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10 CFR 50.4
10 CFR 52.79

August 13, 2009

UN#09-346

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Subject: UniStar Nuclear Energy, NRC Docket No. 52-016
Response to Request for Additional Information for the
Calvert Cliffs Nuclear Power Plant, Unit 3,
RAI No. 123, Emergency Planning

Reference: John Rycyna (NRC) to Robert Poche (UniStar Nuclear Energy), "RAI No 123
ORLT 2511.doc" email dated July 14, 2009

The purpose of this letter is to respond to the request for additional information (RAI) identified in the NRC e-mail correspondence to UniStar Nuclear Energy, dated July 14, 2009 (Reference). This RAI addresses Emergency Planning, as discussed in Part 5 of the Calvert Cliffs Nuclear Power Plant (CCNPP) Unit 3 Combined License Application (COLA), Revision 5.

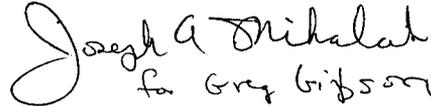
Enclosure 1 provides our response to RAI No. 123, Question 13.03-5. Enclosure 2 provides an evaluation of the CCNPP Unit 3 Emergency Plan against NUREG-0800, Standard Review Plan, Section 13.3. Our response to Question 13.03-5 does not include any new regulatory commitments and does not impact COLA content.

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KRO

If there are any questions regarding this transmittal, please contact me at (410) 470-4205, or Mr. Michael J. Yox at (410) 495-2436.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on August 13, 2009

A handwritten signature in black ink that reads "Joseph A. Michalak" on the top line and "for Greg Gibson" on the bottom line. The signature is written in a cursive style.

Greg Gibson

- Enclosures:
- 1) Response to NRC Request for Additional Information RAI No. 123, Question 13.03-5, Emergency Planning, Calvert Cliffs Nuclear Power Plant, Unit 3
 - 2) Evaluation of the Calvert Cliffs Nuclear Power Plant Unit 3 Emergency Plan Against NUREG-0800, Standard Review Plan, Section 13.3

cc: John Rycyna, NRC Project Manager, U.S. EPR COL Application
Laura Quinn, NRC Environmental Project Manager, U.S. EPR COL Application
Getachew Tesfaye, NRC Project Manager, U.S. EPR DC Application (w/o enclosure)
Loren Plisco, Deputy Regional Administrator, NRC Region II (w/o enclosure)
Silas Kennedy, U.S. NRC Resident Inspector, CCNPP, Units 1 and 2
U.S. NRC Region I Office

UN#09-346

Enclosure 1

**Response to NRC Request for Additional Information
RAI No. 123, Question 13.03-5, Emergency Planning,
Calvert Cliffs Nuclear Power Plant, Unit 3**

RAI No. 123

Question 13.03-5

(Site 47) The staff is unable to locate in Part 5 of the Calvert Cliffs Nuclear Power Plant Unit 3 (CCNPP3) COL application an evaluation that compares the Emergency Plan to the acceptance criteria for Emergency Planning in Section 13.3 of the Standard Review Plan (SRP), Revision 3, dated March 2007 (NUREG-0800), as required by 10 CFR 52.79(a)(41). Therefore, in accordance with 10 CFR 52.79(a)(41), provide an evaluation of the CCNPP3 Emergency Plan against section 13.3 of SRP Revision 3.

Response

The Evaluation of the Calvert Cliffs Nuclear Power Plant Unit 3 Emergency Plan Against NUREG-0800, Standard Review Plan, Section 13.3 is provided in Enclosure 2.

COLA Impact

The COLA will not be revised as a result of this response.

Enclosure 2

**Evaluation of the
Calvert Cliffs Nuclear Power Plant Unit 3 Emergency Plan
Against NUREG-0800, Standard Review Plan, Section 13.3**

CC3 COL NUREG-0800 Standard Review Plan Section 13.3 Conformance Evaluation

Appendix 1 and 2 of the E-Plan contain tables that provide specific cross references to the regulations and EP related guidance documents.

Section II. Acceptance Criteria	Section Reference/Comments
<p>1. All of the standards of 10 CFR 50.47(b), as supported by the guidance in the corresponding planning standards and evaluation criteria of NUREG 0654/FEMA-REP-1, Rev. 1, (including the March 2002 addenda) must be met before an OL is issued pursuant to 10 CFR 50.57 or a COL is issued pursuant to 10 CFR 52.97.</p> <p>In addition, for the first reactor at a site, Appendix E to 10 CFR Part 50 requires that a full participation exercise be conducted within 2 years before NRC issuance of an operating license for full power (i.e., one authorizing operation above 5 percent of rated power). Because this exercise would be included in the ITAAC required for a COL, its acceptance criteria would have to be satisfied before fuel loading pursuant to a COL (see Table 14.3.10-1).</p>	<p>The CC3 E-Plan has been developed in a format consistent with the format of NUREG-0654. It contains the 10 CFR 50.54(b) standards specified by each of the NUREG-0654 evaluation criteria.</p> <p>As documented in ITAAC# 8.1, a full participation exercise (test) will be conducted within the specified time periods of Appendix E to 10 CFR Part 50.</p>
<p>2. The onsite and, except as provided in 10 CFR 50.47(d), offsite emergency response plans for nuclear power reactors must meet the standards established in 10 CFR 50.47(b) and applicable requirements of Appendix E to 10 CFR Part 50. Compliance with these regulations is determined by using the guidance in Regulatory Guide (RG) 1.101, Rev. 2, which endorses NUREG-0654/FEMA-REP-1, Rev. 1, and through it NUREG-0396, and NUREG-0696. NUREG-0654/FEMA-REP-1, Rev. 1, establishes an acceptable basis for NRC licensees and State, tribal and local governments to develop radiological emergency plans and procedures, and improve their overall state of emergency preparedness.</p> <p>NUREG-0696 discusses the facilities and systems to be provided by nuclear power plant licensees to aid the licensee's response to emergency situations.</p> <p>Additional guidance is provided in NUREG-0718, NUREG-0737, Supplement 1 to NUREG-0737, NUREG-0814, and Supplement 3 to NUREG-0654/FEMA-REP-1, Rev. 1.</p>	<p>The CC3 E-Plan has been developed in a format consistent with the format of NUREG-0654 to specifically address each of the NUREG-0654 evaluation criteria.</p> <p>Applicable NUREG-0396 elements are addressed in E-Plan Part II Sections A-2 and B-7.</p> <p>NUREG-0696 elements are addressed in E-Plan Part II Sections F and H.</p> <p>E-Plan related onsite facility and equipment elements of NUREG-0718, NUREG-0737, NUREG-0737 Supplement 1 and NUREG-0814 elements are addressed in E-Plan Part II Sections F and H and in the DCD. The CC3 EOF is shared with Units 1 and 2. It has been built to the specifications of the applicable requirements and approved by the NRC for the use of the existing operating stations.</p> <p>Adherence to NUREG-0737 is referenced in the US EPR DCD.</p> <p>NUREG-0654 Supplement 3 is addressed in E-Plan Part II Section J.</p>

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Section II. Acceptance Criteria	Section Reference/Comments
<p>3. 10 CFR 50.47(b)(4) requires a standard emergency classification and action level scheme. Section IV.C, "Activation of Emergency Organization," of Appendix E identifies the four emergency classes. Section IV.B, "Assessment Actions," of Appendix E to 10 CFR Part 50 also requires emergency action levels.</p> <p>The emergency plan should include the emergency classification level scheme described in Appendix 1 and Supplement 3 to NUREG-0654. The staff anticipates that any new application will use an emergency action level scheme similar to that described in Revision 4 of NEI 99-01, "Methodology for Development of Emergency Action Levels," dated January 2003, which was endorsed in Revision 4 Regulatory Guide (RG) 1.101, "Emergency Planning and Preparedness for Nuclear Power Reactors," dated October 2003. However, Revision 4 of NEI 99-01, "Methodology for Development of Emergency Action Levels," dated January 2003, is not considered to be entirely applicable to advanced light water reactor designs. Even though the majority of Revision 4 of NEI 99-01 may be applicable to any reactor design and should be used, the unique characteristics of the new reactor should be addressed in the development of emergency action levels specific to the new plant and the site. The format of the emergency action level scheme should follow the convention established in Regulatory Information Summary 2003-18, "Use of Nuclear Energy Institute (NEI) 99-01, Methodology for Development of Emergency Action Levels," Revision 4, dated January 2003, and its supplements.</p> <p>Section IV.B. "Assessment Actions," of Appendix E to 10 CFR Part 50 also requires that the initial emergency actions be discussed and agreed on by the State and local governmental authorities. The applicant should provide some form of confirmation of the agreement, such as a letter signed by State and local governmental authorities, in the emergency plan, if the applicant provides emergency action levels different from those for the existing reactor(s) on the site.</p>	<p>An EAL scheme has been developed in accordance with NEI 99-01 Rev 5.</p> <p>As identified in the response to NRC RAI No. 81, question 13.03-4, certain parameter values for the CC3 EALs required by 10 CFR 50.47(b)(4) and App. E.IV.B of 10 CFR Part 50 cannot be determined at this time. Specifically, several EAL thresholds cannot be derived until related as-built plant design information and Technical Specification set points are finalized.</p> <p>As such, UniStar has withdrawn the submitted COLA Part 5 related EAL Enclosures A, B and C and has used Option 2. Option 2 calls for the re-submittal of Section D of the E-Plan which addresses the four critical elements of an EAL scheme.</p> <p>Part 5 Enclosure D of the submittal includes signed letters from MEMA and the EPZ counties that provide agreement with the EAL scheme developed in accordance with NEI 99-01 Rev 5.</p>

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Section II. Acceptance Criteria	Section Reference/Comments
<p>4. Appendix 2, "Meteorological Criteria for Emergency Preparedness at Operating Nuclear Power Plants," to NUREG-0654/FEMA-REP-1, Rev. 1, provides guidance related to the planning standards codified in 10 CFR 50.47(b)(8) and (9) and the requirements of Section IV.E.2 of Appendix E to 10 CFR Part 50. Proposed revision 1 to Regulatory Guide 1.23, "Meteorological Programs in Support of Nuclear Power Plants," is referenced in Appendix 2 to NUREG-0654 as a source of acceptance criteria for meteorological measurements. Since Appendix 2 was issued, additional guidance related to meteorological systems has been developed. NUREG-0696, "Functional Criteria for Emergency Response Facilities," refers to the guidance in proposed Revision 1 to Regulatory Guide 1.23, Revision 2 to Regulatory Guide 1.97, and Appendix 2 to NUREG-0654/FEMA-REP-1, Rev. 1. Supplement 1 to NUREG-0737, "Clarification of TMI Action Plan Requirements," (Generic Letter 82-33) clarifies the guidance in Revision 2 of Regulatory Guide 1.97, "Instrumentation for Light-water-cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," and contains guidance related to the need to provide reliable indication of meteorological variables in the control room, Technical Support Center, and Emergency Operations Facility in the vicinity (up to about 10 miles) of the plant site. Revision 3 of Regulatory Guide 1.97 was issued in May 1983 and Revision 4 was issued in June 2006. Revision 1 to Regulatory Guide 1.23 was issued in March 2007.</p>	<p>A meteorological monitoring system that meets the regulatory requirements and guidance documents is described in E-Plan Part II Section H.5.a.1.</p>
<p>5. Supplement 1 to NUREG-0737, "Clarification of TMI Action Plan Requirements," (Generic Letter 82-33) clarifies the guidance in Revision 2 of Regulatory Guide 1.97, "Instrumentation for Light-water-cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," and contains guidance related to upgrading emergency response facilities and meeting the requirements of 10 CFR 50.47(b)(6), (8), (9) and Section IV.E of 10 CFR Part 50.</p>	<p>Information that describes the major elements for SPDS and ERFs are contained in E-Plan Part II Sections H.1, H.2, H.5.c.2 and Annex Section 4.1. Meteorological monitoring capabilities that meet the elements of the reference are described in E-Plan Part II Section H.5.a.1.</p>
<p>6. Appendix 3, "Means for Providing Prompt Alerting and Notification of Response Organizations and the Population," to NUREG-0654/FEMA-REP-1, Rev. 1, provides guidance related to 10 CFR 50.47(b)(5) and (6).</p>	<p>A public alert notification system that meets the regulatory requirements and guidance documents is described in E-Plan Part II Section E.6 and the FEMA ANS design certification for CCNPP Units 1 & 2.</p>
<p>7. Supplement 3, "Criteria for Protective Action Recommendations for Severe Accidents," to NUREG-0654/FEMA-REP-1, Rev. 1, provides guidance for the development of protective action recommendations for the public for severe reactor accidents. The guidance updates and simplifies the decision-making process for protective actions for severe reactor accidents given in Appendix 1 to NUREG-0654/FEMA-REP-1, Rev. 1.</p>	<p>A PAR process that meets the criteria of NUREG-0654 Supplement 3 is described in E-Plan Part II Sections J.7, J.9 and J.10.</p>

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Section II. Acceptance Criteria	Section Reference/Comments
<p>8. RG 1.101, Rev. 2, states that the criteria and recommendations in NUREG-654/FEMAREP-1, Rev. 1, are considered by the NRC staff to be acceptable methods for complying with the standards in 10 CFR 50.47. Except for cases in which the applicant or licensee proposes acceptable alternative methods for complying with specific portions of the regulations, the methods described in NUREG 0654/FEMA-REP-1, Rev. 1, will be used as a basis for evaluating the adequacy of the emergency plans. If an applicant proposes alternative practice or method for complying with the regulations, the application should provide an appropriate justification.</p>	<p>The CC3 E-Plan has been developed in a format consistent with the format of NUREG-0654 to specifically address each of the NUREG-0654 evaluation criteria. Alternate methods are not used.</p>
<p>9. In addition to NUREG-0654/FEMA-REP-1, Rev. 1, FEMA will evaluate State, tribal, and local government planning and preparedness on the basis of applicable policies and guidance, including approved alternative approaches and methods. FEMA will base its findings and determinations, relating to the adequacy of offsite radiological emergency planning and preparedness, on these evaluations.</p>	<p>Not applicable to this conformance evaluation.</p>
<p>10. 10 CFR 50.33(g), 10 CFR 50.47(c)(2), and Section I of Appendix E to 10 CFR Part 50 require that the size of the EPZ for a nuclear power plant shall be determined in relation to local emergency response needs and capabilities, as they are affected by such conditions as demography, topography, land characteristics, access routes, and jurisdictional boundaries. 10 CFR 52.77 requires that the COL application must contain all of the information required by 10 CFR 50.33. 10 CFR 50.33(g) requires that an applicant for an operating license submit radiological emergency response plans of State and local government entities that are wholly or partially within the 10-mile plume exposure EPZ, as well as the plans of State governments wholly or partially within the 50- mile ingestion pathway EPZ. An applicant should also submit plans for tribal governmental entities affected by the 10-mile EPZ. NUREG-0396 provides additional guidance relating to the definition of the EPZs.</p>	<p>The EPZ boundary has been previously established in accordance with the regulations for CCNPP Units 1 & 2 and is described in E-Plan Part I Section B.</p>
<p>11. Section IV of Appendix E to 10 CFR Part 50, through 10 CFR 52.79(a)(21) and 10 CFR 50.34, requires that an application for an OL or COL provide an analysis of the time required to evacuate various sectors and distances within the plume exposure pathway EPZ; i.e., an ETE. The NRC regulations do not specify a limit for such estimated evacuation times. An ETE can identify physical characteristics unique to the proposed site that could pose a significant impediment to the development of emergency plans. An ETE provides an analysis of the time required to evacuate and for taking other protective actions for various sectors and distances within the plume exposure EPZ. This information can be used by decision makers in responding to an actual emergency to aid in deciding what protective actions to implement. Appendix 4 to NUREG-0654/FEMAREP-1, Rev. 1, and Supplement 2 to NUREG-0654/FEMA-REP-1, Rev. 1, provide guidance relating to performing an ETE analysis. NUREG/CR-6863 provides additional information on ETEs.</p>	<p>The ETE study report developed in accordance with the applicable regulations and guidance documents is referenced in E-Plan Part II Section J.8 and Appendix 5, and is provided in Part 5 of the COL application.</p>

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Section II. Acceptance Criteria	Section Reference/Comments
<p>12. Section VI of Appendix E to 10 CFR Part 50 requires an emergency response data system (ERDS). The ERDS is a direct near real-time electronic data link between a licensee's onsite computer system and the NRC Operations Center, and provides for the automated transmission of a limited data set of selected parameters from a licensee's installed onsite computer system in the event of an emergency. NUREG-1394 provides the minimum standards and acceptable methods that may be used to implement and comply with the ERDS requirements.</p>	<p>An ERDS that meets the regulatory requirements and guidance documents is described in E-Plan Part II Section F.1.b-d.5.</p>
<p>13. Insofar as emergency planning and preparedness requirements are concerned, 10 CFR 50.47(d) provides that a license authorizing fuel loading and/or low-power testing and training (up to 5 percent of the rated power) may be issued after a finding is made by the NRC that the state of onsite emergency preparedness provides reasonable assurance that adequate protective measures can and will be taken in the event of a radiological emergency. The assessment of the applicant's onsite emergency plan will be based on the pertinent standards in 10 CFR 50.47(b) and the requirements of Appendix E to 10 CFR Part 50. However, the acceptability of an applicant's emergency plans will be reviewed against the standards with offsite aspects presented in 10 CFR 50.47(d)(1)-(7).</p> <p>50.47(d)(1) Arrangements for requesting and effectively using offsite assistance on site have been made, arrangements to accommodate State and local staff at the licensee's near-site Emergency Operations Facility have been made, and other organizations capable of augmenting the planned onsite response have been identified.</p> <p>50.47(d)(2) Procedures have been established for licensee communications with State and local response organizations, including initial notification of the declaration of emergency and periodic provision of plant and response status reports.</p> <p>50.47(d)(3) Provisions exist for prompt communications among principal response organizations to offsite emergency personnel who would be responding onsite.</p> <p>50.47(d)(4) Adequate emergency facilities and equipment to support the emergency response onsite are provided and maintained.</p> <p>50.47(d)(5) Adequate methods, systems, and equipment for assessing and monitoring actual or potential offsite consequences of a radiological emergency condition are in use onsite.</p>	<p>Offsite assistance used onsite is described in E-Plan Part I Section H.</p> <p>As stated in ITAAC 5.2, the EOF is a shared facility with CCNPP Units 1 & 2 and includes space to accommodate offsite responders and has been previously inspected.</p> <p>Augmenting organizations are described in E-Plan Sections B.8 and C.4.</p> <p>E-Plan Appendix 2 lists EP-AN-400, Emergency Notifications, as the procedure for the described function. Actual procedure development is a product of the ITAAC# 9.1.</p> <p>Notification of offsite personnel are described in E-Plan Part II Sections E.1-4</p> <p>Offsite EOCs are described in E-Plan Part II Section H.3.</p> <p>Radiation monitoring capabilities are described in E-Plan Part II Sections H.5.b &c, H.6.b, I.2, I.4 and I.7.</p>

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Section II. Acceptance Criteria	Section Reference/Comments
<p>50.47(d)(6) Arrangements are made for medical services for contaminated and injured onsite individuals.</p> <p>50.47(d)(7) Radiological emergency response training has been made available to those offsite who may be called to assist in an emergency onsite.</p>	<p>Medical response capabilities are described in E-Plan Part II Section L.</p> <p>Training offered to offsite agencies is described in E-Plan Part II Section O.1.</p>
<p>14. Where an applicant for an OL or COL asserts that its inability to demonstrate compliance with the offsite emergency planning requirements of 10 CFR 50.47(b) is wholly or substantially the result of the non-participation of State and/or local governments, an operating license may be issued if the applicant demonstrates to the Commission's satisfaction those elements listed in 10 CFR 50.47(c)(1)(i)-(iii). (See 10 CFR 50.47(c)(1) and 10 CFR 52.79(a)(22)(ii).) Supplement 1 to NUREG-0654/FEMA-REP-1, Rev. 1, provides guidance for the development, review, and evaluation of utility offsite radiological emergency response planning and preparedness, for those situations in which State and/or local governments decline to participate in emergency planning.</p>	<p>The state and local government response agencies support and participate in the emergency planning of CC3 as documented in the submitted LOAs.</p>
<p>15. The minimum acceptance criteria for all ESP applications, located in 10 CFR 52.17(b)(1), require that ESP applications identify physical characteristics unique to the proposed site that could pose a significant impediment to the development of emergency plans. If such physical characteristics are identified, the applicant must also identify measures that would, when implemented, mitigate or eliminate the significant impediment. Applications providing only the information required by 10 CFR 52.17(b)(1) must also include a description of contacts and arrangements (preferably letters of agreement) made with local, State, and Federal governmental agencies with emergency planning responsibilities, in accordance with 10 CFR 52.17(b)(4). The applicant may choose to submit additional emergency planning information in the ESP application to address the two options in 10 CFR 52.17(b)(2). The two options allow an ESP applicant to propose either major features of the emergency plans, or to provide complete and integrated emergency plans. While neither option is required, each would provide for a more definitive finding concerning emergency plans and preparedness at the ESP stage than would be the case for submittal of only the minimum required information. Complete and integrated emergency plans in an ESP application will be reviewed in accordance with the applicable requirements of 10 CFR 50.47 and Appendix E to 10 CFR Part 50. Supplement 2 to NUREG-0654/FEMA-REP-1, Rev. 1, provides guidance relating to emergency planning information in an ESP application.</p>	<p>Not applicable.</p> <p>CC3 has provided a complete E-Plan as part of a COL submittal (not an ESP).</p>

CC3 COL NUREG-0800 Standard Review Plan Section 13.3 Conformance Evaluation

Section II. Acceptance Criteria	Section Reference/Comments
<p>16. For an ESP application, a preliminary analysis of evacuation times is one example of how some significant impediments to the development of emergency plans may be identified. Other factors, such as the availability of adequate shelter facilities, in consideration of local building practices and land use (e.g., outdoor recreation facilities, including camps, beaches, hunting or fishing areas), and the presence of large institutional or other special needs populations (e.g., schools, hospitals, nursing homes, prisons) should also be addressed when identifying significant impediments to the development of emergency plans. Any ETE analysis or other identification of physical impediments should include the latest population census numbers and reflect the most recent local conditions. Appendix 4 to NUREG-0654/FEMA-REP-1, Rev. 1, and Supplement 2 to NUREG-0654/FEMA-REP-1, Rev. 1, provide guidance relating to performing an ETE analysis. NUREG/CR-6863 provides additional information on ETEs.</p>	<p>Not applicable. CC3 has provided a complete E-Plan as part of a COLA submittal (not an ESP).</p>
<p>17. For applications that require site approval for a stationary power reactor subject to 10 CFR Part 50 or 10 CFR Part 52 (e.g., CP, OL, ESP and COL), 10 CFR 100.1 and 10 CFR 100.21(g) require the identification of physical characteristics unique to the proposed site that could pose a significant impediment to the development of emergency plans. This siting requirement is similar to that in 10 CFR 52.17(b)(1) for an ESP application, and the means for identifying significant impediments (e.g., an analysis of evacuation times or ETE) could apply to non-ESP applications. Further, if such physical characteristics are identified, the application must also identify measures that would, when implemented, mitigate or eliminate the significant impediment. Where unfavorable physical characteristics of the site exist, the proposed site may nevertheless be found to be acceptable if the design of the facility includes appropriate and adequate compensating engineering safeguards (see 10 CFR 100.10(d), which applies to applications submitted before January 10, 1997).</p> <p>The application should provide a projection of the population within the 10-mile EPZ throughout the requested duration of the application; including a discussion of the sources of information and methodology that supports the population projection.</p> <p>The application should specifically address whether the projected population creates a significant impediment to the development of emergency plans over the requested duration of the ESP or COL application, including how it would affect the ETE. If a significant impediment is created, then the applicant should identify measures that would, when implemented, mitigate or eliminate the significant impediment. Additional site-related guidance is provided in RG 4.7, and in ESP-related guidance documents (e.g., Supplement 2 to NUREG-654/FEMA-REP-1, Rev. 1).</p>	<p>CC3 is sited next to the existing operating Units 1 & 2. No physical impediments have been identified concerning the operation of plants at this site.</p> <p>The population from the 2000 Census was projected forward to the year the application was submitted as prescribed by the NRC during the 05/11/07 public meeting.</p> <p>CC3 is sited next to the existing operating Units 1 & 2. No physical impediments have been identified concerning the operation of plants at this site.</p>

CC3 COL NUREG-0800 Standard Review Plan Section 13.3 Conformance Evaluation

Section II. Acceptance Criteria	Section Reference/Comments
<p>18. Copies of letters of agreement or other certifications, reflecting contacts and arrangements made with local, State, and Federal agencies with supporting emergency responsibilities, should be included in a CP, OL, ESP or COL application, as required by 10 CFR 52.17(b)(4), 10 CFR 52.79(a)(22), or Section II.B of Appendix E to 10 CFR Part 50.9</p> <p>The agreement information should be up-to-date when the application is submitted, and should reflect use of the proposed site for possible construction of a new reactor (or reactors).</p> <p>In addition, a discussion of the details associated with any ambiguous or incomplete language in the letters of agreement should be provided in the application. For an existing reactor site, the letters of agreement or other certifications should clearly address the presence of an additional reactor (or reactors) at the site, and any impact that would have on governmental agency or private organization emergency planning responsibilities, including acknowledgment by the agencies or organization of the proposed expanded responsibilities.</p> <p>If the applicant is unable to make arrangements with local, tribal, State, and Federal governmental agencies with emergency planning responsibilities, for whatever reason, the applicant should discuss its efforts to make such arrangements and describe any compensatory measures the applicant has taken or plans to take because of the lack of such arrangements. Supplement 1 to NUREG-654/FEMA-REP-1, Rev. 1, provides guidance for the development, review, and evaluation of utility offsite radiological emergency response planning and preparedness (i.e., a utility plan), for those situations in which State and/or local governments decline to participate in emergency planning. (See also 10 CFR 50.47(c)(1).)</p>	<p>The state and local government and private response agencies support and participate in the emergency planning of CC3 as documented in the submitted LOAs.</p> <p>The LOAs for CC3 were current at the time of submittal.</p> <p>The LOAs were written specifically for CC3, and is explicitly stated as such in the submitted LOAs.</p> <p>Not applicable.</p> <p>CC3 has obtained agreement for support arrangements with applicable agencies.</p>
<p>19. Supplement 2 to NUREG-0654/FEMA-REP-1, Rev. 1, will be used as the primary guidance for the review of emergency preparedness information and plans submitted with an ESP application pursuant to Subpart A of 10 CFR Part 52. For a pre-existing nuclear facility, all major features of the emergency plan (i.e., all 14 planning standards) identified in Supplement 2 to NUREG-0654/FEMA-REP-1, Rev. 1, should be addressed in the ESP application. The detailed, specific evaluation criteria for each of the major features in Supplement 2 should be addressed for both a pre-existing nuclear facility, as well as for applicable major features associated with a site without a pre-existing nuclear facility. If emergency planning information is not provided on all 14 major features (including the detailed, specific evaluation criteria) in Section V of Supplement 2, the ESP application will not be rejected. The review and evaluation will, however, only be based on, and specifically limited to, the submitted information that relates to the guidance in Supplement 2 of NUREG-0654/FEMA-REP-1, Rev. 1.</p>	<p>Not applicable.</p> <p>CC3 has provided a complete E-Plan as part of a COL submittal (not an ESP).</p>

CC3 COL NUREG-0800 Standard Review Plan Section 13.3 Conformance Evaluation

Section II. Acceptance Criteria	Section Reference/Comments
<p>20. The planning standards and evaluation criteria for preparing and evaluating an ESP application containing complete and integrated emergency plans are provided in NUREG-0654/FEMA-REP-1, Rev. 1. Under this ESP option, the applicant should make a good-faith effort to obtain from the government agencies certifications that (1) the proposed emergency plans are practicable; (2) these agencies are committed to participating in any further development of the plans, including any required field demonstrations; and (3) these agencies are committed to executing their responsibilities under the plans in the event of an emergency. The application must contain any certifications that have been obtained. If these certifications cannot be obtained, the application must contain information, including a utility plan pursuant to 10 CFR 50.47(c)(1), sufficient to show that the proposed plans nonetheless provide reasonable assurance that adequate protective measures can and will be taken in the event of a radiological emergency at the site. The utility-prepared emergency plans and preparedness will be reviewed and evaluated using the guidance in Supplement 1 to NUREG-0654/FEMA-REP-1, Rev. 1.</p>	<p>Not applicable. CC3 has provided a complete E-Plan as part of a COL submittal (not an ESP).</p>
<p>21. 10 CFR 52.17(b)(3) allows an applicant for an ESP, that proposes major features of the emergency plans or complete and integrated emergency plans, to include proposed ITAAC which are necessary and sufficient to provide reasonable assurance that, if the inspections, tests and analyses are performed and the acceptance criteria met, the facility has been constructed and will operate in conformity with the license, the provisions of the Atomic Energy Act, and the NRC's regulations.</p>	<p>Not applicable. CC3 has provided a complete E-Plan as part of a COL submittal (not an ESP).</p>
<p>22. 10 CFR 52.47(b)(1) allows an applicant for a design certification to include proposed ITAAC, including those applicable to emergency planning, which are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, a plant that incorporates the design certification is built and will operate in accordance with the design certification, the provisions of the Atomic Energy Act, and the NRC's regulations.</p>	<p>Not applicable. CC3 has provided a complete E-Plan as part of a COL submittal (not a DCD).</p>
<p>23. 10 CFR 52.80(a) requires that an application for a combined license includes proposed emergency planning ITAAC which are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, the facility has been constructed and will operate in conformity with the combined license, the provisions of the Atomic Energy Act, and the NRC's regulations.</p>	<p>An Emergency Planning ITAAC is included in the submitted COL application.</p>

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<p>24. Table 14.3.10-1 [of SECY 05-0197] provides an acceptable set of generic emergency planning ITAAC that an applicant may use to develop application-specific ITAAC, tailored to the specific reactor design and emergency planning program requirements. A smaller set of ITAAC is acceptable if the application contains information that fully addresses emergency preparedness requirements associated with any of the generic ITAAC in Table 14.3.10-1 that are not used. Table 14.3.10-1 is not all-inclusive, or exclusive of other ITAAC an applicant may propose. Additional plant-specific emergency planning ITAAC (i.e., beyond those listed in Table 14.3.10-1) may be proposed, and they will be examined to determine their acceptability on a case-by-case basis. In general, ITAAC are inappropriate for procedure-level details associated with the emergency plans, in that procedure adequacy and implementation can be evaluated under the exercise ITAAC, and should be limited to those aspects of emergency planning and preparedness that can not reasonably be addressed prior to construction of the plant. Each EP-ITAAC must have an objective acceptance criteria stated.</p>	<p>An Emergency Planning ITAAC is included in the submitted COL application.</p>
<p>25. For those licensees subject to 10 CFR 50.34(f), 10 CFR 50.34(f)(2)(xxv) requires that an applicant provide a TSC, OSC, and, for a CP application only, a near-site emergency operations facility (EOF) (TMI Item III.A. 1.213). NUREG-0696, Appendix B to NUREG-0718, NUREG-0737, and Supplement 1 to NUREG-0737 provide guidance relating to the design and implementation of emergency response facilities (e.g., TSC, OSC, EOF). In addition, 10 CFR 50.47(b)(8) and Subsection IV.E.8 of Appendix E to 10 CFR Part 50 requires that the design should include adequate emergency facilities and equipment to support emergency response. NUREG-0696, NUREG-0737, and Supplement 1 to NUREG-0737 provide guidance relating to occupancy and radiological habitability of vital areas (including the TSC), which aid in the mitigation of or recovery from an accident.</p>	<p>Not applicable. 10 CFR 50.34(f) applies to applicants for a light-water-reactor construction permit or manufacturing license whose application was pending as of February 16, 1982. The CC3 application is for a COL and was submitted after 02/16/82.</p>
<p>26. For those licensees subject to 10 CFR 50.34(f), 10 CFR 50.34(f)(2)(iv) requires that an applicant seeking an operating license shall provide an SPDS in both the TSC and EOF (TMI Item I.D.2). The SPDS includes the minimum set of plant parameters needed to assess the safety status of the plant in a timely manner, and is capable of indicating when process limits are being approached or exceeded. Supplement 1 to NUREG-0737, NUREG-0696, and NUREG-0814 provide guidance regarding the SPDS. (The SPDS is reviewed under SRP Sections 7.5 and 18.2.)</p>	<p>Not applicable. 10 CFR 50.34(f) applies to applicants for a light-water-reactor construction permit or manufacturing license whose application was pending as of February 16, 1982. The CC3 application is for a COL and was submitted after 02/16/82.</p>

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<p>27. For those licensees subject to 10 CFR 50.34(f), 10 CFR 50.34(f)(2)(viii) requires that an applicant provide a capability to promptly obtain and analyze samples from the reactor coolant system and containment that may contain accident source term radioactive materials, while ensuring that no individual receives radiation exposure in excess of 0.05 Sv (5 rem) to the whole body or 0.5 Sv (50 rem) to the extremities (TMI Item II.B.3).</p> <p>In addition, 10 CFR 50.47(b)(9) requires adequate methods, systems, and equipment for assessing and monitoring actual or potential offsite consequences of a radiological emergency condition. To address this regulation, the NRC has concluded that source term information should be obtained and analyzed, to continuously assess and refine dose assessments and confirm or modify initial protective action recommendations.</p> <p>Finally, 10 CFR 50.47(b)(11) requires the establishment of the means for controlling radiological exposure to emergency workers. Post-accident sampling systems are discussed in the October 31, 2000, Model Safety Evaluation, as it relates to the development of contingency plans for sampling and analysis of highly radioactive samples from the reactor coolant system, containment sump, and containment atmosphere.</p>	<p>Not applicable.</p> <p>10 CFR 50.34(f) applies to applicants for a light-water-reactor construction permit or manufacturing license whose application was pending as of February 16, 1982.</p> <p>The CC3 application is for a COL and was submitted after 02/16/82.</p>
<p>28. For those licensees subject to 10 CFR 50.34(f), 10 CFR 50.34(f)(2)(xvii) requires instrumentation to measure, record and readout of various containment parameters, including noble gas effluents at all potential, accident release points. In addition, an applicant must provide for continuous sampling of radioactive iodines and particulates in gaseous effluents from all potential accident release points, and for onsite capability to analyze and measure these samples (TMI Item II.F.1). RG 1.97 provides guidance relating to instrumentation to assess plant and environmental conditions during and following an accident.</p>	<p>Not applicable.</p> <p>10 CFR 50.34(f) applies to applicants for a light-water-reactor construction permit or manufacturing license whose application was pending as of February 16, 1982.</p> <p>The CC3 application is for a COL and was submitted after 02/16/82.</p>
<p>29. 10 CFR 50.72(a)(3) and (c)(3) require the notification of the NRC Operations Center following the declaration of an emergency in accordance with the licensee's approved emergency plans, and the establishment of an open and continuous communications channel when requested by the NRC.</p> <p>10 CFR 50.72(a)(4) establishes requirements for the activation of the ERDS following the licensee's declaration of an alert, site area emergency, or general emergency. NUREG-1022 provides the minimum standards and acceptance methods that may be used to comply with these NRC reporting requirements.</p> <p>10 CFR 73.71(a) requires the notification of the NRC Operations Center, after the discovery of an imminent or actual safeguards threat against the facility or other safeguards events. Regulatory Guide 5.62 provides the minimum standards and acceptance methods that may be used to comply with these NRC reporting requirements.</p>	<p>The NRC is notified in accordance with the applicable regulations and is documented in E-Plan Part II Sections E.2.b.2 and E.4.</p> <p>ERDS is activated in accordance with the regulations and is documented in E-Plan Part II Section F.1.b-d.5.</p> <p>Not contained in E-Plan. Refer to Safeguards Contingency Plan and/or subsequent security procedures.</p>

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<p>30. The emergency planning and preparedness standards and requirements in 10 CFR Part 50, 10 CFR Part 52, and 10 CFR Part 100 are supplemented by various generic communications and Commission Orders. Those generic communications that relate to emergency planning and are currently in effect are identified in Subsection VI (below). They provide additional guidance and criteria for meeting the relevant emergency planning standards and requirements. Any subsequently issued generic communications or Commission Orders that pertain to emergency planning and preparedness and are relevant to the application should also be addressed by the applicant.</p>	
<p>65. Administrative Letter (AL) 94-04, "Change of the NRC Operations Center Commercial Telephone & Facsimile Numbers," April 11, 1994.</p>	<p>CC3 equipment used for NRC communications is inventoried and tested quarterly per E-Plan Part II Sections F.3, H.10 and N.2.a. Current NRC phone numbers will be used.</p>
<p>66. AL 94-07, "Distribution of Site-Specific and State Emergency Planning Information," May 6, 1994.</p>	<p>This AL has been replaced by the NRC electronic library request for information contained in RIS 2006-21. Publication of the annual public information brochure will be a coordinated effort with Units 1 & 2 and will be updated to reflect the addition of Unit 3 prior to full power operation as documented in the impact evaluation submitted with Part 5.</p>
<p>67. AL 94-16, "Revision of NRC Core Inspection Program for Annual Emergency Preparedness Exercise," November 30, 1994.</p>	<p>The CC3 exercise schedule was not altered by the change in the NRC inspection program and continues to meet the requirements of 10 CFR Part 50 Appendix E, Section IV.F.2 and 3.</p>
<p>68. Bulletin (BL) 79-18, "Audibility Problems Encountered on Evacuation of Personnel from High-Noise Areas," August 7, 1979.</p>	<p>Notification of site personnel in high noise areas and out buildings is addressed in the CC3 E-Plan Part II Section J.1</p>
<p>69. BL 80-15, "Possible Loss of Emergency Notification System (ENS) with Loss of Offsite Power," June 18, 1980.</p>	<p>The CC3 E-Plan does not specifically address backup power to the FTS. Bulletin 80-15 was addressed in response to NRC RAI 105¹.</p>

¹ Greg Gibson (UniStar) to Document Control Desk (NRC) letter UN#09-277, Submittal of Response to RAI No. 105, Communications Systems, dated June 5, 2009.

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<p>70. BL 05-02, "Emergency Preparedness and Response Actions for Security-Based Events," July 18, 2005 (ADAMS Accession No. ML051740058).</p>	<p>The EAL changes prescribed by the bulletin are part of NEI 99-01 R5, which is used by CC3.</p> <p>The expedited NRC notification prescribed by the bulletin is addressed in E-Plan Part II Section E.2.b.2 and D.1.g.</p> <p>The alternate mustering location for the ERO prescribed by the bulletin is addressed in E-Plan Part II Section H.1.d.</p> <p>Specific mention of hostile action based drills is not addressed in the CC3 E-Plan and is not required to be by the bulletin. Development of hostile action based drill requirements is ongoing as part of NEI 06-04 and proposed EP related regulation. Any new enacted regulation will be addressed in the CC3 E-Plan when it is enacted.</p>
<p>71. Generic Letter (GL) 82-33, "Supplement 1 to NUREG-0737 – Requirements for Emergency Response Capability (Generic Letter 82-33)," December 17, 1982.</p>	<p>This reference is addressed by the CC3 E-Plan. Refer to Acceptance Criteria response #2 for information related to NUREG-0737.</p>
<p>72. GL 91-14, "Emergency Telecommunications," September 23, 1991 (ADAMS Accession No. ML031140150).</p>	<p>This GL describing the 1991 upgrade to the FTS is made obsolete by the changes to the system described in RIS 2000-11.</p> <p>The ENS is described in E-Plan Part II Section F.1.f with routine testing requirements contained in Section N.2.a.</p>
<p>73. Information Notice (IN) 81-34, "Accidental Actuation of Prompt Public Notification System," November 16, 1981.</p>	<p>The CC3 PANS system is designed with a signal encoder to prevent inadvertent siren actuation from radio interference.</p>
<p>74. IN 85-41, "Scheduling of Pre-Licensing Emergency Preparedness Exercises," May 25, 1985.</p>	<p>Exercise completion prior to full power operation is contained in the EP ITAAC.</p>
<p>75. IN 85-44, "Emergency Communication System Monthly Test," May 30, 1985.</p>	<p>The ENS and HPN are tested monthly in accordance with regulation as documented in E-Plan Part II Section N.2.a.</p>

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76. IN 85-52, "Errors in Dose Assessment Computer Codes and Reporting Requirements Under 10 CFR Part 21," July 10, 1985.	The CC3 dose model will be similar to the NRC RASCAL model as described in E-Plan Part II Section I.4. Any identified errors in the code, should they be found, would be reported in accordance with regulations.
77. IN 85-80, "Timely Declaration of an Emergency Class, Implementation of an Emergency Plan, and Emergency Notifications," October 15, 1985.	The CC3 E-Plan does not state the 15 minute classification goal of EPPOS-2 and NEI 99-02. Timely local, state and federal notifications are in accordance with regulations and contained in E-Plan Part II Section E.2.b.
78. IN 86-18, "NRC On-Scene Response During a Major Emergency," March 26, 1986.	Adequate accommodations for the NRC site team in the TSC are documented in E-Plan Part II Section H.1.b and E-Plan Annex Section 4.1.B. Adequate accommodations for the NRC site team in the emergency response facilities are described in the E-Plan Part II Section C.1.c.
79. IN 86-43, "Problems with Silver Zeolite Sampling of Airborne Radioiodine," June 10, 1986.	E-Plan and annex references to the use of silver zeolite sample media is limited to portable equipment that would not be used in high hydrogen environments. Refer to E-Plan Part II Table H-1 and Annex Section 4.2.B.1.e.
80. IN 86-55, "Delayed Access to Safety-Related Areas and Equipment During Plant Emergencies," July 10, 1986.	Operator access into controlled area in not addressed in the E-Plan. Personnel access to controlled areas within the plant is governed during normal operations and events in accordance with plant procedures.
81. IN 86-98, "Offsite Medical Services," December 2, 1986.	Medical services are provided in accordance with FEMA GM MS-1 as described in E-Plan Part II Section L.
82. IN 87-54, "Emergency Response Exercises (Off-Year Exercises)," October 23, 1987.	More realistic event scenarios described in the information notice are conducted during off-year exercises as addressed in E-Plan Part II Section N.1.b.

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83. IN 87-58, "Continuous Communications Following Emergency Notification," November 16, 1987.	Continuous communication capabilities provided in accordance with regulations are addressed in E-Plan Part II Sections B.1 (onshift) and E.4.
84. IN 88-15, "Availability of U.S. Food and Drug Administration (FDA)-Approved Potassium Iodide for Use in Emergencies Involving Radioactive Iodine," April 18, 1988.	This IN concerns non-reactor licensees using FDA-approved "radiation emergency potassium iodide" as no policy exists for this group. Policy for reactor licensees is provided in Federal Register, Vol. 50, No. 142, p. 30258, July 24, 1985. The storage and use of KI is documented in CC3 E-Plan Part II Sections H.1.b, H.9, H.10 and J.6.
85. IN 89-72, "Failure of Licensed Senior Operators to Classify Emergency Events Properly," October 24, 1989.	The issue raised in the IN concerns SRO classification errors caused by implementing procedures that are not "user friendly" and to ineffective training methods. CC3 will implement the latest version of NEI 99-01 EALs as described in E-Plan Part II Section D and Annex Section 3 as stated in Part 5 of the submittal. Operator and Emergency Director EAL classification performance will be evaluated during designated training sessions, drills and exercises.
86. IN 90-74, "Information on Precursors to Severe Accidents," December 4, 1990.	The applicable accident sequence precursor study report events referenced in the IN are addressed within the appropriate EAL technical basis sections included in Part 5 of the submittal.
87. IN 91-64, "Site Area Emergency Resulting from a Loss of Non-Class 1E Uninterruptible Power Supplies," October 9, 1991.	Common mode failure of uninterruptible power supplies used in nonsafety-related applications is not specifically addressed in the CC3 E-Plan, although EALs have been developed to bound loss of power and loss of indication events as documented in Part 5 of the submittal.
88. IN 91-64, Supp. 1, "Supplement 1, Site Area Emergency Resulting from a Loss of Non-Class 1E Uninterruptible Power Supplies," October 7, 1992.	The E-Plan does not reference replacement intervals for the Exide Electronics, Incorporated (Exide) 75 KVA uninterruptible power supply (UPS) model No. 575-60T3-120/208 or preventative maintenance requirements for them.

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89. IN 91-77, "Shift Staffing at Nuclear Power Plants," November 26, 1991.	On shift staffing requirements that meet regulations and NUREG-0654 guidance are described in E-Plan Annex Table B-1a.
90. IN 92-32, "Problems Identified with Emergency Ventilation Systems for Near-Site (Within 10 Miles) Emergency Operations Facilities and Technical Support Centers," April 29, 1992.	<p>The issue of inadequate maintenance and testing of EOF and TSC emergency ventilation systems will be addressed in plant maintenance and testing procedures.</p> <p>The TSC habitability design is described in E-Plan annex Section 4.1.B.</p> <p>The EOF location and habitability design considerations are not contained in the E-Plan; however the CCNPP EOF is a shared facility for CNNPP U1 &2 and U3 and was previously inspected for U 1 & 2.</p>
91. IN 92-38, "Implementation Date for the Revision to the EPA Manual of Protective Action Guides and Protective Actions for Nuclear Incidents (EPA-400-R-92-001)," May 12, 1992.	<p>Not applicable.</p> <p>EPA 400-R-92-001, October 1991, is used throughout the CC3 E-Plan as a basis for protective action concerning emergency workers and the public.</p>
92. IN 93-53, "Effect of Hurricane Andrew on Turkey Point Nuclear Generating Station and Lessons Learned," July 20, 1993.	<p>The Hurricane Andrew lessons learned applicable to the emergency plan regarding loss of communications is addressed in E-Plan Part II Section F.1.</p> <p>Examples of the CC3 communications network include systems such as normal and dedicated telephone lines on landlines, microwave and fiber-optic voice channels, cell phones, satellite phones, base and mobile radio units, and computer peripherals.</p>
93. IN 93-81, "Implementation of Engineering Expertise on Shift," October 12, 1993.	On shift staffing requirements for the STA are described in E-Plan Annex Table B-1a.
94. IN 93-94, "Unauthorized Forced Entry into the Protected Area at Three Mile Island Unit 1 on February 7, 1993," December 9, 1993.	The ability of an intruder to drive an unauthorized vehicle into the protected area has been significantly reduced following the post 9/11 order and physical security upgrade requirements. The results of the security order enhancements and plans for response to security related events are contained in the site security plan.

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95. IN 94-27, "Facility Operating Concerns Resulting from Local Area Flooding," March 31, 1994.	Hydrological monitor are discussed in E-Plan Part II Section H.5.a.3. Flooding conditions (internal and external) are addressed in the EALs.
96. IN 95-23, "Control Room Staffing Below Minimum Regulatory Requirements," April 24, 1995.	On shift staffing requirements that meet regulations and NUREG-0654 guidance are described in E-Plan Annex Table B-1a.
97. IN 95-48, "Results of Shift Staffing Study," October 10, 1995.	The capabilities of the on-shift staff to respond to an event are addressed in E-Plan Part II Section B.1.
98. IN 96-19, "Failure of Tone Alert Radios to Activate When Receiving a Shortened Activation Signal," April 2, 1996.	<p>The primary public alerting method for CC3 is sirens and EAS. Tone alert radios are one of several supplemental options included in E-Plan Part II Section E.6.</p> <p>If tone alert radios are used they will be new products (post FAA change to the EAS) and tested in accordance with manufacturer's specifications to ensure that the activation signal is sufficient to operate the equipment.</p>
99. IN 97-05, "Offsite Notification Capabilities," February 27, 1997.	<p>Examples of the CC3 communications network include systems such as normal and dedicated telephone lines on landlines, microwave and fiber-optic voice channels, cell phones, satellite phones, base and mobile radio units, and computer peripherals; which are addressed in E-Plan Part II Section F.1.</p> <p>Post 9/11 and the new nation response framework adopting the NIMS have improved the offsite communications systems capability and reliability in the areas surrounding CC3.</p>
100. IN 98-20, "Problems with Emergency Preparedness Respiratory Programs," June 3, 1998.	<p>Training and qualification of personnel who may use respiratory protection equipment is described in E-Plan Part II Sections O.4.e and O.5.b.</p> <p>Equipment availability and storage is discussed in E-Plan Part II Sections H.1.b, H.1.c and J.6.</p> <p>Primary responsibility for the respiratory protection program is the responsibility of the radiation protection department as described in E-Plan Part II Section J.6.</p>

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<p>101. IN 02-14, "Ensuring a Capability to Evacuate Individuals, Including Members of the Public, from the Owner-Controlled Area," April 8, 2002.</p>	<p>The notification of personnel outside the protected area but within the owner controlled area is addressed in E-Plan Part II Section J.1.</p> <p>Evacuation of the non-essential personnel from the site is addressed in E-Plan Part II Section J.2</p>
<p>102. IN 02-25, "Challenges to Licensees' Ability to Provide Prompt Public Notification and Information During an Emergency Preparedness Event," August 26, 2002.</p>	<p>The siren system is tested and maintained commensurate with FEMA operability requirements as referenced in FEMA-REP-10 per E-Plan Part II Section E.6.</p> <p>Siren test frequency includes weekly silent tests as documented in E-Plan Part II Section F.3, although the system includes a continuous feedback monitoring capability.</p> <p>CC3 does not use PHADS.</p> <p>EAS activation is controlled and implemented by offsite governmental agencies as described in E-Plan Part II Sections E.5, E.6 and E.7.</p> <p>ERO notifications are performed primarily with pagers as described in E-Plan Part II Section F.1.e.</p>
<p>103. IN 04-19, "Problems Associated with Back-up Power Supplies to Emergency Response Facilities and Equipment," November 4, 2004.</p>	<p>Documentation of back-up power to the emergency response facilities is not contained in the E-Plan.</p> <p>Back-up power to the TSC is described in the FSAR and DCD.</p> <p>The CC3 EOF is a shared facility. It is currently an operation facility for Units 1 & 2 and was previously inspected for those units.</p>
<p>104. IN 05-06, "Failure to Maintain Alert and Notification System Tone Alert Radio Capability," March 30, 2005.</p>	<p>The Public Alert and Notification System (PANS) consist of fixed sirens as described in E-Plan Part II Section E.6.</p> <p>If tone alert radios are used they will be maintained and tested in accordance with the applicable regulations and guidance documents.</p>

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105. IN 05-19, "Effect of Plant Configuration Changes on the Emergency Plan," July 18, 2005.	Changes to procedures, equipment, and facilities are controlled by the station's records management process where an impact on 50.59, security, emergency planning, QA and other programs are evaluated to determine whether they create a reduction in effectiveness.
106. Regulatory Issue Summary (RIS) 2000-08, "Voluntary Submission of Performance Indicator Date," March 29, 2000 (ADAMS Accession No. ML003685821).	Discussion of the voluntary submission of PI data is not applicable to the E-Plan, however CC3 will participate in the submission of information consistent with Units 1 & 2 following construction and operation.
107. RIS 2000-11, "NRC Emergency Telecommunications System," June 30, 2000 (ADAMS Accession No. ML003727812).	NRC communications systems are addressed in E-Plan Part II Section F.1.f with monthly testing requirements documented in Section H.2.a.
108. RIS 2000-11, Supp. 1, "NRC Emergency Telecommunications System," March 22, 2001 (ADAMS Accession No. ML010570103).	NRC communications systems are addressed in E-Plan Part II Section F.1.f with monthly testing requirements documented in Section H.2.a.
109. RIS 2001-16, "Update of Evacuation Time Estimates," August 1, 2001 (ADAMS Accession No. ML012070310).	The ETE study report applicable to CC3 was issued in 2002 in accordance with the applicable regulations and guidance documents. The ETE study is referenced in E-Plan Part II Section J.8 and Appendix 5, and is provided in Part 5 of the COL application.
110. RIS 2002-01, "Changes to NRC Participation in the International Nuclear Event Scale," January 14, 2002 (ADAMS Accession No. ML013200502).	The International Nuclear Event Scale is not applicable to licensees. CC3 will continue to report events in accordance with the regulations as specified in the RIS as addressed in E-Plan Part II Section D.1
111. RIS 2002-16, "Current Incident Response Issues," September 13, 2002 (ADAMS Accession No. ML022560256).	Identification of a radioactive release during event notification is defined and documented in E-Plan Part II Section E.3. The ENS Communicator is provided access to the information necessary to perform their function.
112. RIS 2002-21, "National Guard and Other Emergency Responders Located in the Licensee's Controlled Area," November 8, 2002 (ADAMS Accession No. ML023160020).	Not applicable. National Guard or state/local law enforcement organizations are no longer stationed at the nuclear power plants in Maryland.

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<p>113. RIS 2003-12, "Clarification of NRC Guidance for Modifying Protective Actions," June 24, 2003 (ADAMS Accession No. ML031680611).</p>	<p>The CC3 E-Plan does not explicitly state that areas previously recommended to be evacuated are retained when new PARs are issued for wind shifts, although that is the current integrated practice with the state of Maryland.</p> <p>Additional changes to the CC3 E-Plan regarding PARs will be reviewed following issuance of the pending regulatory changes to PARs to ensure consistency with the new rule.</p>
<p>114. RIS 2003-18, "Use of NEI 99-01, "Methodology for Development of Emergency Action Levels," Revision 4, Dated January 2003," October 8, 2003 (ADAMS Accession No. ML032580518).</p>	<p>Not applicable.</p> <p>CC3 will be implementing NEI 99-01 Rev 5 based EALs (not Rev 4 described in the RIS) under full submittal as their initial set.</p>
<p>115. RIS 2003-18, Supp. 1, "Supplement 1, Use of Nuclear Energy Institute (NEI) 99-01, "Methodology for Development of Emergency Action Levels," Revision 4, Dated January 2003," July 13, 2004 (ADAMS Accession No. ML041550395).</p>	<p>Not applicable.</p> <p>CC3 will be implementing NEI 99-01 Rev 5 based EALs (not Rev 4 described in the RIS) under full submittal as their initial set.</p>
<p>116. RIS 2003-18, Supp. 2, "Supplement 2, Use of Nuclear Energy Institute (NEI) 99-01, "Methodology for Development of Emergency Action Levels," Revision 4, Dated January 2003," December 12, 2005 (ADAMS Accession No. ML051450482).</p>	<p>Not applicable.</p> <p>CC3 will be implementing NEI 99-01 Rev 5 based EALs (not Rev 4 described in the RIS) under full submittal as their initial set.</p>
<p>117. RIS 2004-07, "Release of Final Review Standard (RS)-002, Processing applications for Early Site Permits," May 19, 2004</p>	<p>Not applicable.</p> <p>CC3 is a COL application without an ESP.</p>
<p>118. RIS 2004-13, "Consideration of Sheltering in Licensee's Range of Protective Action Recommendations," August 2, 2004 (ADAMS Accession No. ML041210046).</p>	<p>Sheltering is considered as a PAR option as illustrated in E-Plan Figure J-1; however E-Plan Part II Section J.10.m.1 does not contain detailed considerations for sheltering.</p> <p>Additional changes to the CC3 E-Plan regarding PARs will be reviewed following issuance of the pending regulatory changes to PARs to ensure consistency with the new rule.</p>
<p>119. RIS 2004-13, Supp. 1, "Consideration of Sheltering in Licensee's Range of Protective Action Recommendations, Dated August 2004," March 10, 2005 (ADAMS Accession No. ML050340531).</p>	<p>Not applicable.</p> <p>The 90 day period provided by the RIS has expired.</p>

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<p>120. RIS 2004-15, "Emergency Preparedness Issues: Post 9/11," (Official Use Only – See RIS 2006-02), October 18, 2004.</p>	<p>State/local and NRC event notifications are made in accordance to the time requirements specified by regulations as described in E-Plan Part II Section E.2.b.</p> <p>The E-Plan does not restrict security from contacting LLEA without shift manager approval.</p> <p>Potential increased demand on LLEA resources has been addressed by the offsite agencies in support of the current operating units located at the site.</p>
<p>121. RIS 2004-15, Supp. 1, "Emergency Preparedness Issues: Post-9/11," May 25, 2006 (ADAMS Accession No. ML053000046).</p>	<p>Changes to procedures, equipment, and facilities are controlled by the station's records management process where an impact on 50.59, security, emergency planning, QA and other programs are evaluated to determine whether they create a reduction in effectiveness.</p>
<p>122. RIS 2005-02, "Clarifying the Process for Making Emergency Plan Changes," February 14, 2005 (ADAMS Accession No. ML042580404).</p>	<p>Changes to the emergency plan are evaluated in accordance with 10 CFR 50.54(q) as described in E-Plan Part II Section P.4.</p>
<p>123. RIS 2005-08, "Endorsement of Nuclear Energy Institute (NEI) Guidance 'Range of Protective Actions for Nuclear Power Plant Incidents,'" June 6, 2005 (ADAMS Accession No. ML050870432).</p>	<p>The E-Plan does not contain PARs that include consideration for sheltering due to impediments. Shelter PARs for impediments are usually developed by the offsite agencies as described in E-Plan Part II Section J.10.m.4 paragraph 4.</p>
<p>124. RIS 2006-02, "Good Practices for Licensee Performance During the Emergency Preparedness Components of Force-On-Force Exercises," February 23, 2006 (ADAMS Accession No. ML052970294).</p>	<p>Not applicable.</p> <p>The scenario development and conduct recommendations are appropriate to drill development guidelines, not the E-Plan.</p>
<p>125. RIS 2006-03, "Guidance on Requesting an Exemption from Biennial Emergency Preparedness Exercise Requirements," February 24, 2006 (ADAMS Accession No. ML053390039).</p>	<p>The CC3 E-Plan does not contain information on application for exemption from biennial exercise requirements.</p> <p>Requests for exemption from scheduled evaluated exercises will be addressed on a case basis using the most current guidance at the time of the request.</p>

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Section II. Acceptance Criteria	Section Reference/Comments
126. RIS 2006-12, "Endorsement of Nuclear Energy Institute Guidance "Enhancements to Emergency Preparedness Programs for Hostile Action", July 19, 2006 (ADAMS Accession No. ML061530290).	Not applicable. Participation in the pilot hostile action based drills is scheduled to occur from 2006 to 2009 for operating reactors.
127. Emergency Preparedness Position (EPPOS) No. 1, Rev. 0, "Acceptable Deviations from Appendix 1 of NUREG-0654 Based Upon the Staff's Regulatory Analysis of NUMARC/NESP-007, "Methodology for Development of Emergency Action Levels", June 1, 1995 (ADAMS Accession No. ML022970165).	Not applicable. CC3 will implement the latest version of NEI 99-01 EALs as described in E-Plan Part II Section D and Annex Section 3.
128. EPPOS No. 2, "Timeliness of Classification of Emergency Condition," August 1, 1995.	The CC3 E-Plan does not state the 15 minute classification goal of EPPOS-2 and NEI 99-02, although it provides reference to such in Appendix 1. Changes to the CC3 E-Plan regarding the 15 minute classification goal will be reviewed following issuance of the pending regulatory changes emergency classification timeliness to ensure consistency with the new rule.
129. EPPOS No. 3, "Requirement for Onshift Dose Assessment Capability, November 8, 1995.	The CC3 E-Plan annex table B-1a does not specify the capability for onshift dose assessment as stated in E-Plan appendix 1. Changes to the CC3 E-Plan regarding the function of onshift dose assessment will be reviewed following issuance of the pending regulatory changes to onshift staffing to ensure consistency with the new rule.
130. EPPOS No. 5, "Emergency Planning Information Provided to the Public," December 4, 2002.	The content of information disseminated annually to the public is consistent with the EPPOS as described in E-Plan Part II Section G.1.
131. Circular (CR) 80-09, "Problems with Plant Internal Communications Systems," April 28, 1980.	Not applicable. Power supplies to internal plant communications systems and areas susceptible to portable radio transmission signals are addressed in plant procedures.

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Section II. Acceptance Criteria	Section Reference/Comments
<p>31. For COL reviews, the description of the operational program and proposed implementation milestone(s) for the Emergency Planning program are reviewed in accordance with 10 CFR 50.47, Part 50 Appendix E. The implementation milestones are as follows:</p> <p>full participation exercise conducted within 2 years of scheduled date for initial loading of fuel per 10 CFR 50, Appendix E.IV.F.2a(ii); onsite exercise conducted within 1 year before the schedule date for initial loading of fuel per 10 CFR Part 50, Appendix E.IV.F.2a(ii);</p> <p>and applicant's detailed implementing procedures for its emergency plan submitted no less than within 180 days prior to scheduled date for initial loading of fuel per 10 CFR Part 50, Appendix E.V.</p>	<p>As documented in ITAAC# 8.1, a full participation exercise (test) will be conducted within the specified time periods of Appendix E to 10 CFR Part 50.</p> <p>As documented in ITAAC# 9.1, detailed implementing procedures for the onsite emergency plan will be submitted no less than 180 days prior to fuel load.</p>