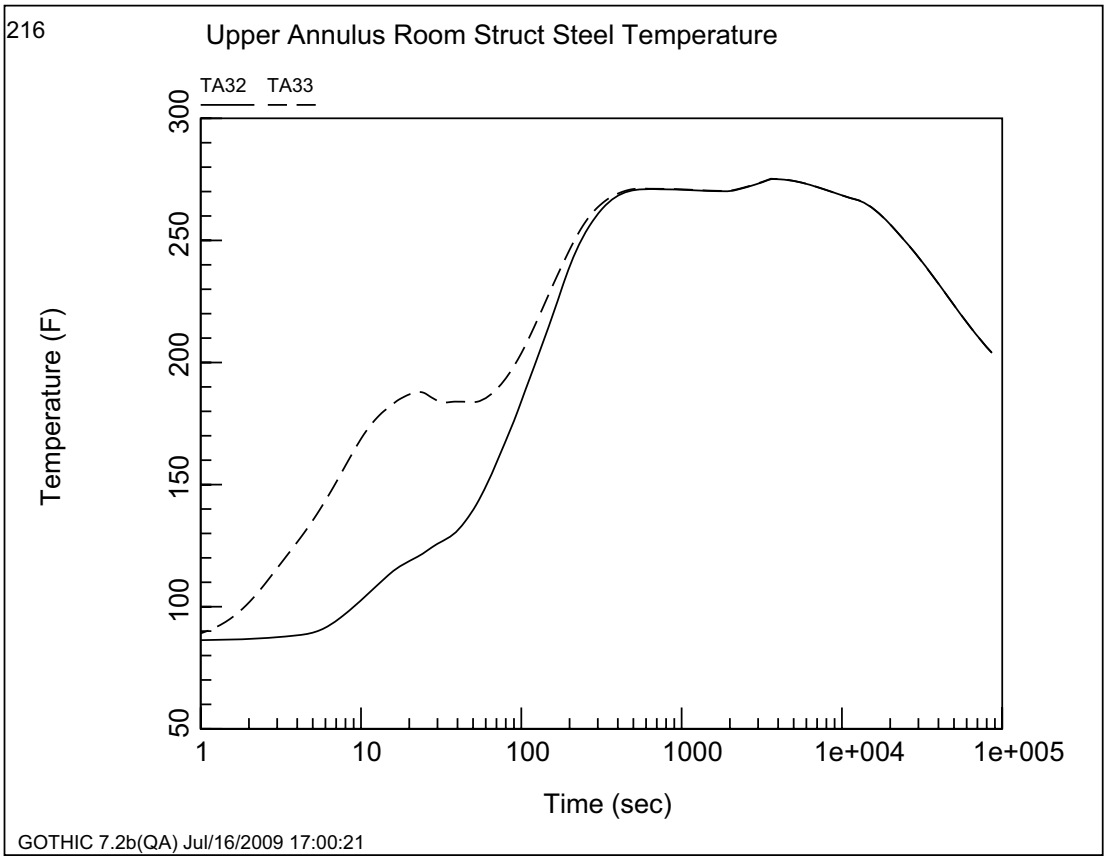


Figure 6.2.1-07d-146—Containment Dome Interior Wall Surface Temperatures from the 63.98 ft to the 71.53 ft Elevation

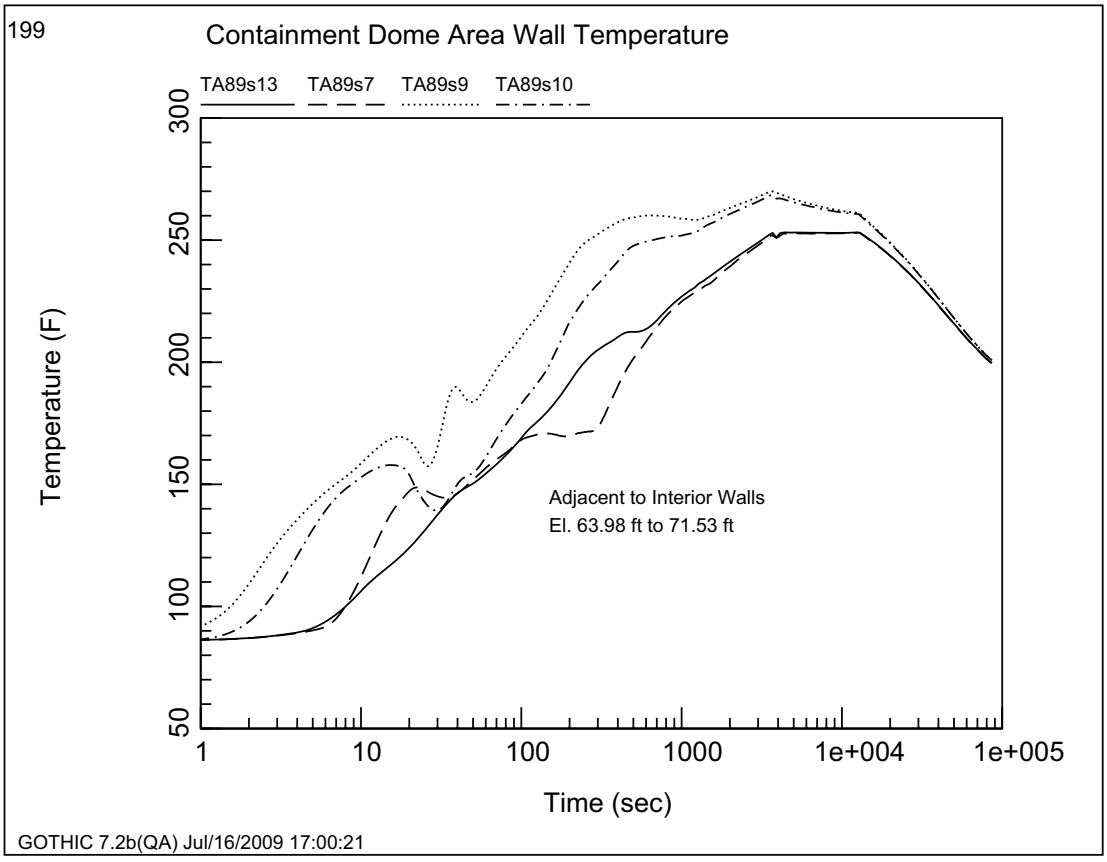


Figure 6.2.1-07d-147—Containment Dome Structural Steel Surface Temperatures from the 63.98 ft to the 71.53 ft Elevation

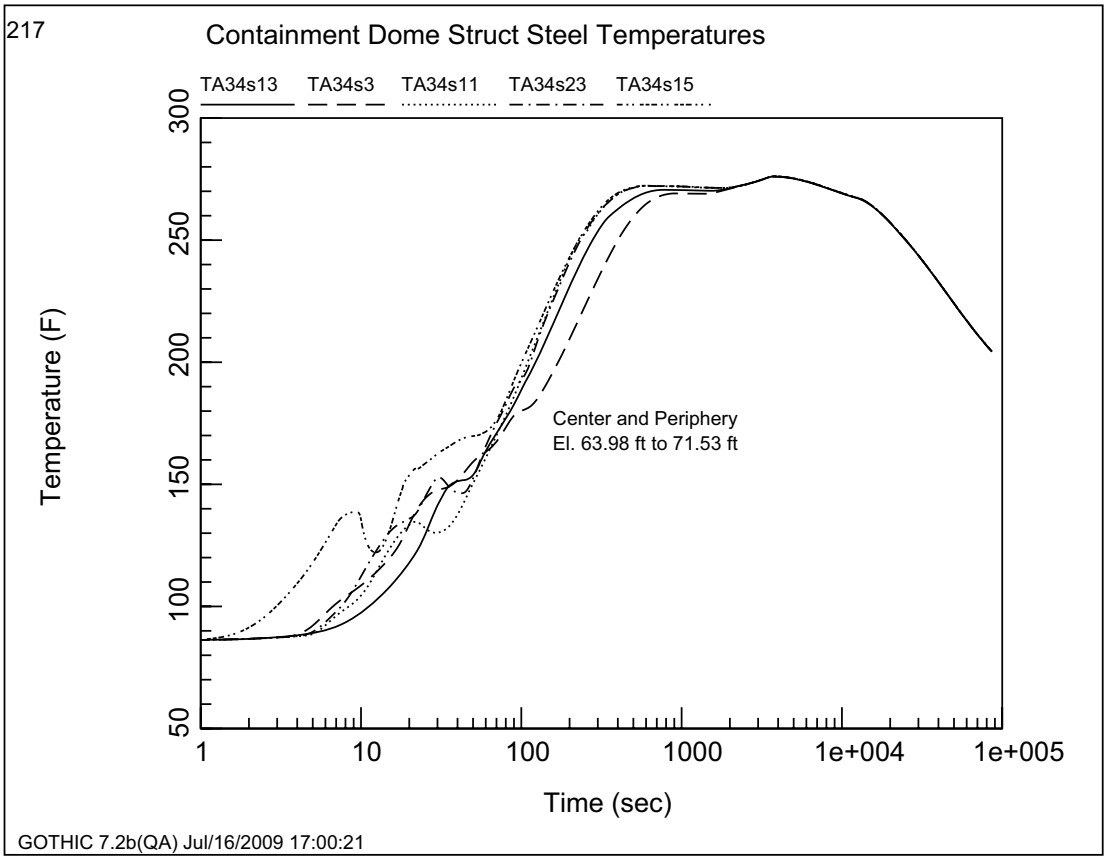


Figure 6.2.1-07d-148—Containment Dome Interior Wall Surface Temperatures from the 79.07 ft to the 86.29 ft Elevation

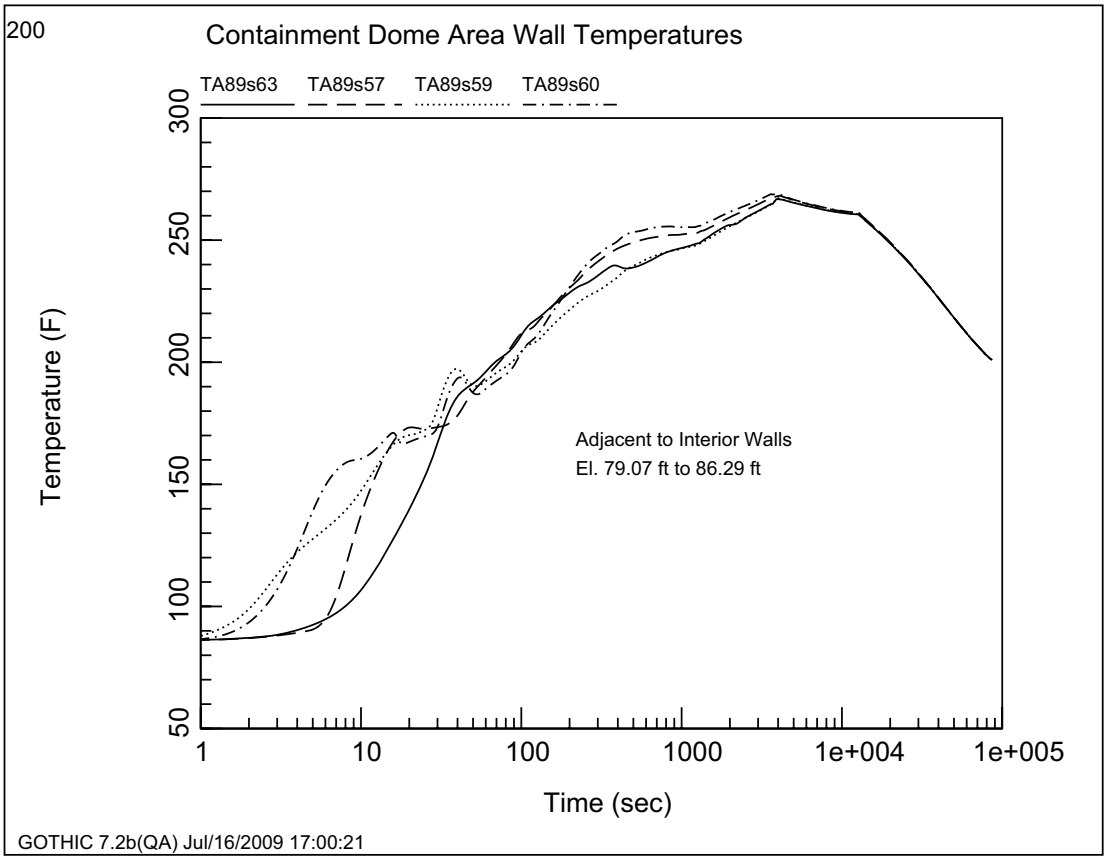


Figure 6.2.1-07d-149—Containment Dome Structural Steel Surface Temperatures from the 79.07 ft to the 86.29 ft Elevation

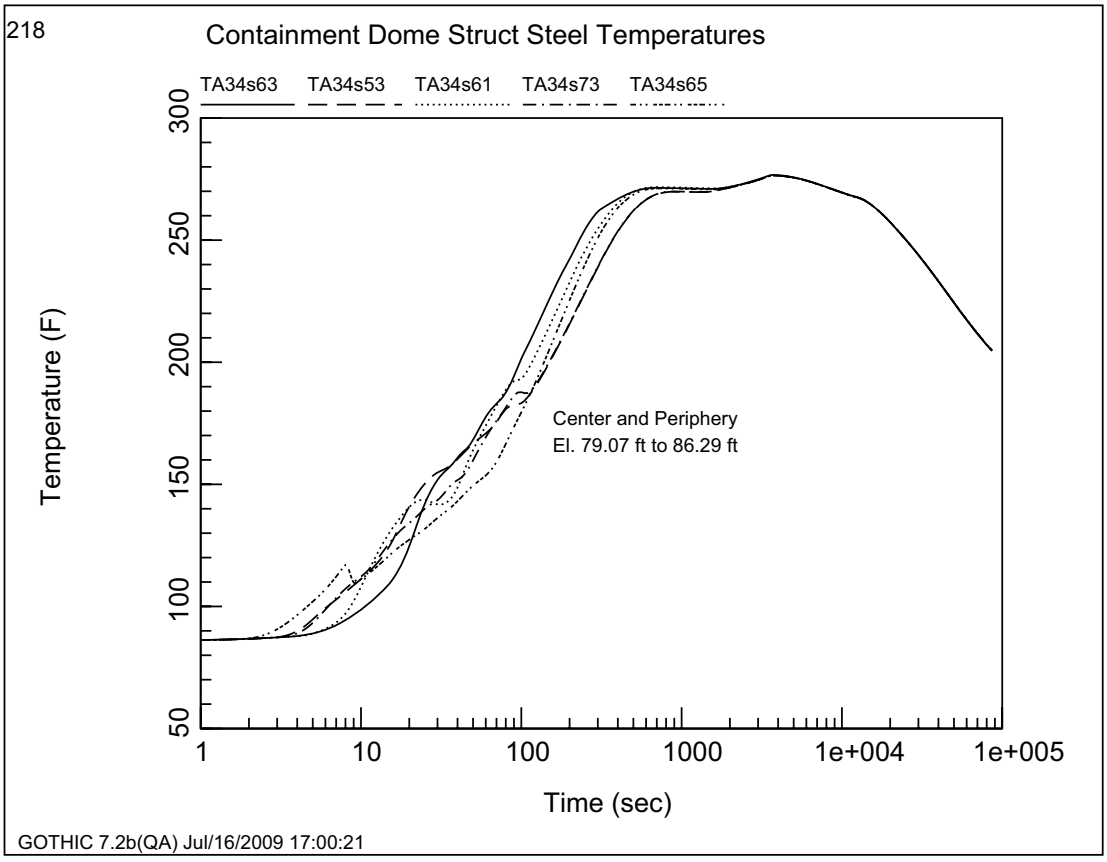


Figure 6.2.1-07d-150—Containment Dome Interior Wall Surface Temperatures from the 93.51 ft to the 98.59 ft Elevation

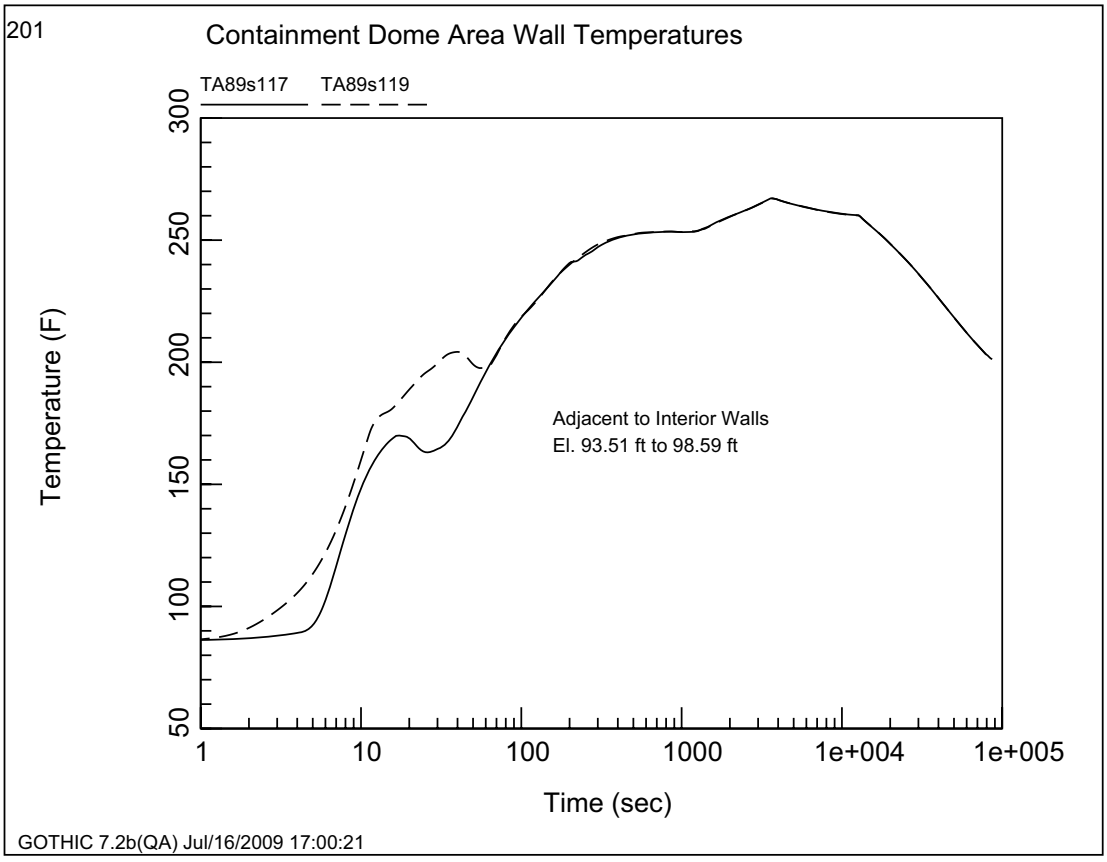


Figure 6.2.1-07d-151—Containment Dome Structural Steel Surface Temperatures from the 93.51 ft to the 98.59 ft Elevation

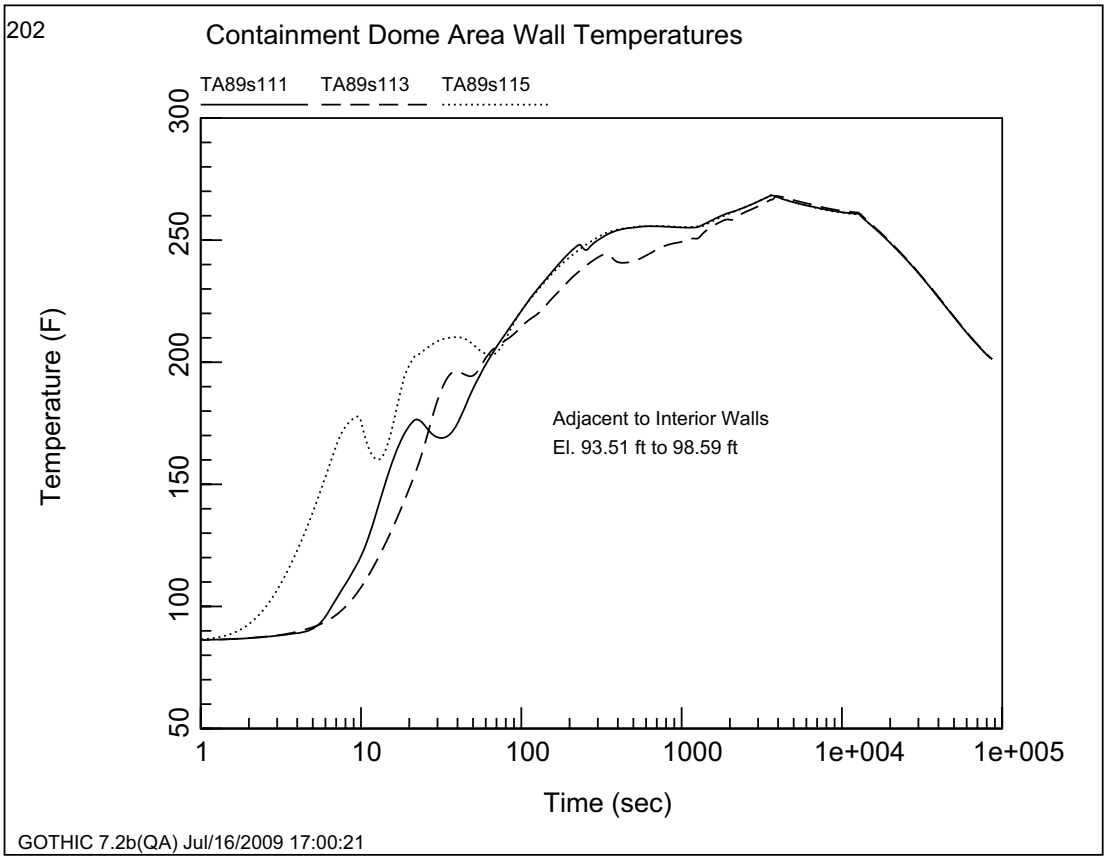


Figure 6.2.1-07d-152—Containment Dome Interior Wall Surface Temperatures from the 93.51 ft to the 98.59 ft Elevation

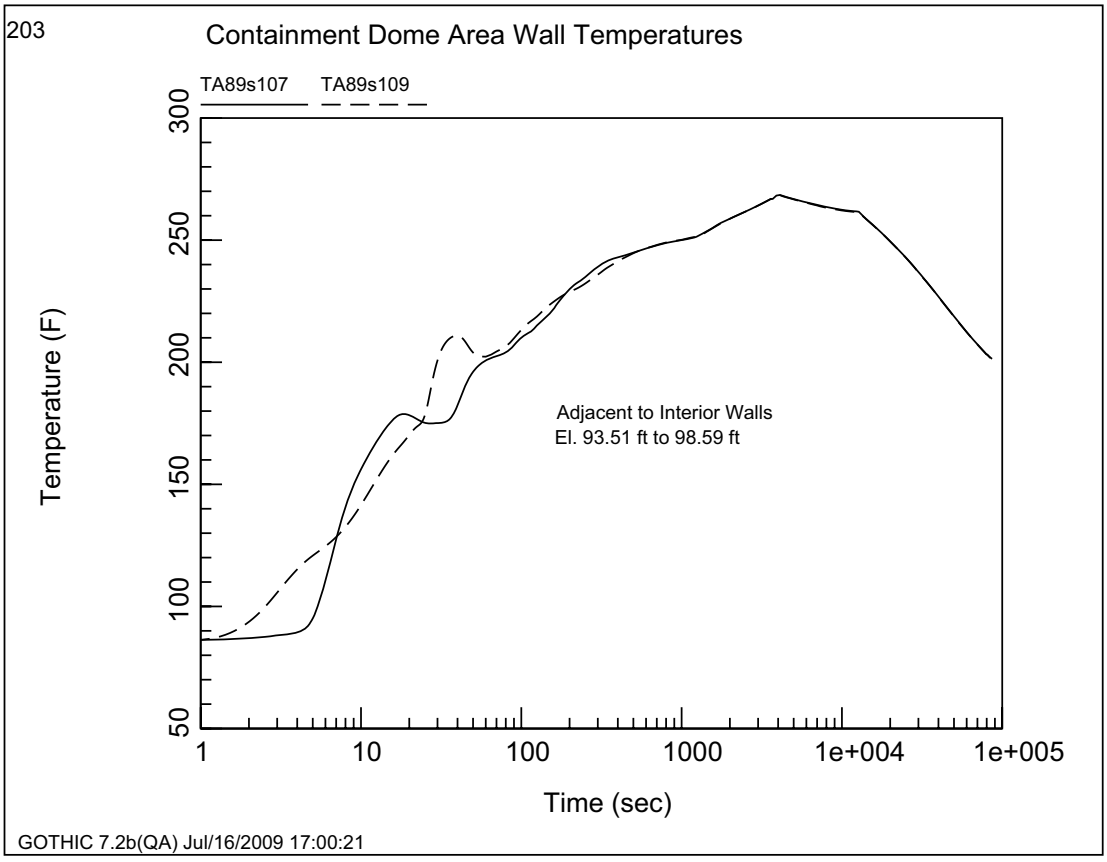


Figure 6.2.1-07d-153—Containment Dome Structural Steel Surface Temperatures from the 93.51 ft to the 98.59 ft Elevation

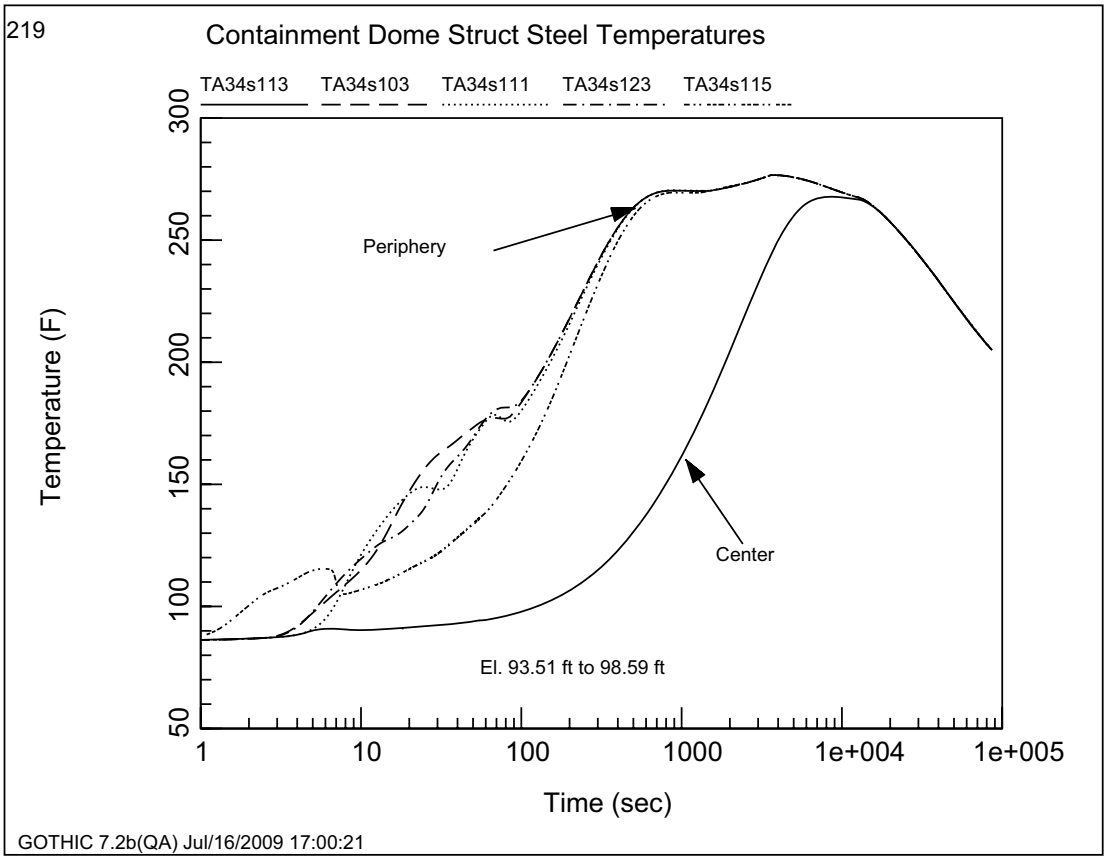


Figure 6.2.1-07d-154—Containment Dome Interior Wall Surface Temperatures from the 103.68 ft to the 108.76 ft Elevation

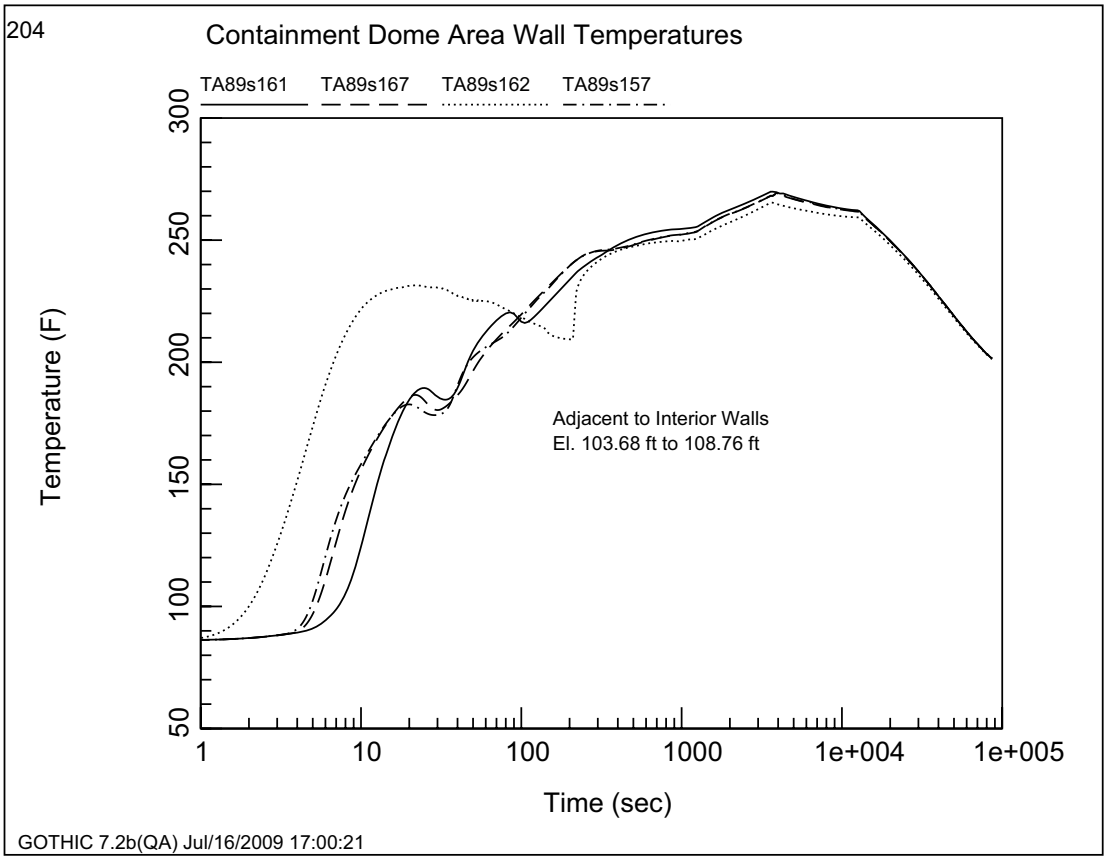


Figure 6.2.1-07d-155—Containment Dome Interior Wall Surface Temperatures from the 103.68 ft to the 108.76 ft Elevation

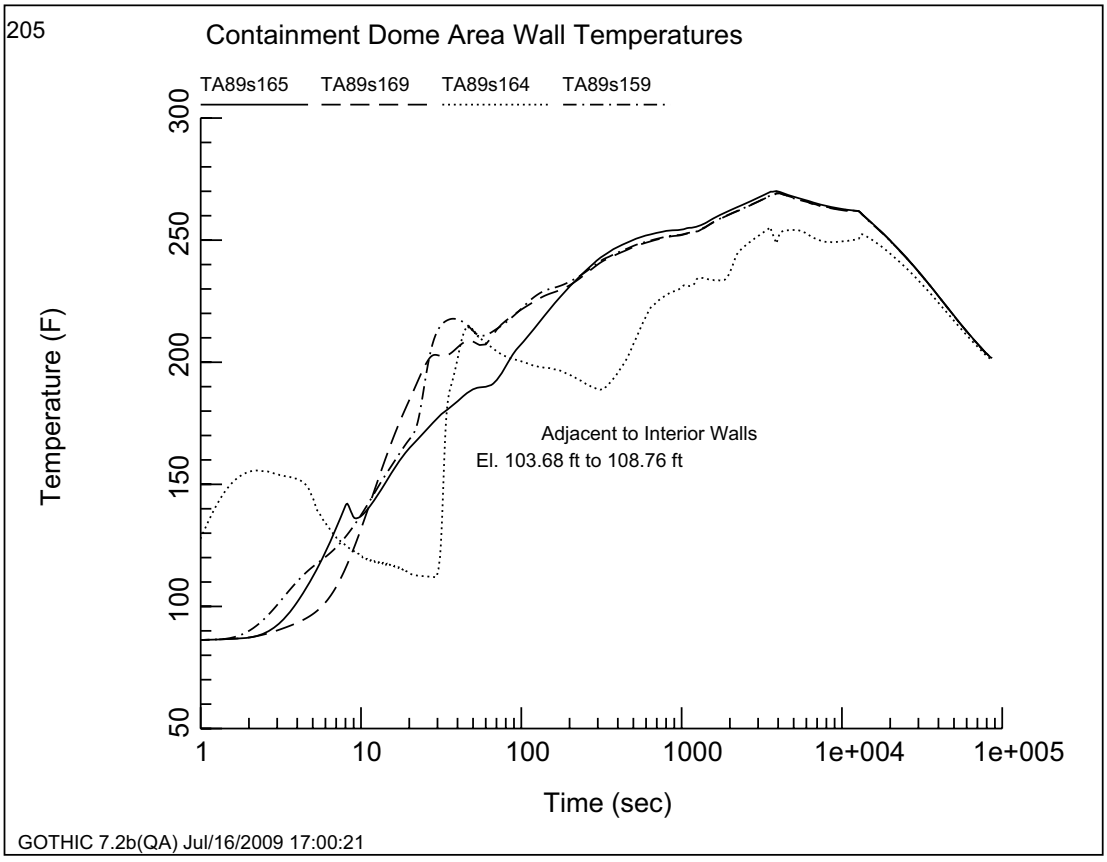


Figure 6.2.1-07d-156—Containment Dome Structural Steel Surface Temperatures from the 103.68 ft to the 108.76 ft Elevation

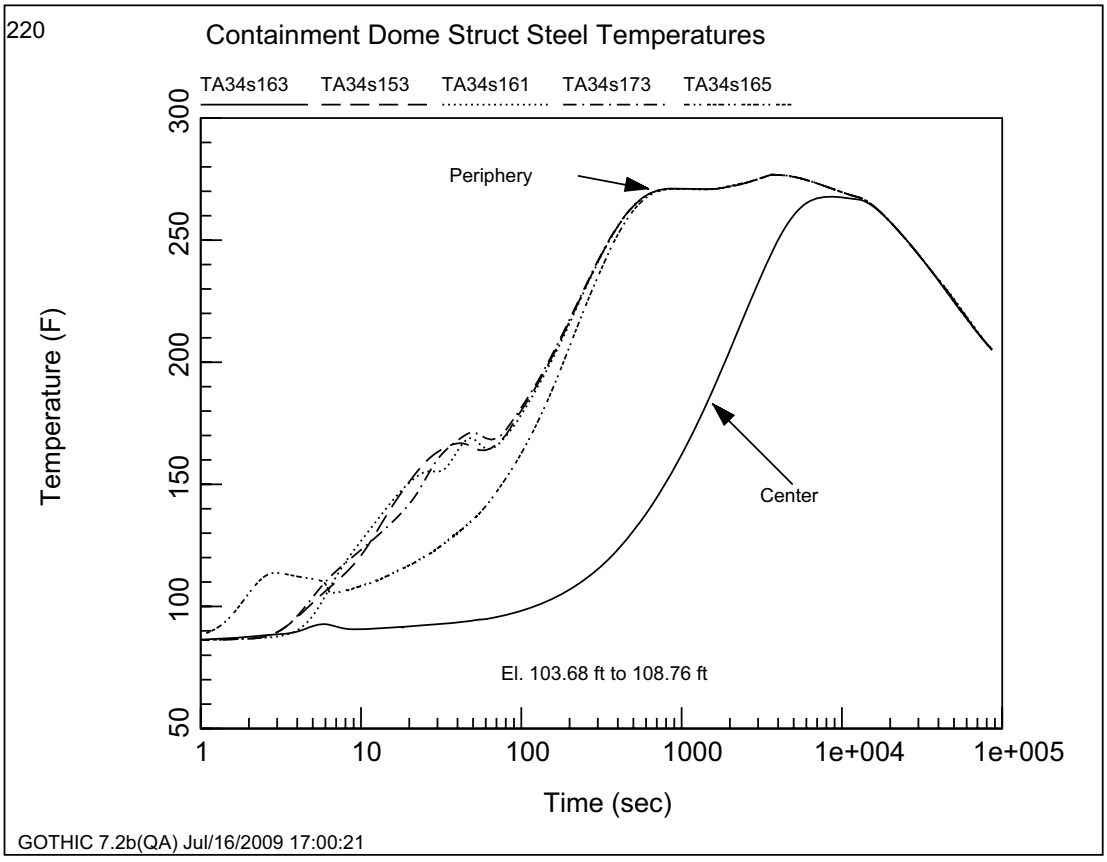


Figure 6.2.1-07d-157—Containment Dome Structural Steel Surface Temperatures from the 113.85 ft to the 119.90 ft Elevation

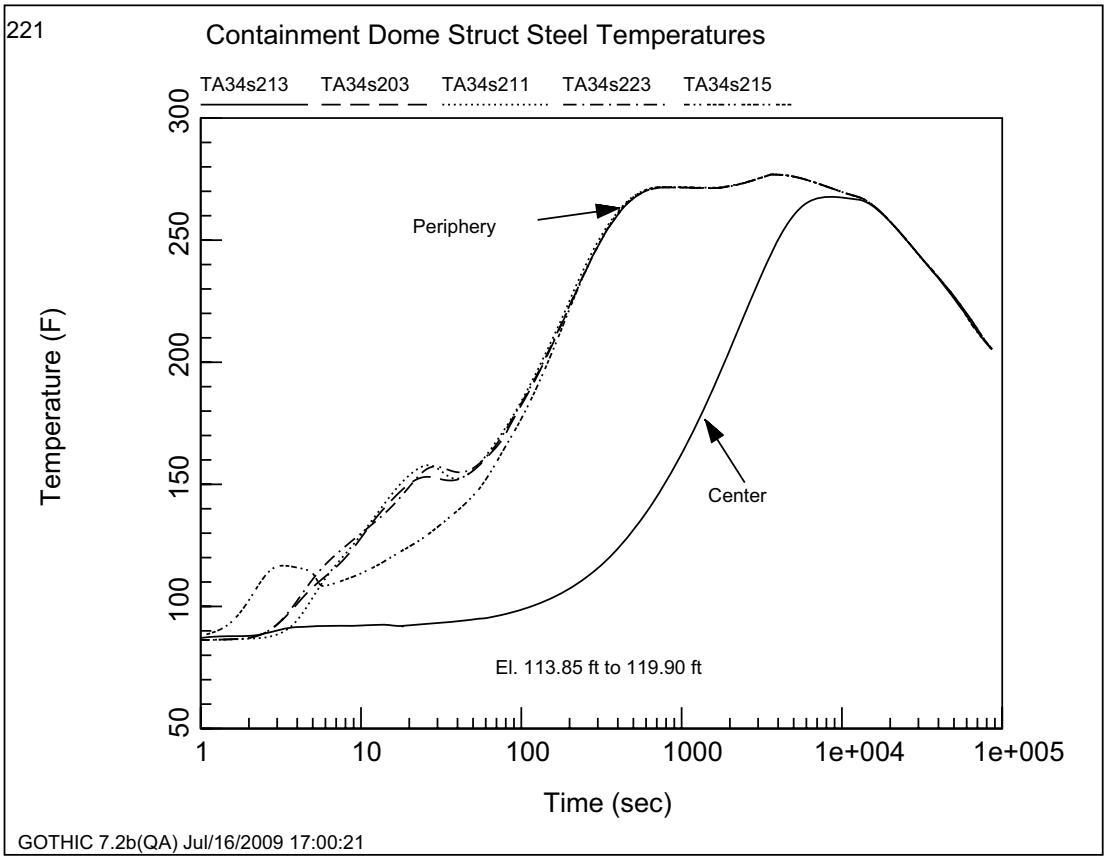


Figure 6.2.1-07d-158—Wall Surface Temperatures in the Access Room and the Hot Piping Penetration Room

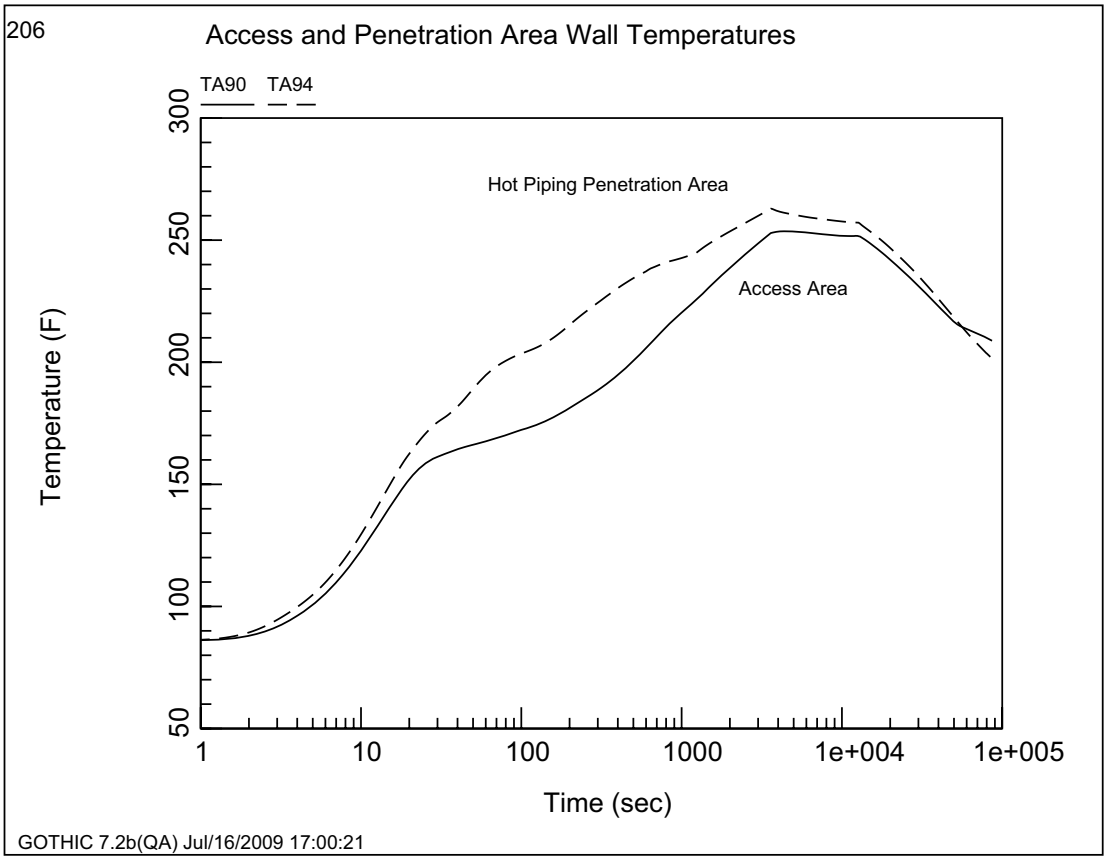


Figure 6.2.1-07d-159—Structural Steel Surface Temperatures in the Access Room and the Hot Piping Penetration Room

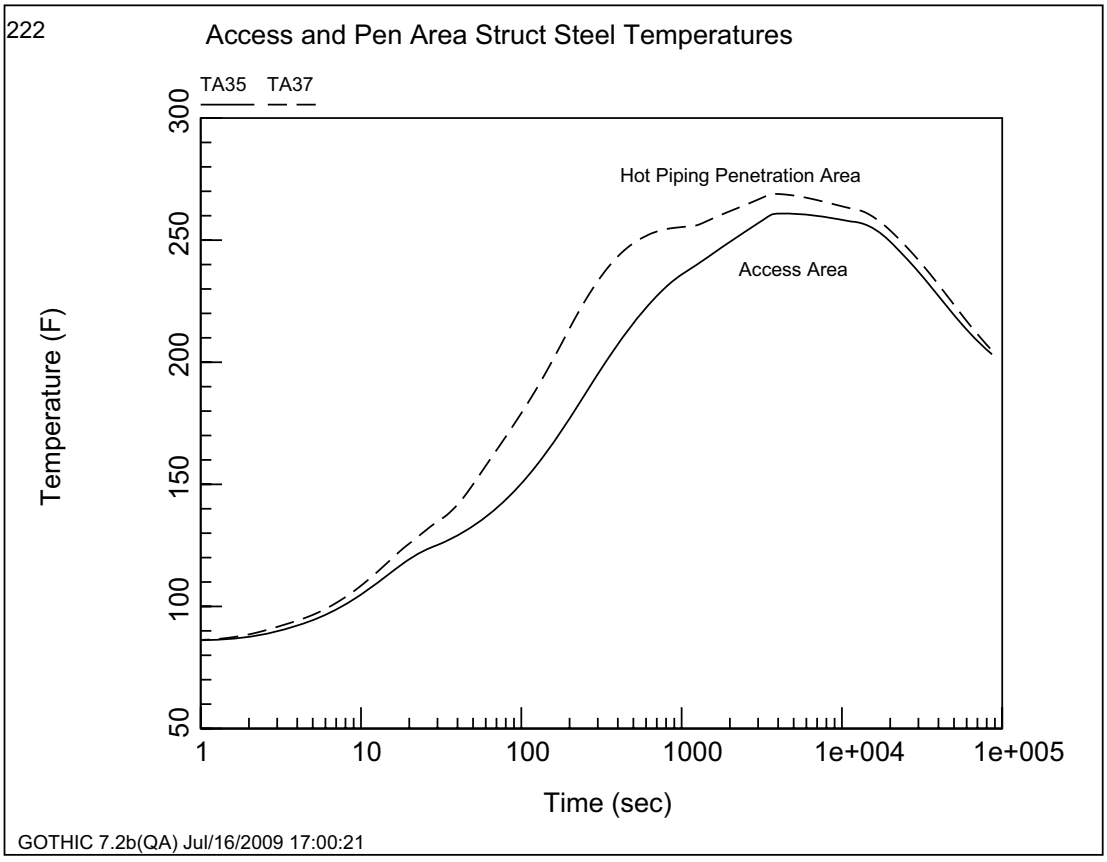


Figure 6.2.1-07d-160—Wall Surface Temperatures in the North and South Staircases and the Elevator

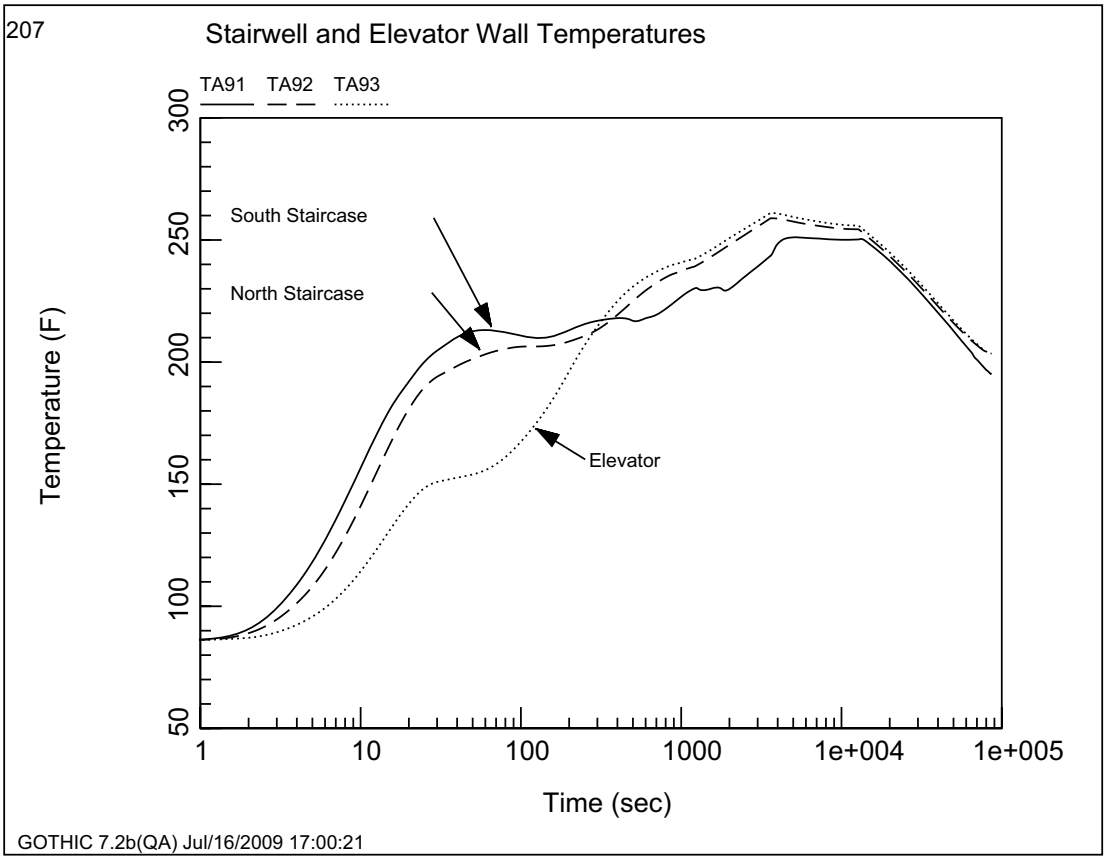
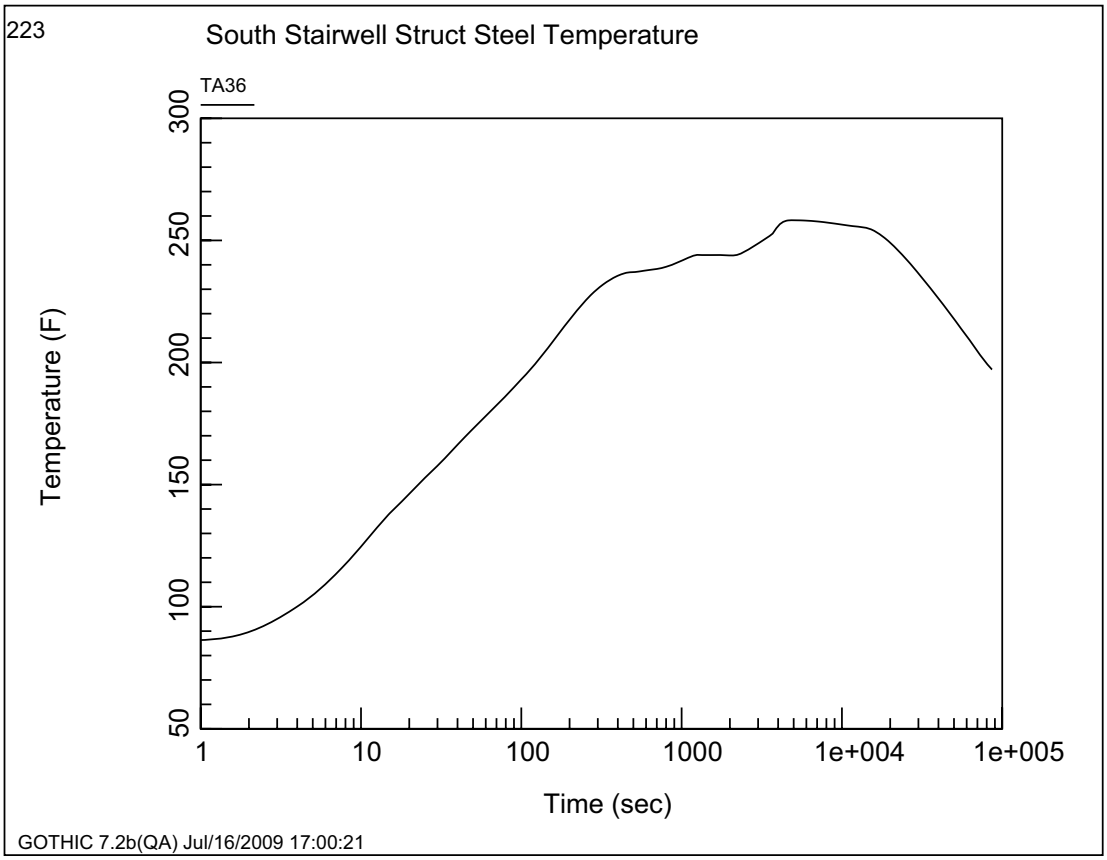


Figure 6.2.1-07d-161—Structural Steel Surface Temperatures in the South Staircase



Containment Heat Conductor Heat Rates

Equipment Area

Figure 6.2.1-07d-162—IRWST Ceiling, Wall and Basemat Heat Rates

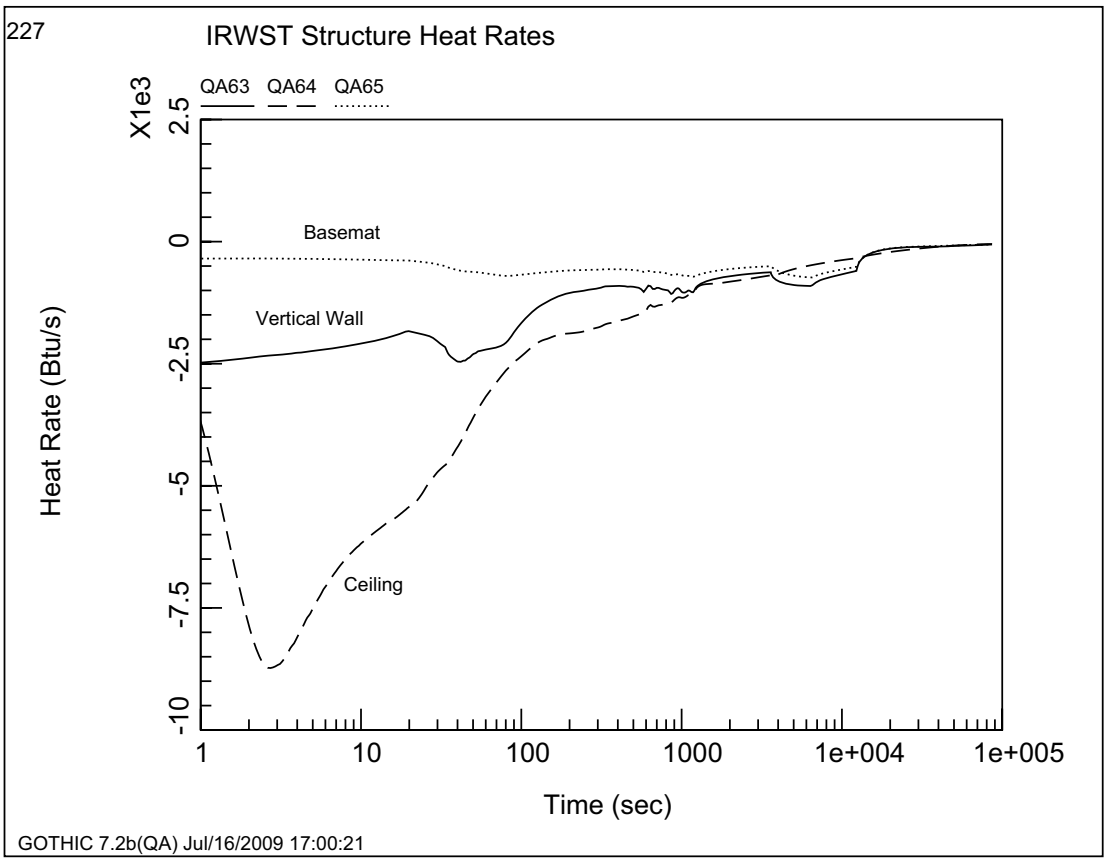


Figure 6.2.1-07d-163—IRWST Structural Steel Heat Rate

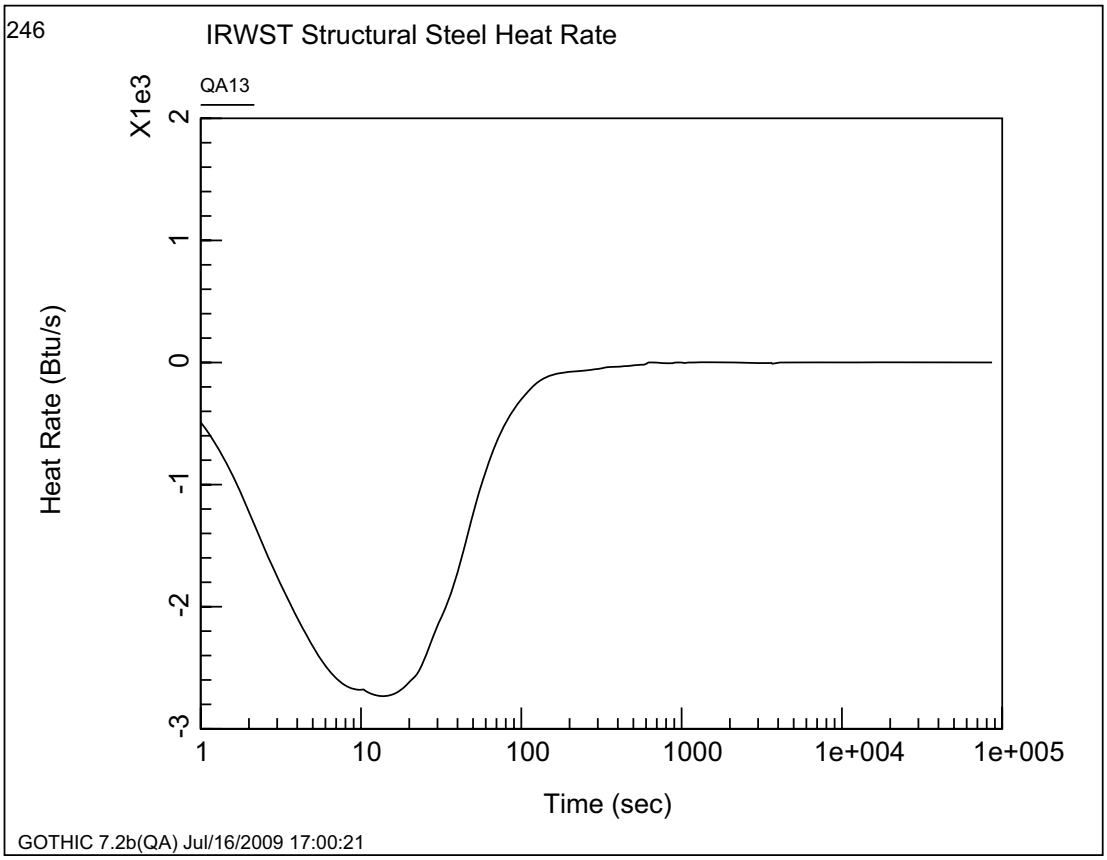


Figure 6.2.1-07d-164—Lower Equipment Room Wall Heat Rates

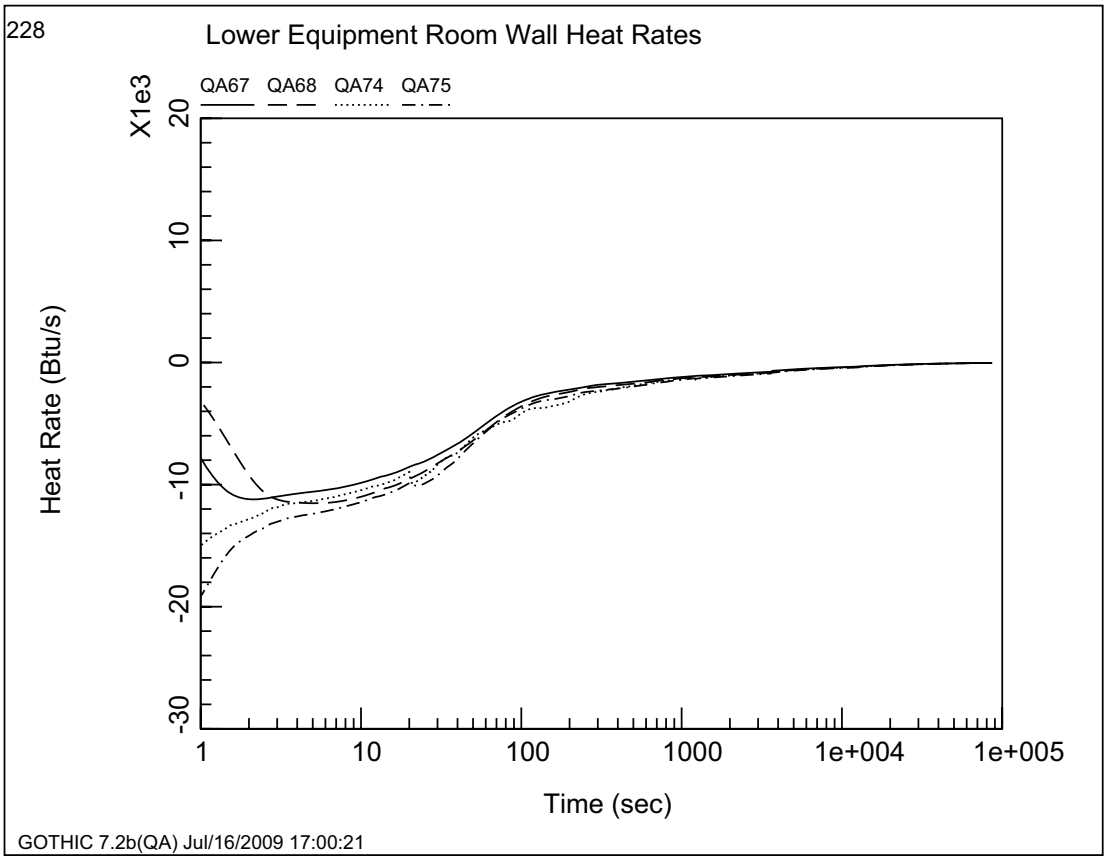


Figure 6.2.1-07d-165—Lower Equipment Room Structural Steel Heat Rates

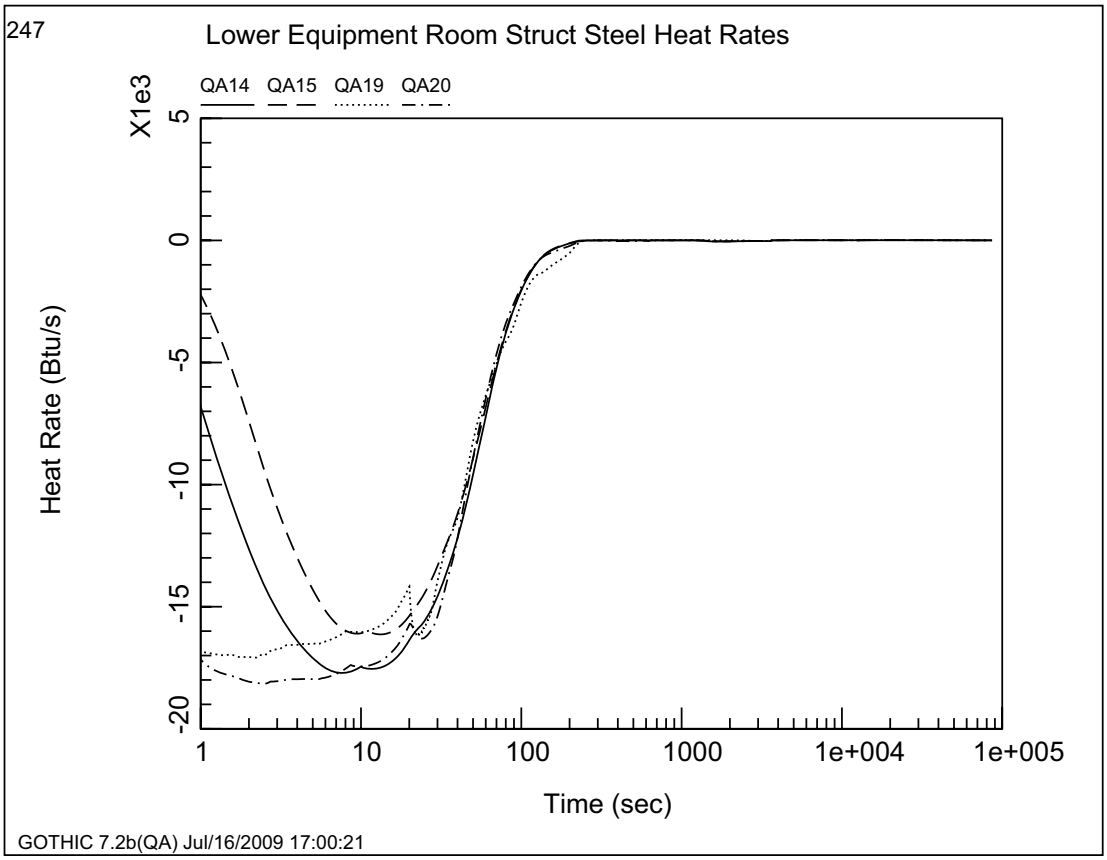


Figure 6.2.1-07d-166—Middle Equipment Room Wall Heat Rates

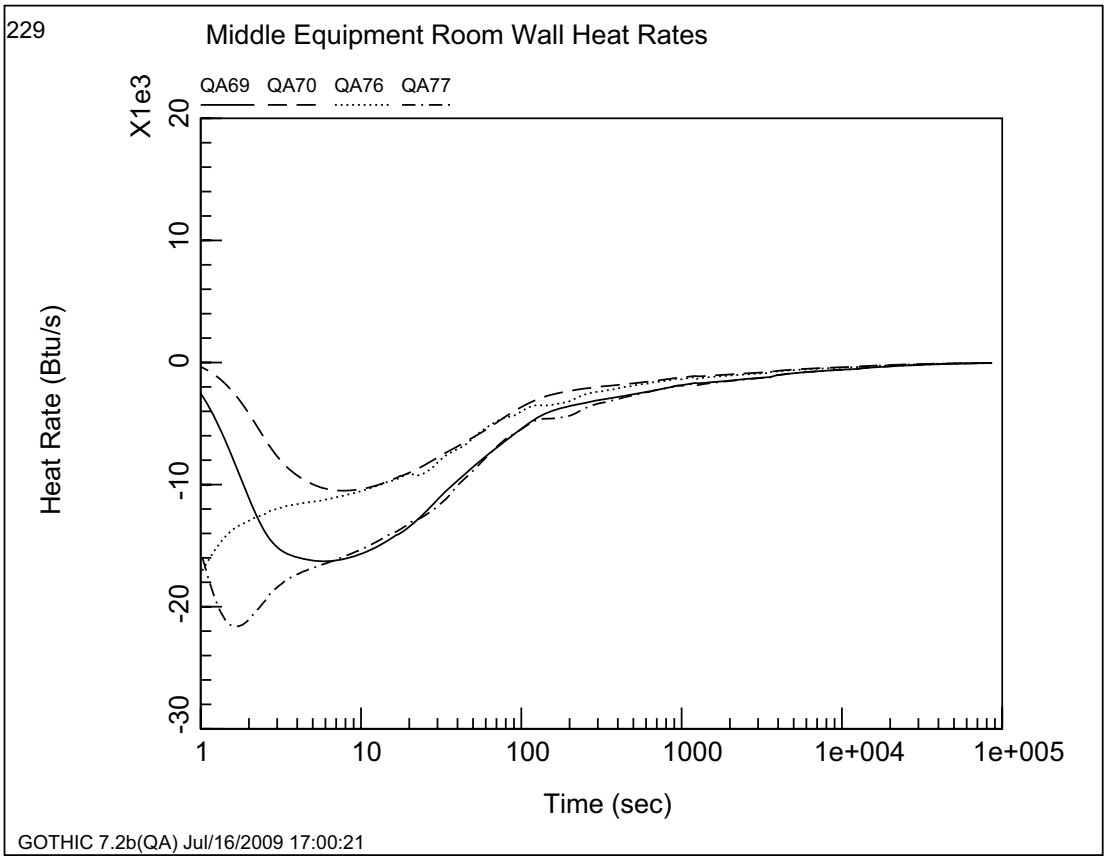


Figure 6.2.1-07d-167—Middle Equipment Room Structural Steel Heat Rates

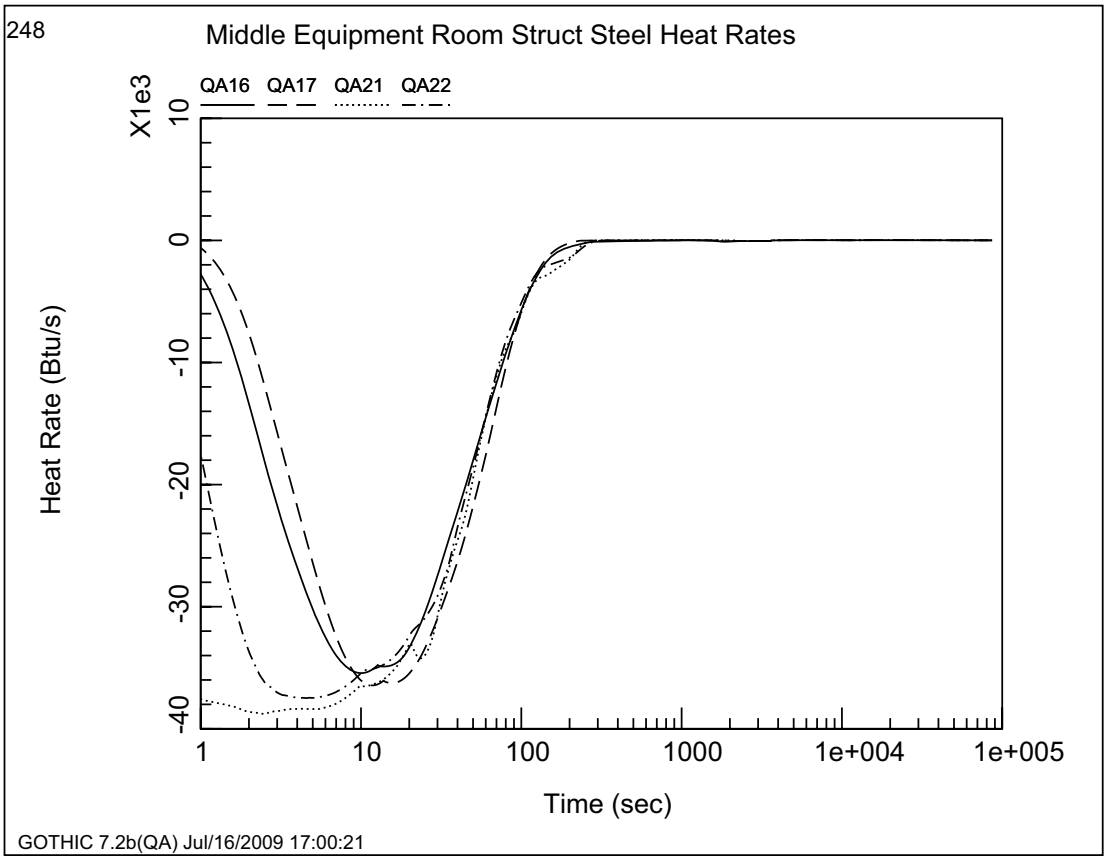


Figure 6.2.1-07d-168—Upper Equipment Room Wall Heat Rates

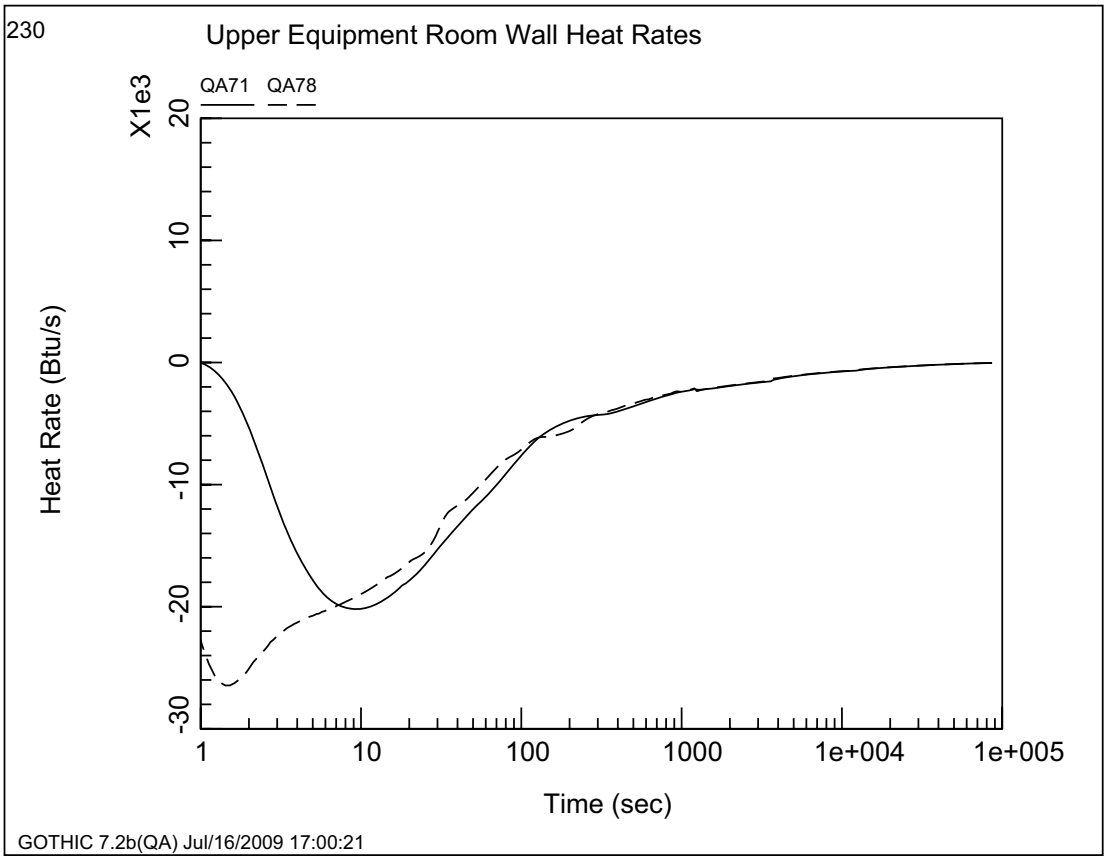


Figure 6.2.1-07d-169—Upper Equipment Room Structural Steel Heat Rates

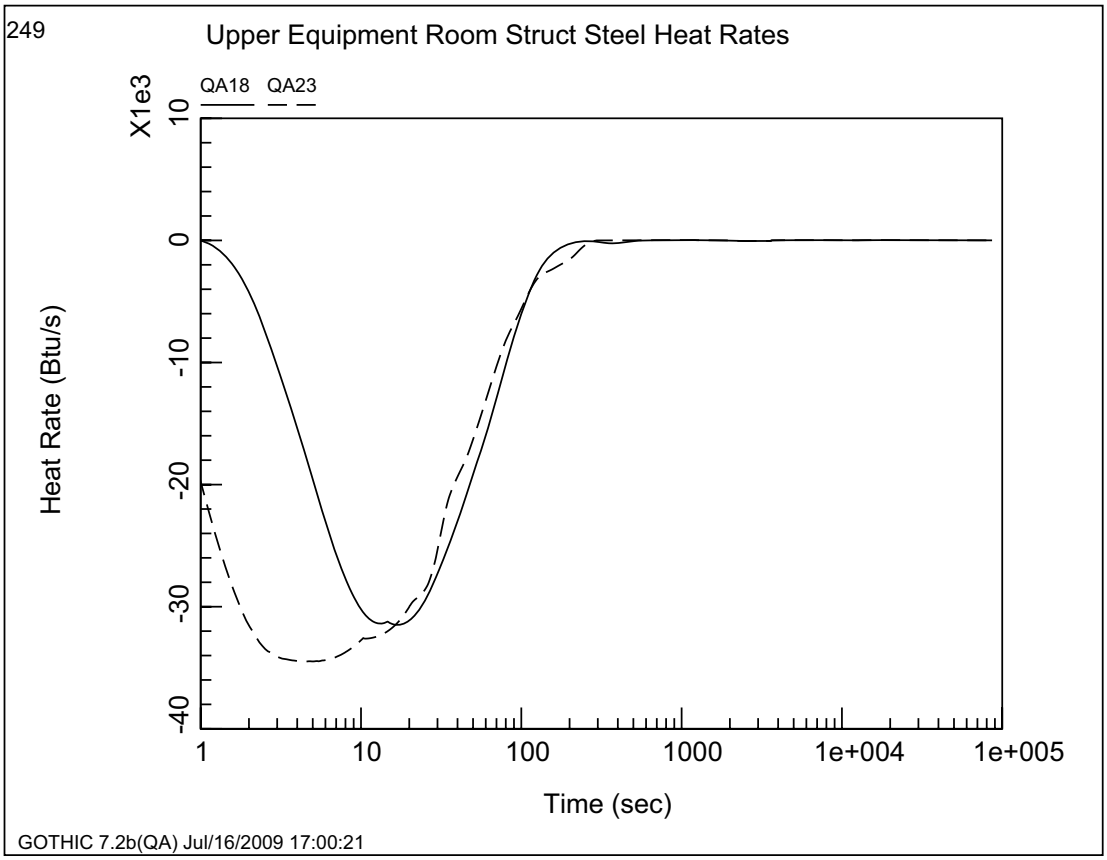


Figure 6.2.1-07d-170—Reactor Pressure Vessel Pit and Reactor Cavity Wall Heat Rates

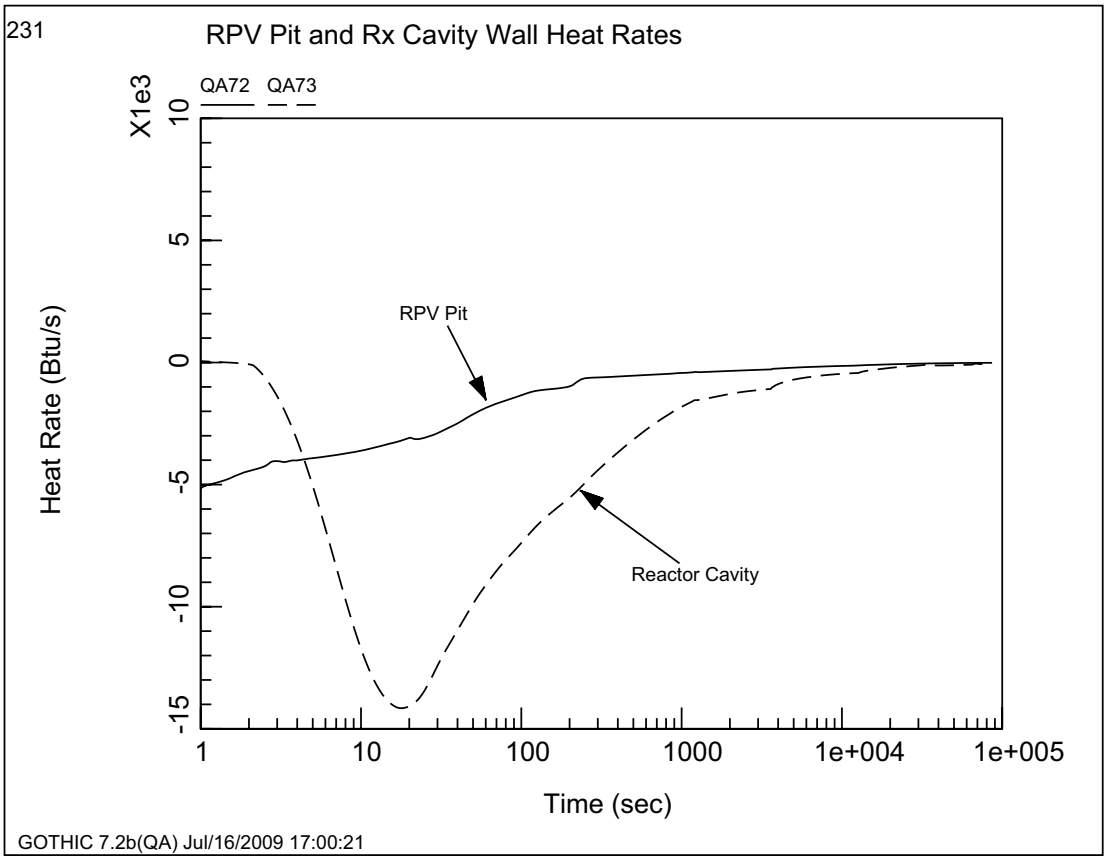


Figure 6.2.1-07d-171—Pressurizer Room and Surge Line Room Wall Heat Rates

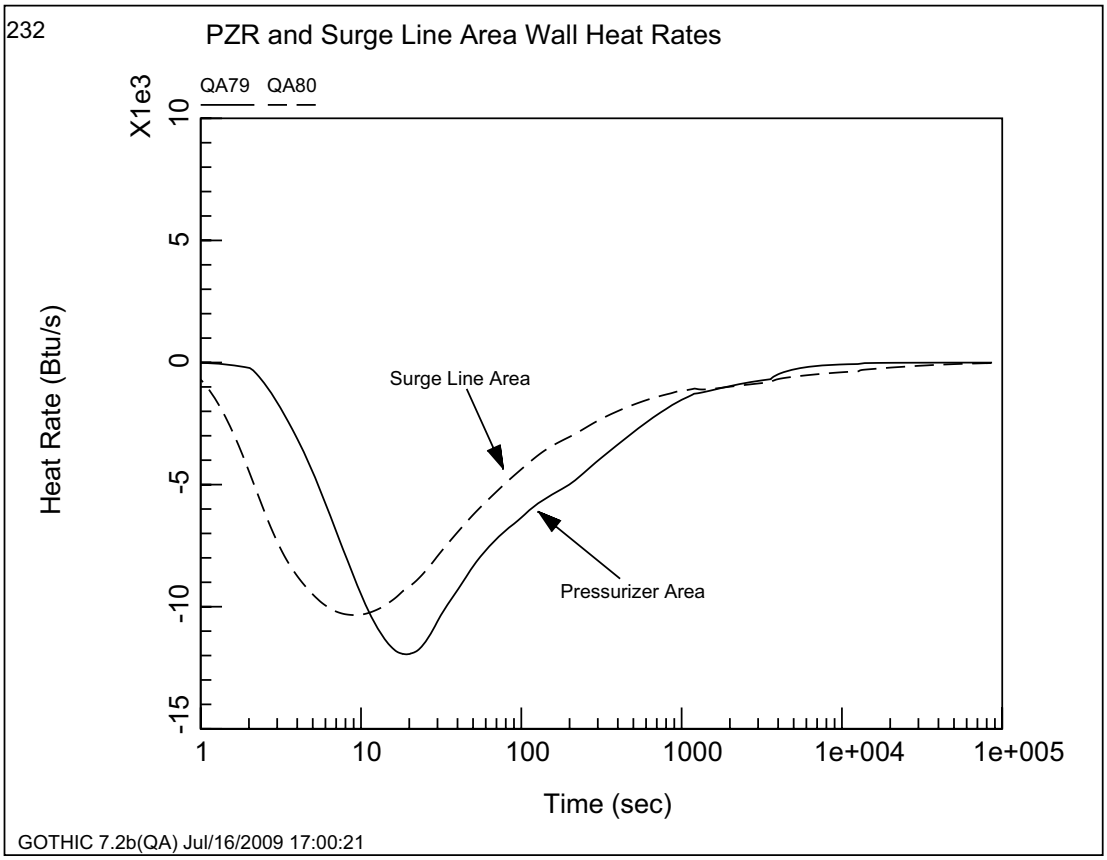
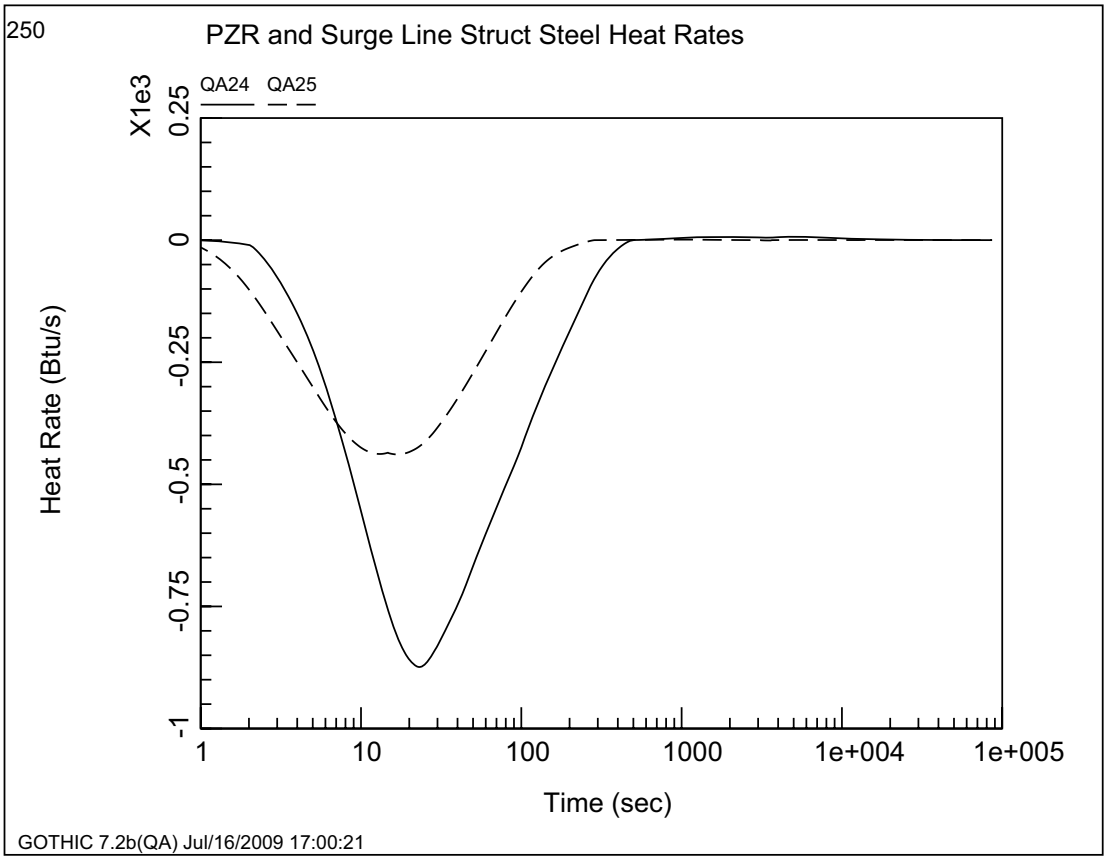


Figure 6.2.1-07d-172—Pressurizer Room and Surge Line Room Structural Steel Heat Rates



Accessible Area

Figure 6.2.1-07d-173—Containment Wall Heat Rates in the Loop 1 and 2 Lower, Middle and Upper Annulus Rooms

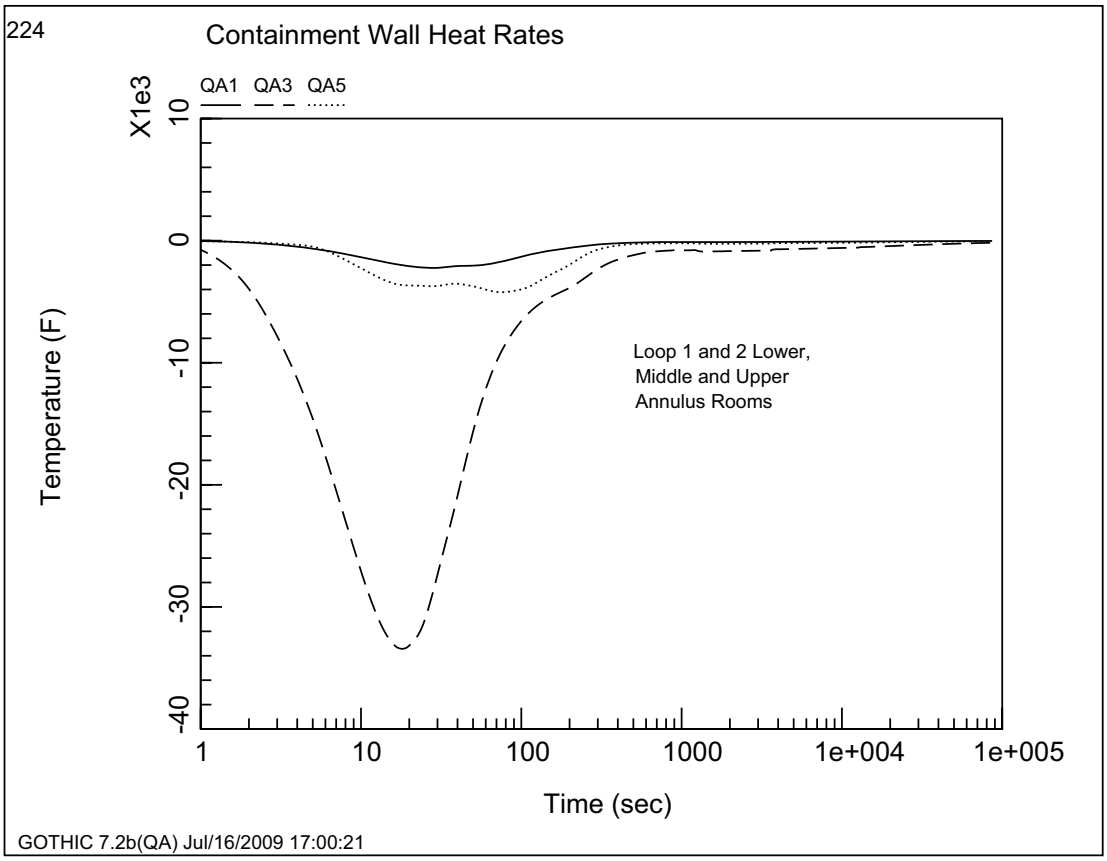


Figure 6.2.1-07d-174—Containment Wall Heat Rates in the Loop 3 and 4 Lower, Middle and Upper Annulus Rooms

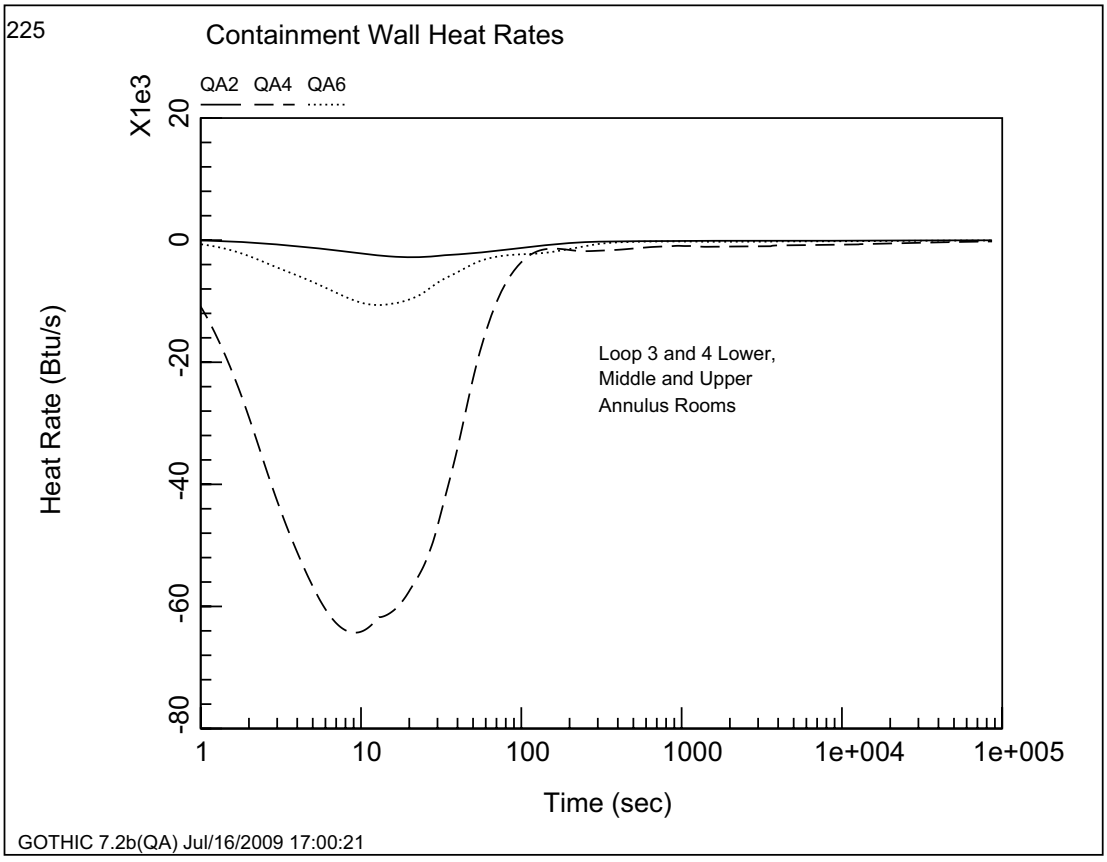


Figure 6.2.1-07d-175—Containment Wall Heat Rates in the Access Room, South Staircase and Hot Piping Penetration Room

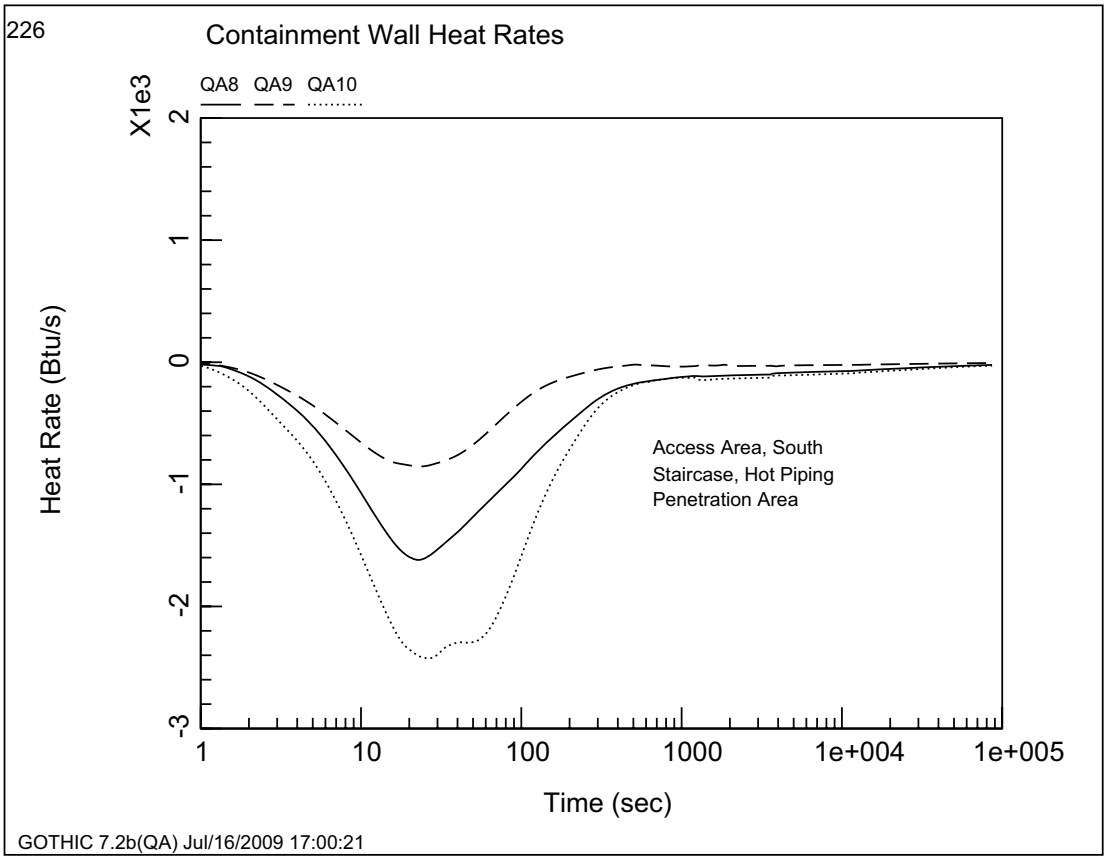


Figure 6.2.1-07d-176—CVCS Room and Steam Generator Blowdown Heat Exchanger Room Wall Heat Rates

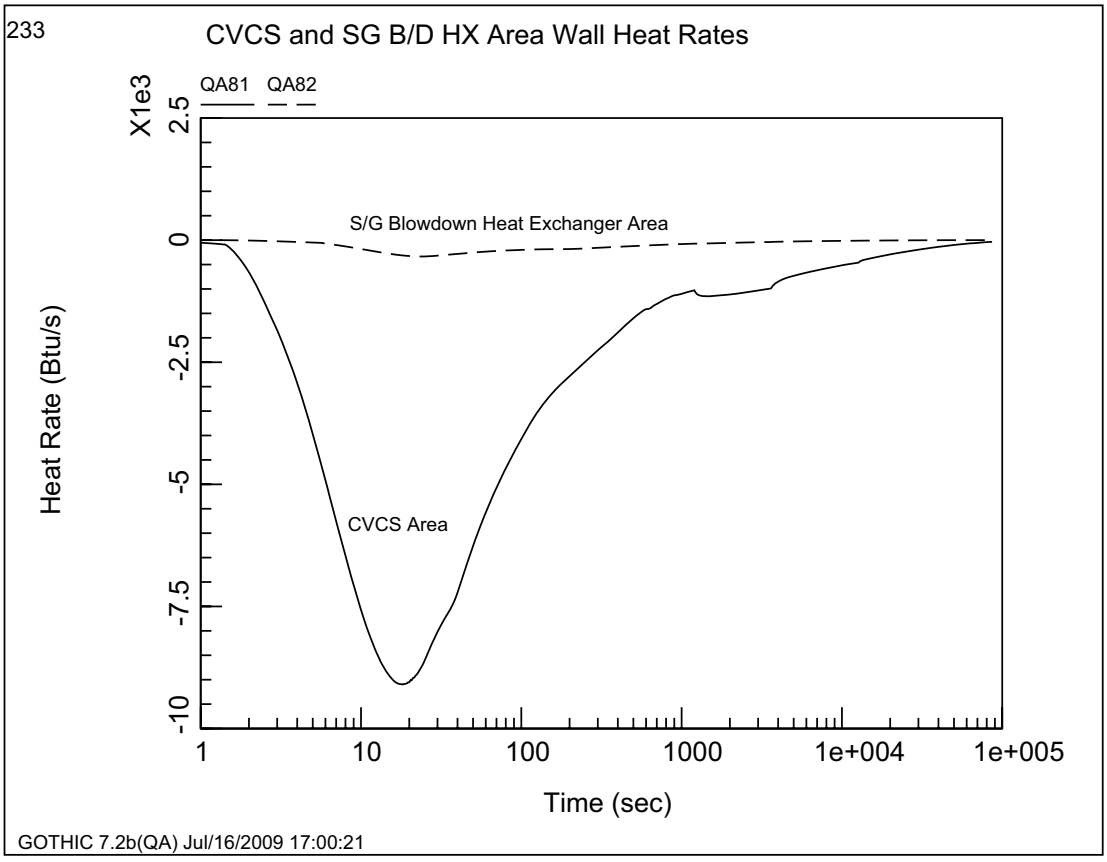


Figure 6.2.1-07d-177—CVCS Room and Steam Generator Blowdown Heat Exchanger Room Structural Steel Heat Rates

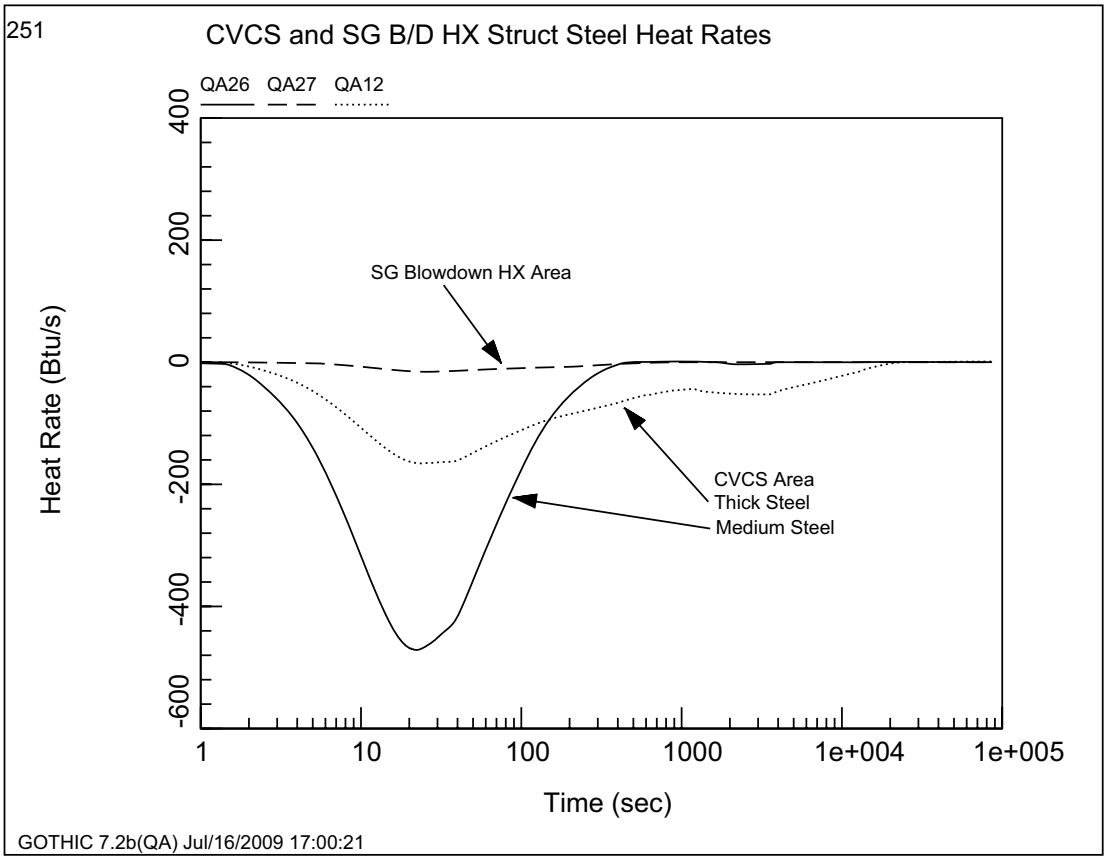


Figure 6.2.1-07d-178—Lower Annulus Room Wall Heat Rates

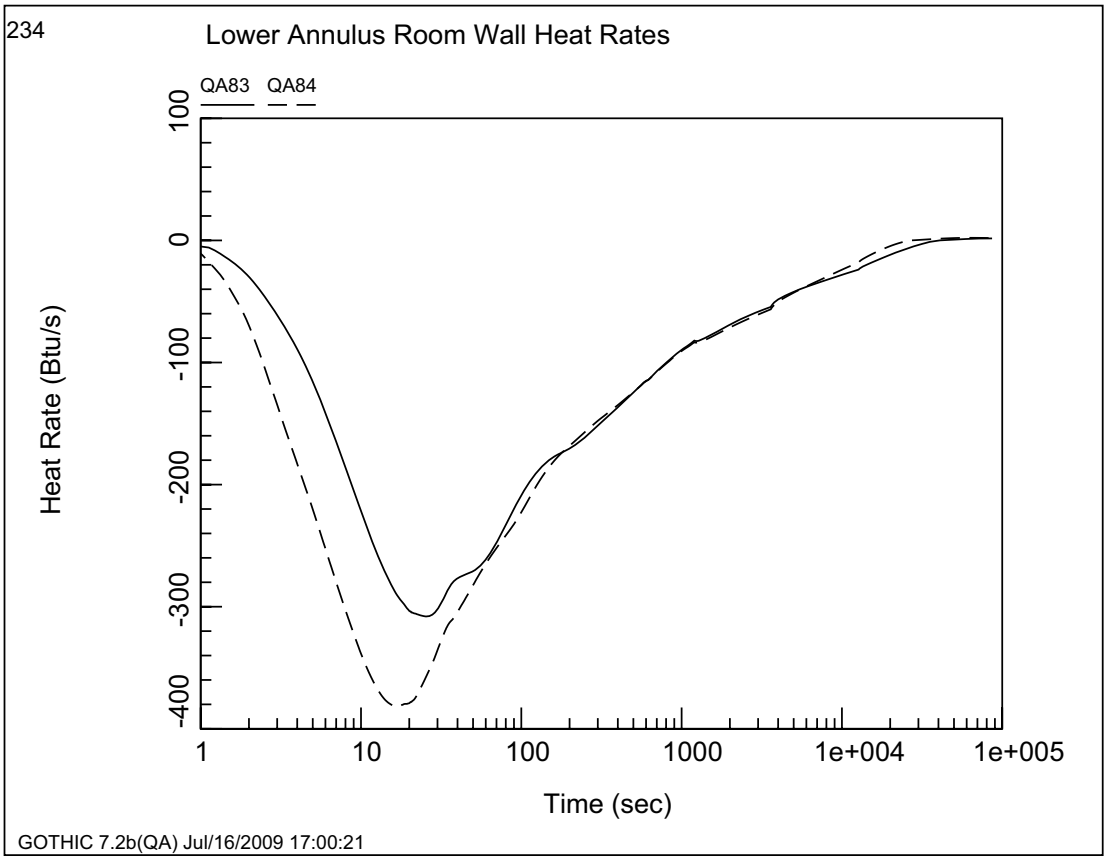


Figure 6.2.1-07d-179—Lower Annulus Room Structural Steel Heat Rates

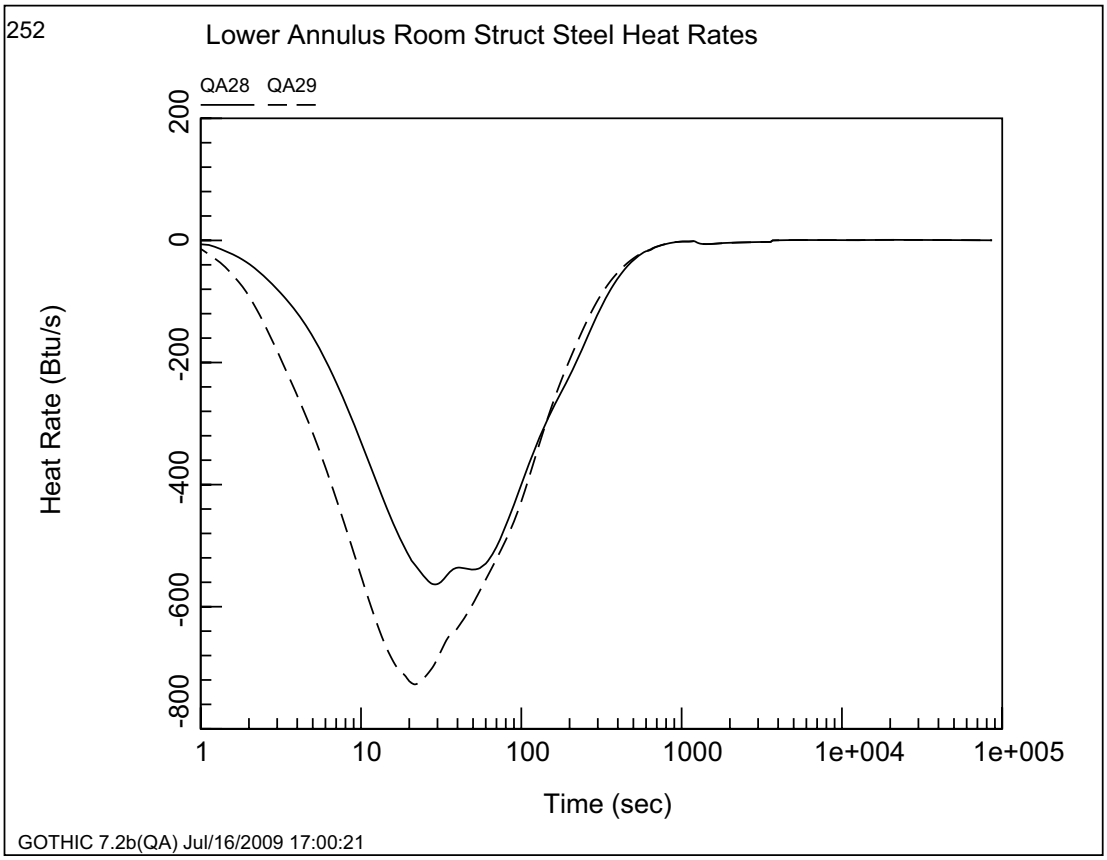


Figure 6.2.1-07d-180—Middle Annulus Room Wall Heat Rates

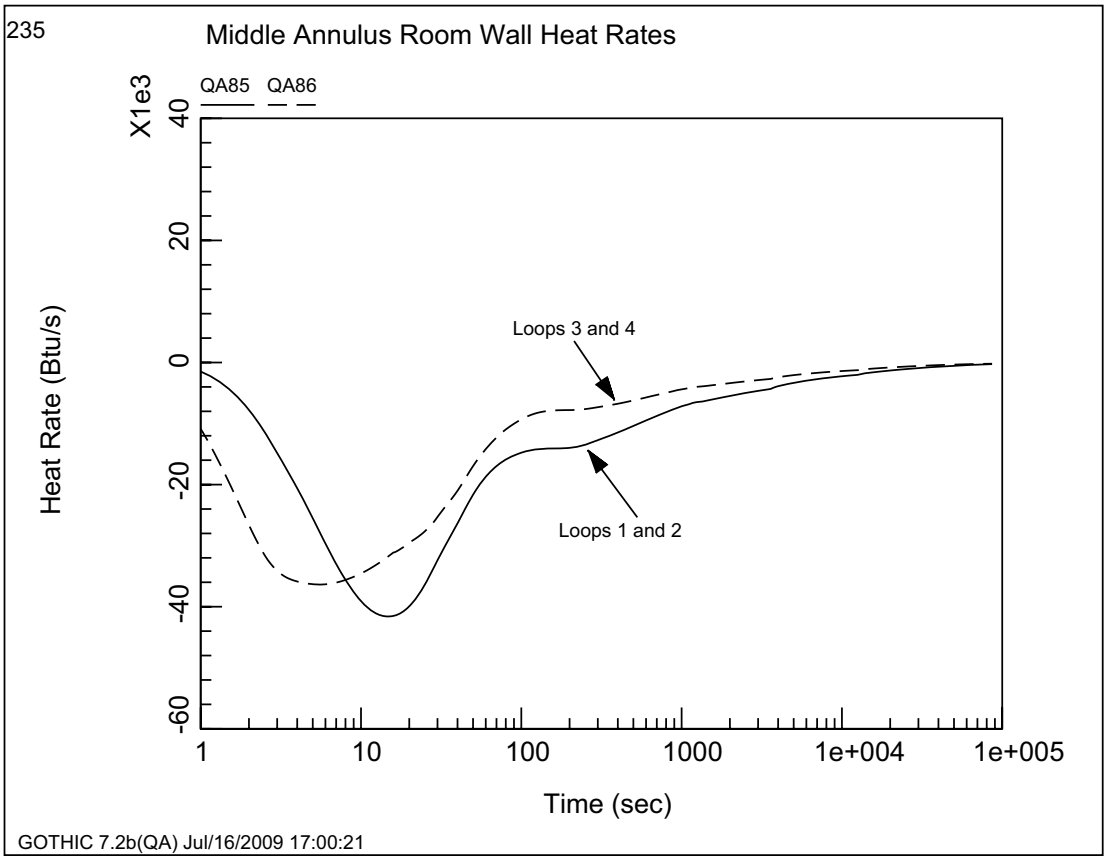


Figure 6.2.1-07d-181—Middle Annulus Room Structural Steel Heat Rates

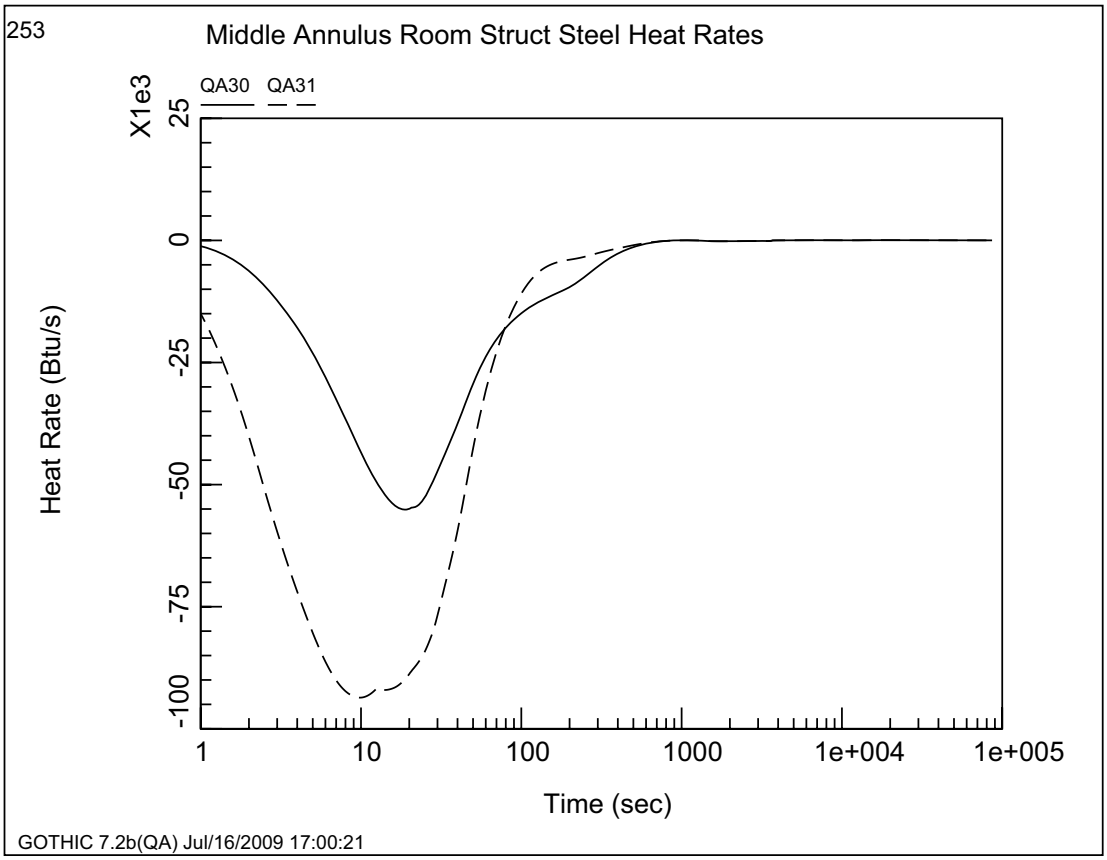


Figure 6.2.1-07d-182—Upper Annulus Room Wall Heat Rates

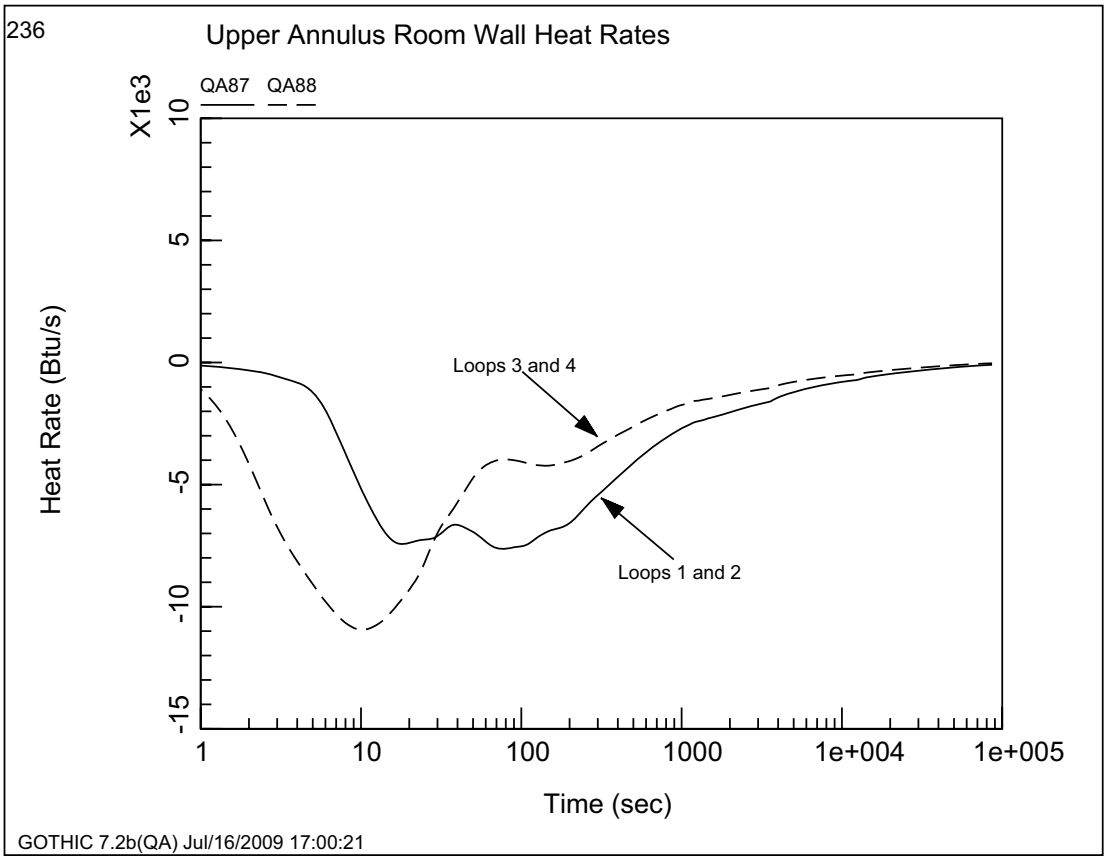


Figure 6.2.1-07d-183—Upper Annulus Room Structural Steel Heat Rates

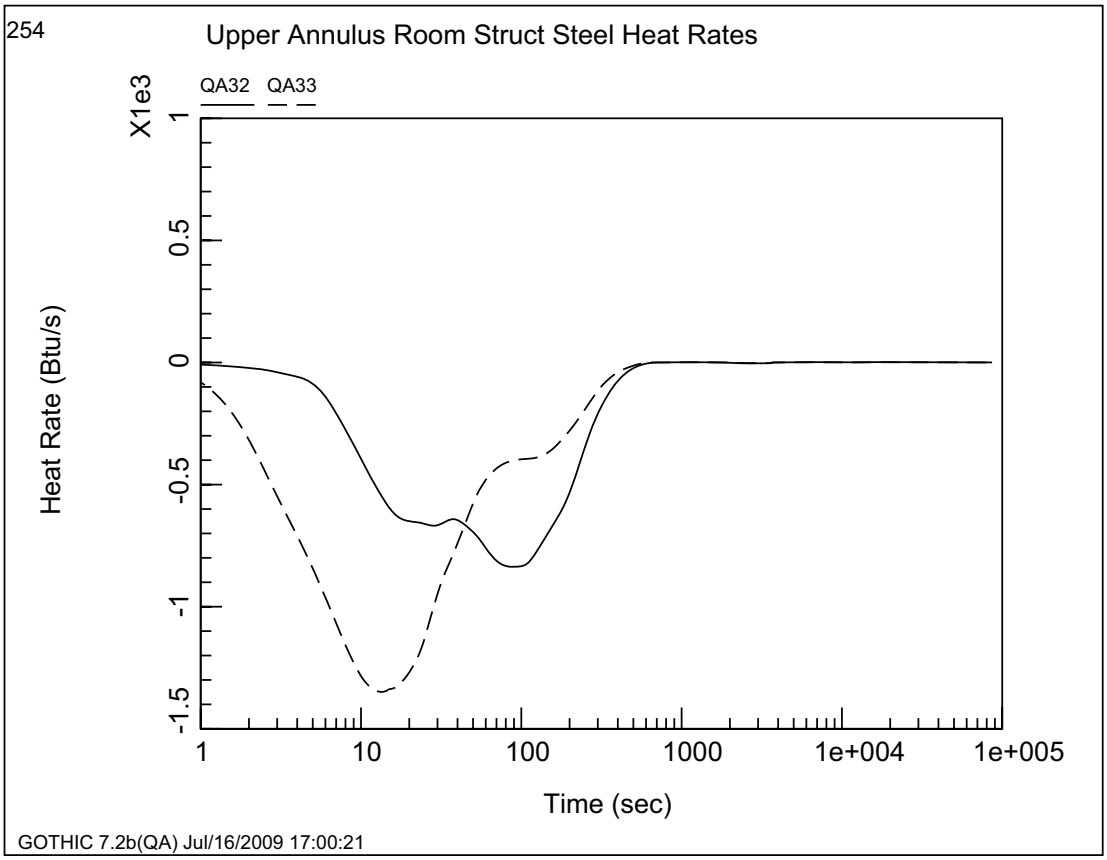


Figure 6.2.1-07d-184—Containment Dome Interior Wall Heat Rates from the 63.98 ft to the 71.53 ft Elevation

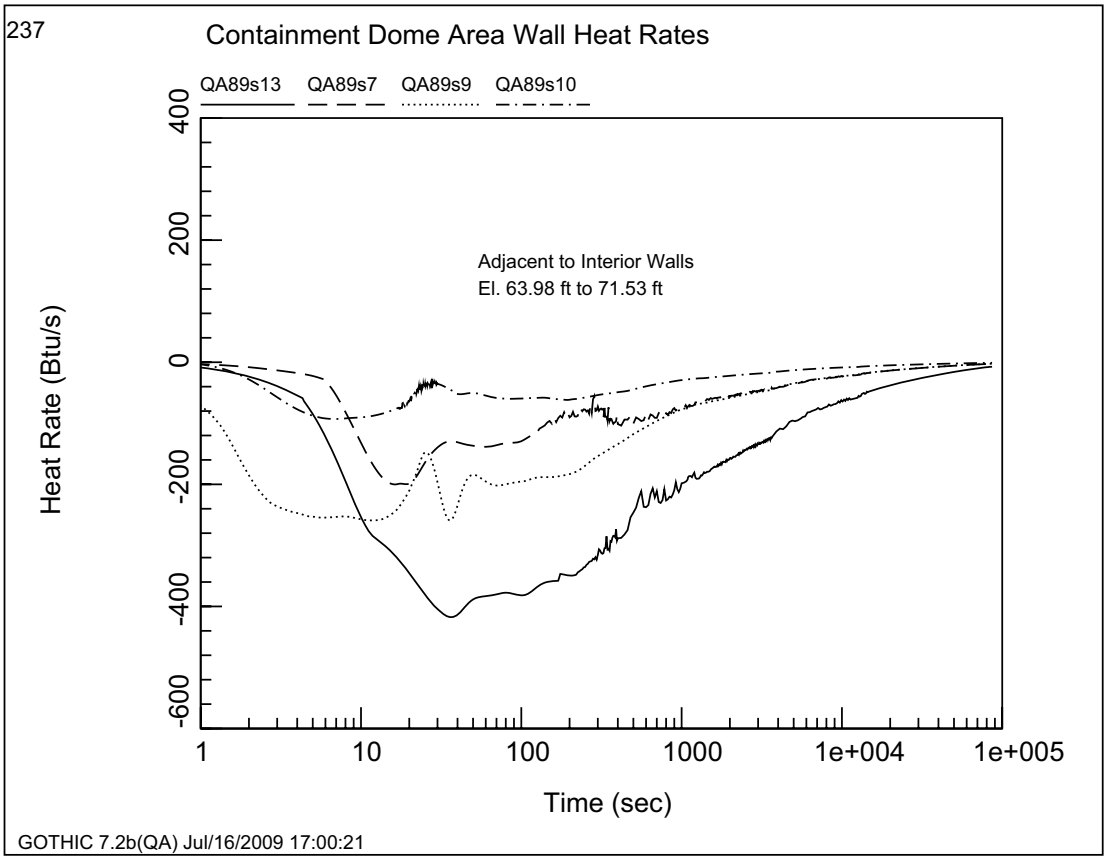


Figure 6.2.1-07d-185—Containment Dome Structural Steel Heat Rates from the 63.98 ft to the 71.53 ft Elevation

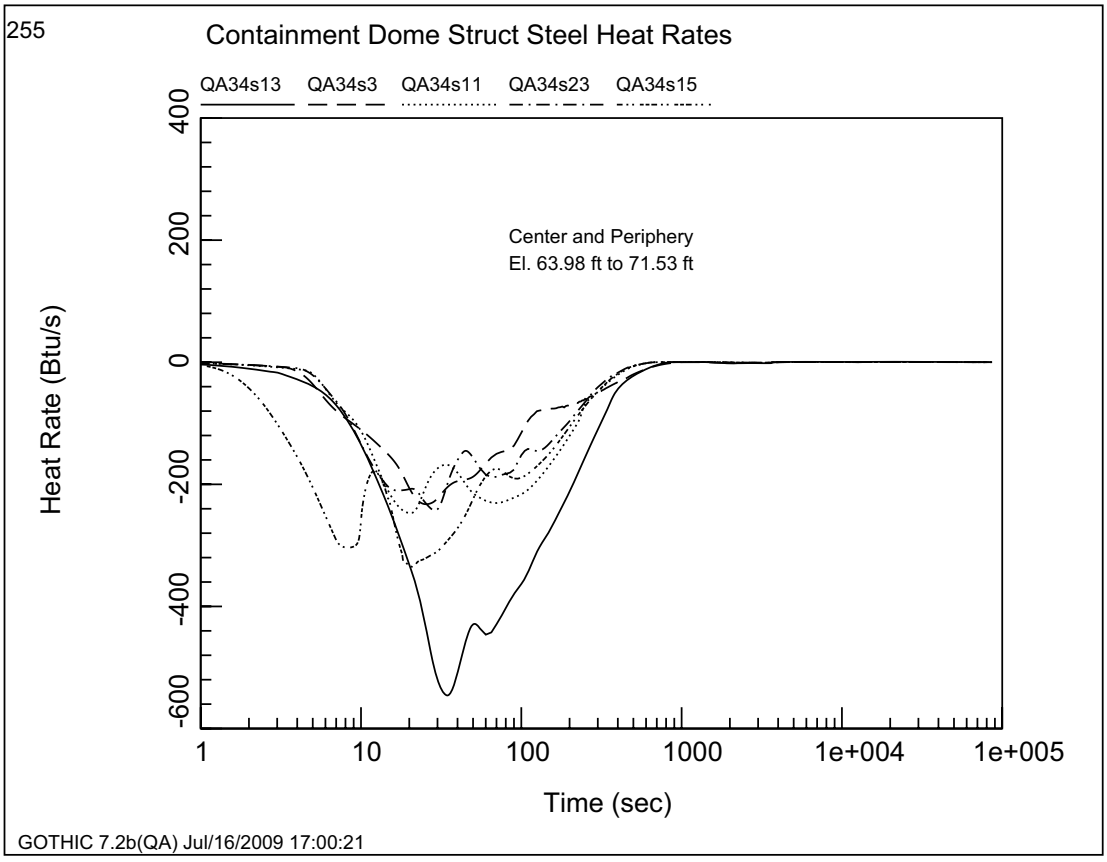


Figure 6.2.1-07d-186—Containment Dome Interior Wall Heat Rates from the 79.07 ft to the 86.29 ft Elevation

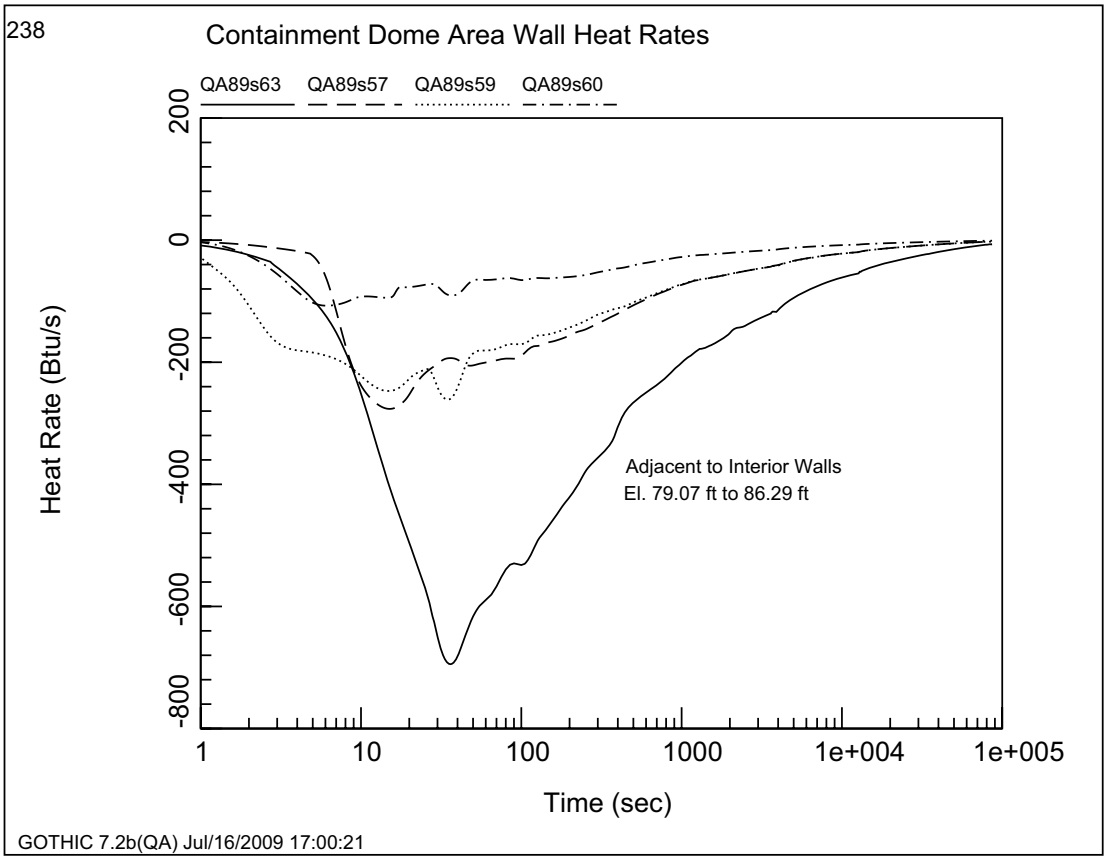


Figure 6.2.1-07d-187—Containment Dome Structural Steel Heat Rates from the 79.07 ft to the 86.29 ft Elevation

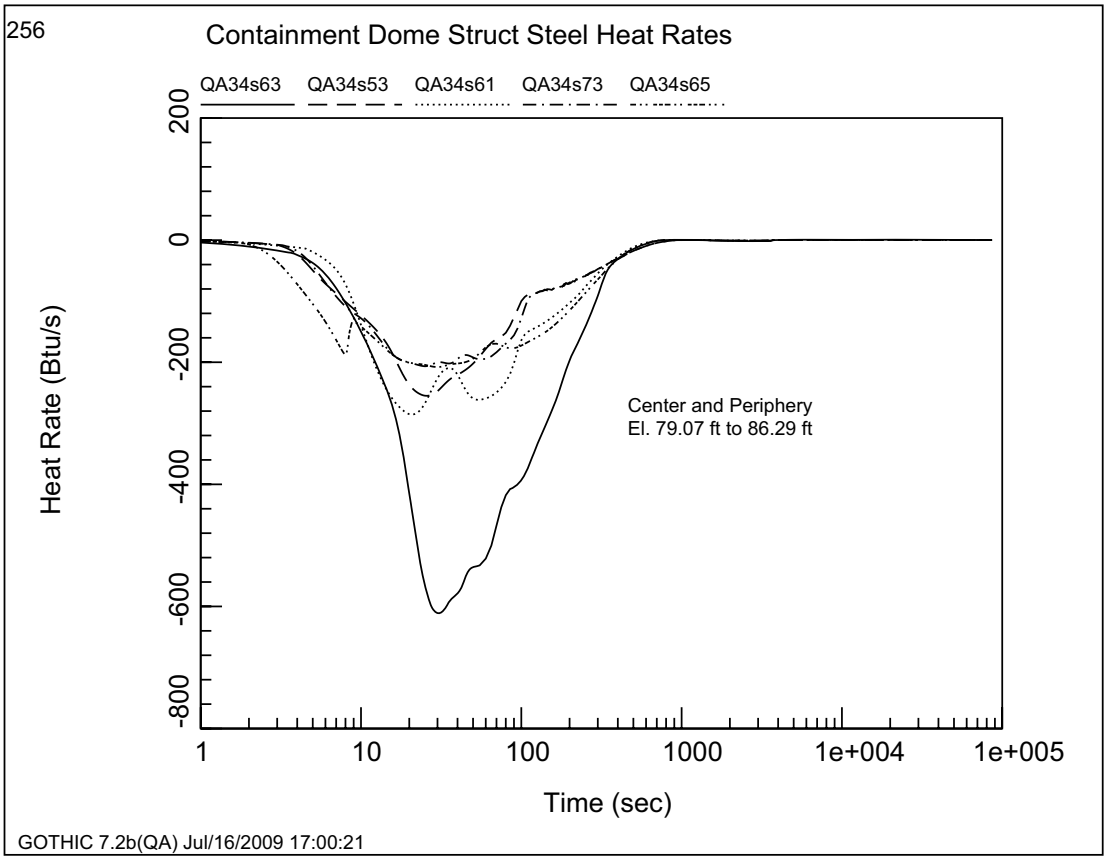


Figure 6.2.1-07d-188—Containment Dome Interior Wall Heat Rates from the 93.51 ft to the 98.59 ft Elevation

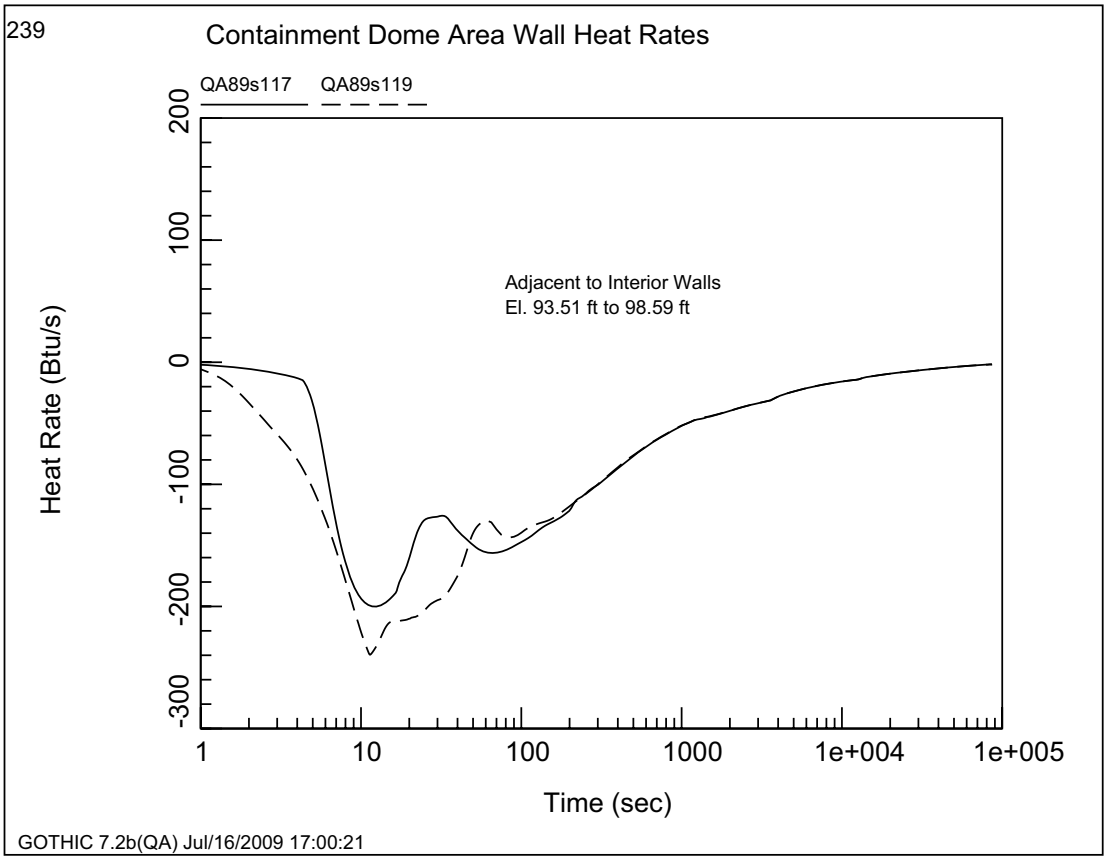


Figure 6.2.1-07d-189—Containment Dome Structural Steel Heat Rates from the 93.51 ft to the 98.59 ft Elevation

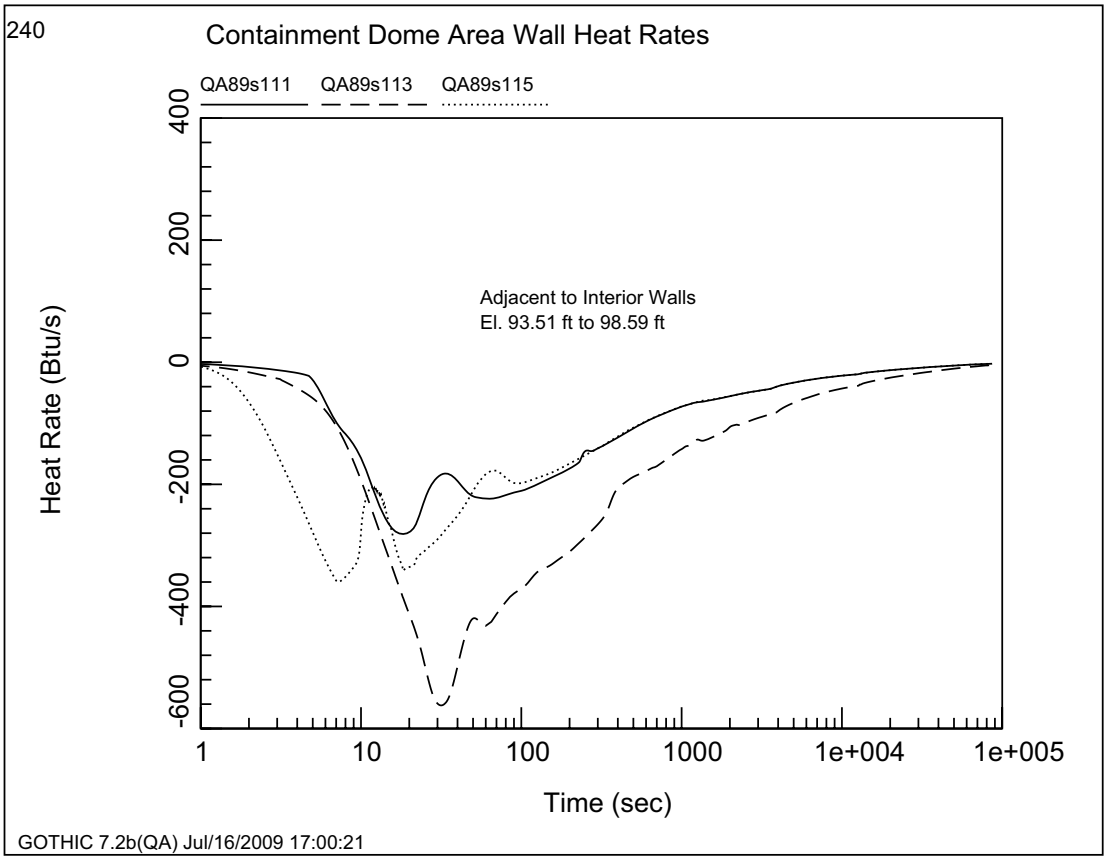


Figure 6.2.1-07d-190—Containment Dome Interior Wall Heat Rates from the 93.51 ft to the 98.59 ft Elevation

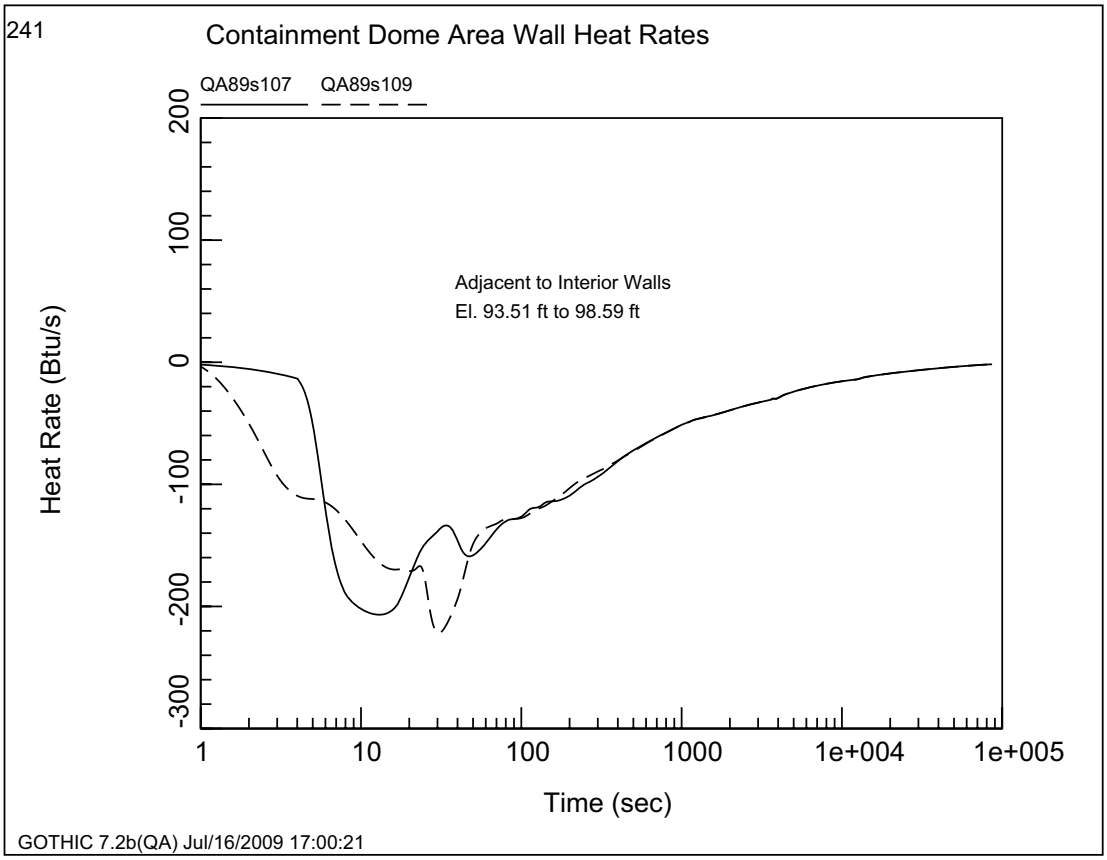


Figure 6.2.1-07d-191—Containment Dome Structural Steel Heat Rates from the 93.51 ft to the 98.59 ft Elevation

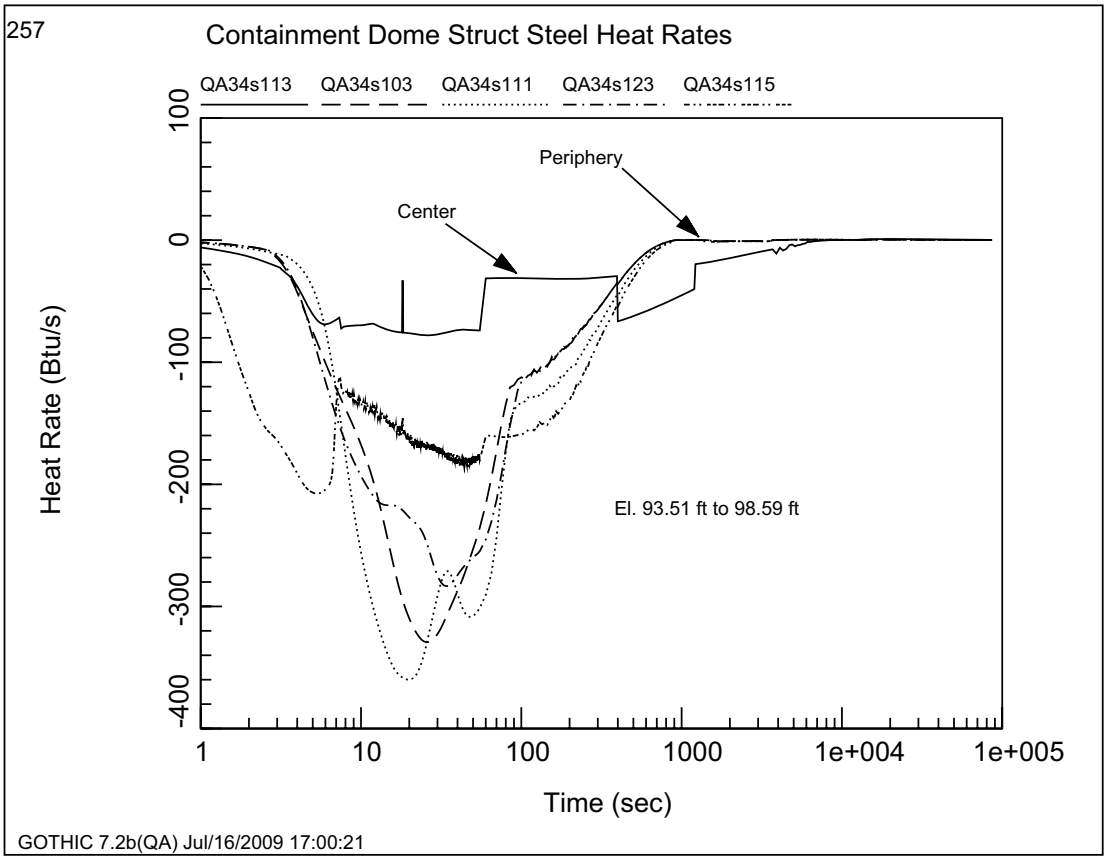


Figure 6.2.1-07d-192—Containment Dome Interior Wall Heat Rates from the 103.68 ft to the 108.76 ft Elevation

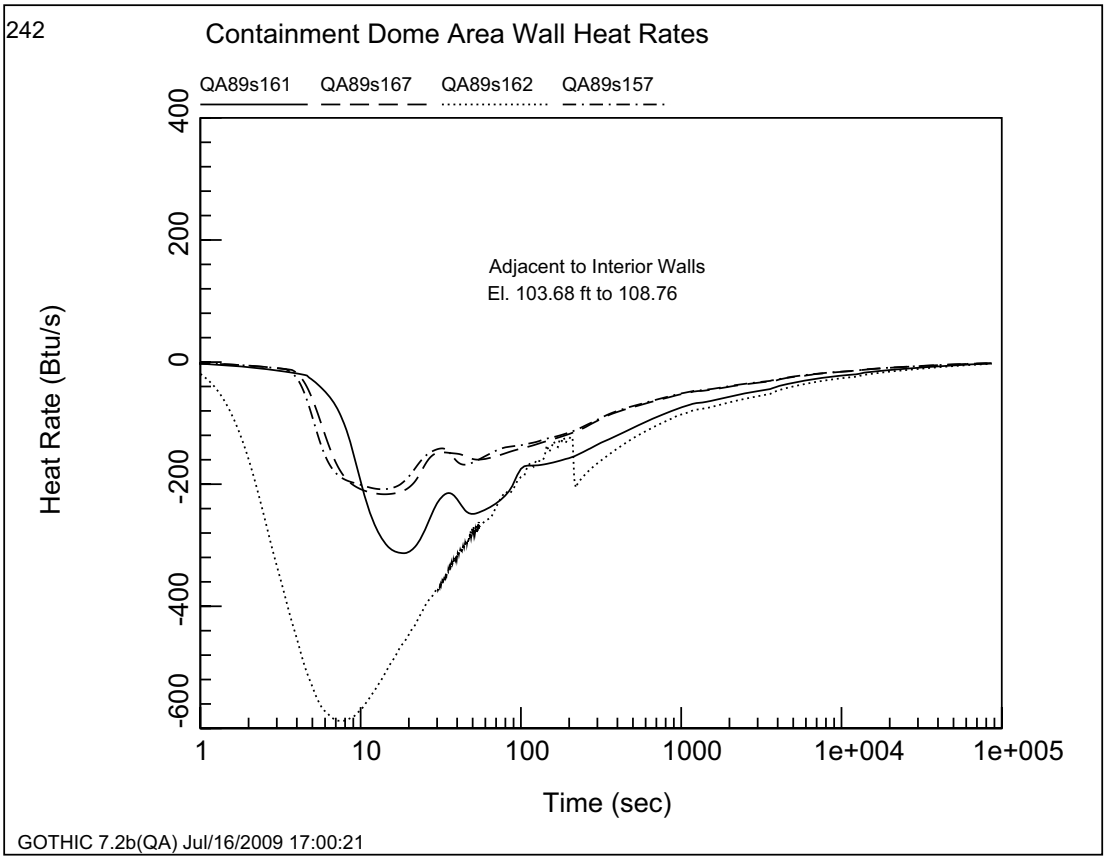


Figure 6.2.1-07d-193—Containment Dome Interior Wall Heat Rates from the 103.68 ft to the 108.76 ft Elevation

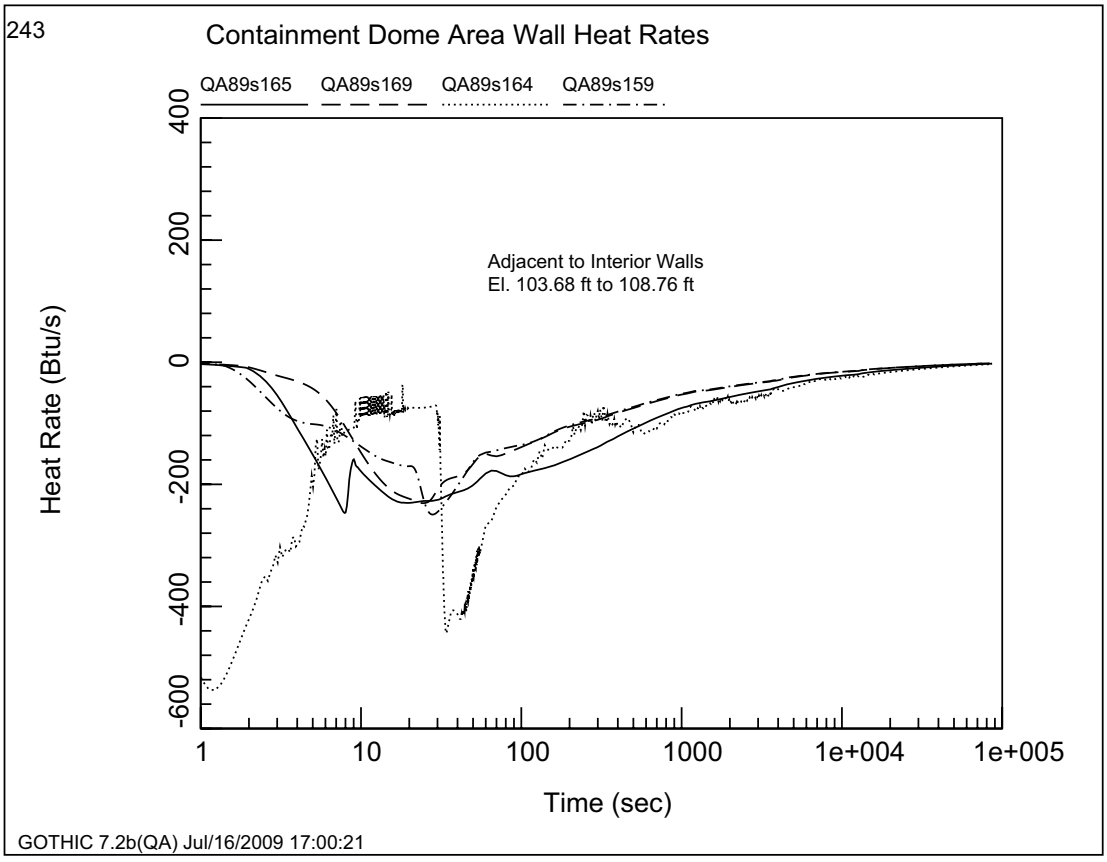


Figure 6.2.1-07d-194—Containment Dome Structural Steel Heat Rates from the 103.68 ft to the 108.76 ft Elevation

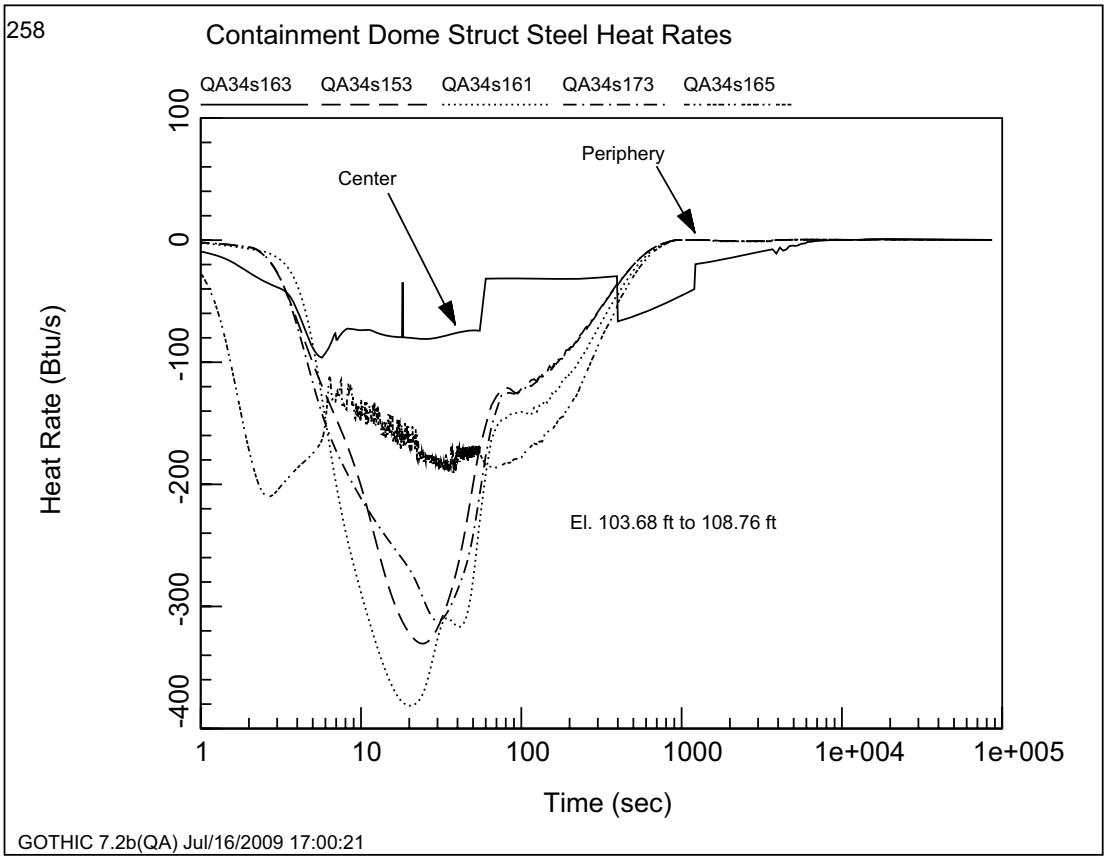


Figure 6.2.1-07d-195—Containment Dome Structural Steel Heat Rates from the 113.85 ft to the 119.90 ft Elevation

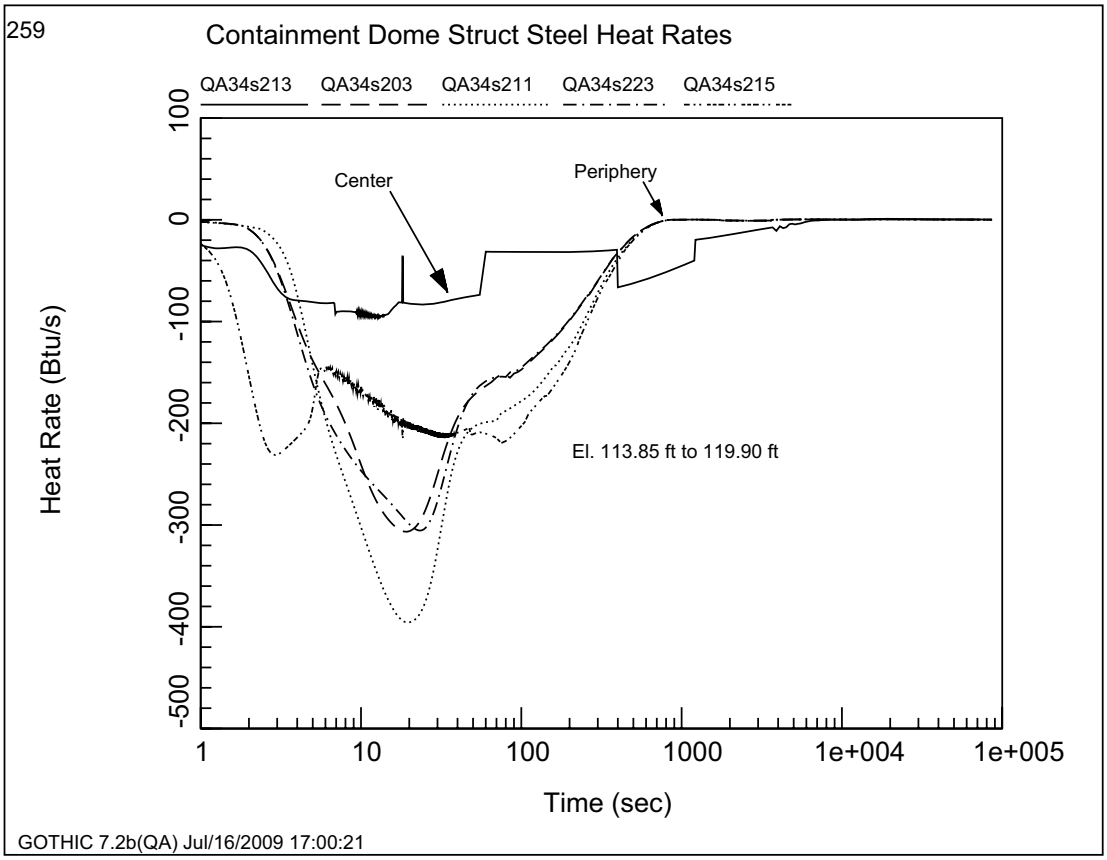


Figure 6.2.1-07d-196—Wall Heat Rates in the Access Room and the Hot Piping Penetration Room

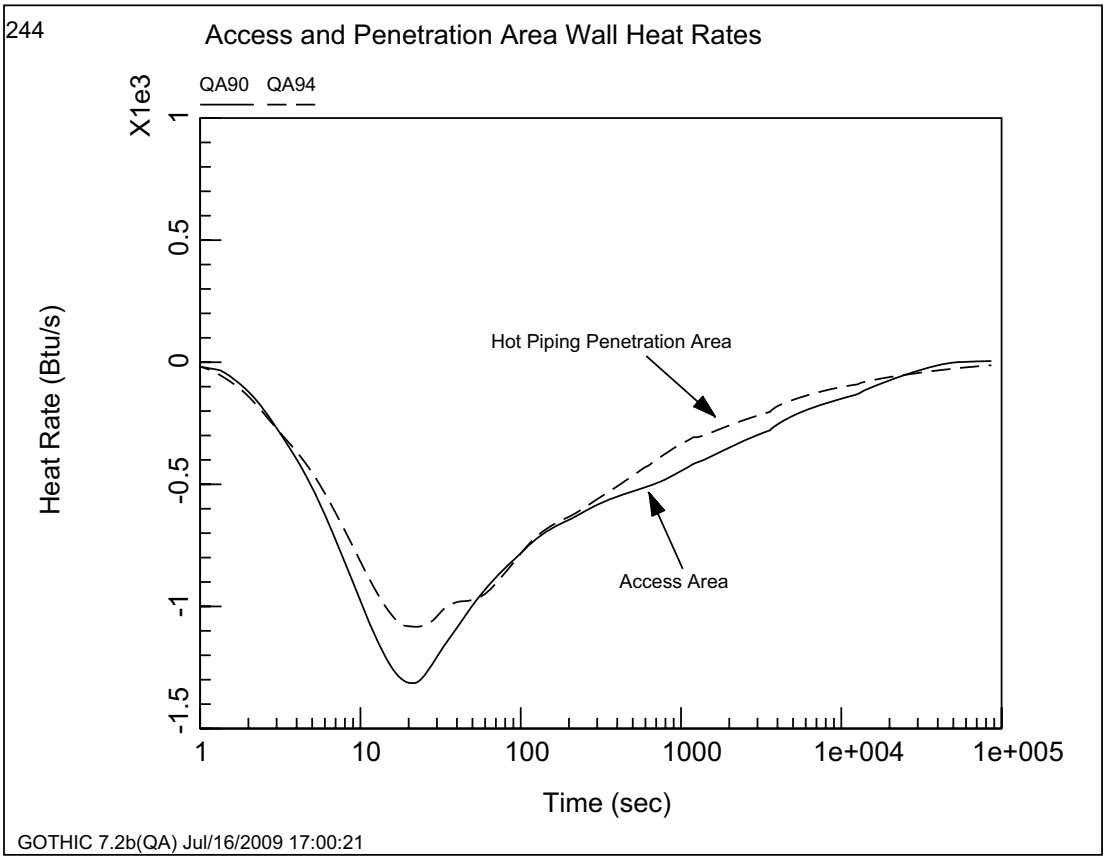


Figure 6.2.1-07d-197—Structural Steel Heat Rates in the Access Room and the Hot Piping Penetration Room

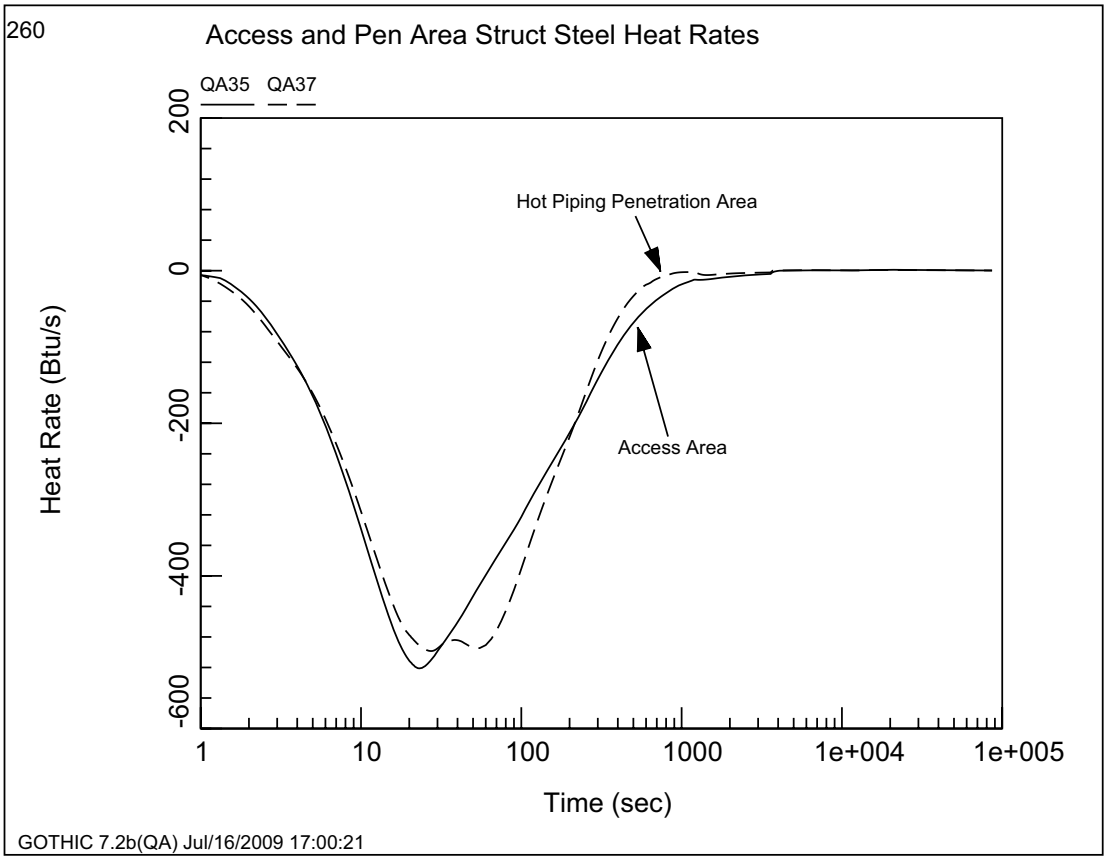


Figure 6.2.1-07d-198—Wall Heat Rates in the North and South Staircases and the Elevator

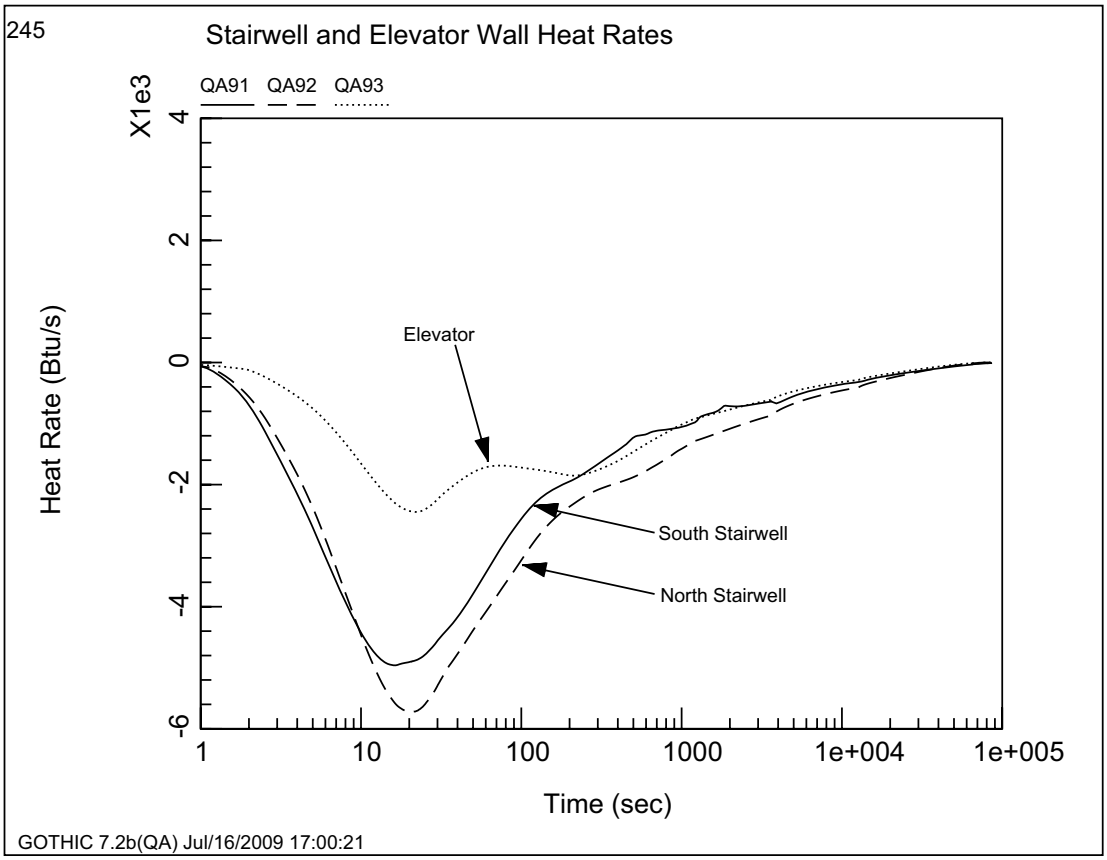


Figure 6.2.1-07d-199—Structural Steel Heat Rates in the South Staircase