

PMSTPCOL PEmails

From: Eudy, Michael
Sent: Friday, July 31, 2009 12:47 PM
To: Puleo, Frederick; Stephens, Scot
Cc: STPCOL
Subject: RE: 7 Requests For Additional Information
Attachments: ML092110186.pdf

Sure thing, just did not want to be redundant.

From: Puleo, Frederick [mailto:fjpuleo@STPEGS.COM]
Sent: Friday, July 31, 2009 12:20 PM
To: Eudy, Michael; Stephens, Scot
Cc: STPCOL
Subject: RE: 7 Requests For Additional Information

Michael would it be possible to get a copy of the letter?

From: Eudy, Michael [mailto:Michael.Eudy@nrc.gov]
Sent: Friday, July 31, 2009 10:37 AM
To: Stephens, Scot; Puleo, Frederick
Cc: STPCOL
Subject: FW: 7 Requests For Additional Information

Scot and Fred,

Heads up that STP Chapter 12 RAI letter #180 was included in this batch sent out.

Michael A. Eudy - Project Manager
U.S. Nuclear Regulatory Commission
NRO/DNRL/NGE1&2
301-415-3104

From: Nagel, Cheri
Sent: Thursday, July 30, 2009 3:12 PM
To: 'CCChappell@STPEGS.com'; 'Mookhoek, William'; STPCOL
Subject: 7 Requests For Additional Information

Please see attached. Thank You.

Cheryl A. Nagel (Cheri)
Secretary
ESBWR/ABWR Projects 1 (NGE1)
Division of New Reactor Licensing
Office of New Reactors
301-415-1126

Hearing Identifier: SouthTexas34Public_EX
Email Number: 1578

Mail Envelope Properties (3D27D29AB75BCD4BAE913B63CBFBEDFDE75B20D87)

Subject: RE: 7 Requests For Additional Information
Sent Date: 7/31/2009 12:46:38 PM
Received Date: 7/31/2009 12:46:40 PM
From: Eudy, Michael

Created By: Michael.Eudy@nrc.gov

Recipients:
"STPCOL" <STP.COL@nrc.gov>
Tracking Status: None
"Puleo, Frederick" <fjpuleo@STPEGS.COM>
Tracking Status: None
"Stephens, Scot" <scstephens@STPEGS.COM>
Tracking Status: None

Post Office: HQCLSTR01.nrc.gov

Files	Size	Date & Time
MESSAGE	1131	7/31/2009 12:46:40 PM
ML092110186.pdf	89690	

Options
Priority: Standard
Return Notification: No
Reply Requested: No
Sensitivity: Normal
Expiration Date:
Recipients Received:

July 30, 2009

Mr. Scott Head, Manager
Regulatory Affairs
STP Nuclear Operating Company
P. O. Box 289
Wadsworth, TX 77483

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION LETTER NO. 180 RELATED TO
SRP SECTION 12.03-12.04 FOR THE SOUTH TEXAS PROJECT COMBINED
LICENSE APPLICATION

Dear Mr. Head

By letter dated September 20, 2007, STP Nuclear Operating Company (STP) submitted for approval a combined license application pursuant to 10 CFR Part 52. The U. S. Nuclear Regulatory Commission (NRC) staff is performing a detailed review of this application to enable the staff to reach a conclusion on the safety of the proposed application.

The NRC staff has identified that additional information is needed to continue portions of the review. The staff's request for additional information (RAI) is contained in the enclosure to this letter.

To support the review schedule, you are requested to respond within 30 days of the date of this letter. If changes are needed to the safety analysis report, the staff requests that the RAI response include the proposed wording changes.

S. Head

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If you have any questions or comments concerning this matter, I can be reached at 301-415-3104 or by e-mail at Michael.Eudy@nrc.gov or you may contact George Wunder at 301-415-1494 or George.Wunder@nrc.gov.

Sincerely,

/RA/

Michael Eudy, Project Manager
ABWR Projects Branch
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-012
52-013

eRAI Tracking Nos. 3115, 3116, 3117, and 3119

Enclosure:
Request for Additional Information

cc: William Mookhoek
Scott Stephens

S. Head

-2-

If you have any questions or comments concerning this matter, I can be reached at 301-415-3104 or by e-mail at Michael.Eudy@nrc.gov or you may contact George Wunder at 301-415-1494 or George.Wunder@nrc.gov.

Sincerely,

/RA/

Michael Eudy, Project Manager
ABWR Projects Branch
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-012
52-013

eRAI Tracking Nos. 3115, 3116, 3117, and 3119

Enclosure:
Request for Additional Information

cc: William Mookhoek
Scot Stephens

Distribution:

PUBLIC
NGE 1/2 R/F
GWunder, NRO
BAbeywickrama, NRO
RKellner NRO
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RidsNroDnrINge2

ccchappell@STPEGS.com
STP.COL@nrc.gov
wemookhoek@STPEGS.com

ADAMS Accession No. ML092110186

NRO-002

OFFICE	CHPB/TR	CHPB/BC	NGE2/PM	OGC	NGE2/L-PM
NAME	RKellner	TFrye	TTai	SKirkwood	GWunder
DATE	6/16/09	6/16/09	6/22/09	7/22/09	7/30/09

***Approval captured electronically in the electronic RAI system.**

OFFICIAL RECORD COPY

Request for Additional Information No. 3115 Revision 2

7/31/2009

South Texas Project Units 3 and 4
South Texas Project Nuclear Operating Co
Docket No. 52-012 and 52-013
SRP Section: 12.03-12.04 - Radiation Protection Design Features
Application Section: 12.3.1.4.4

QUESTIONS for Health Physics Branch (CHPB)

12.03-12.04-5

Section 12.3.1.4.4 of the COL FSAR states that a lead-loaded silicone foam or equivalent is employed whenever possible for penetrations through steam tunnel walls to reduce the available streaming area presented. However, no information, or reference to a material specification located elsewhere in the FSAR, is provided concerning the properties and radiation shielding characteristics of the materials for sealing the penetrations. In accordance with NUREG-0800 and RG-1.206, C.I.12.3.2, please provide the required radiation shielding properties of the material to be used, or a reference to the applicable section of the FSAR in which the information is located.

Request for Additional Information No. 3116 Revision 2

7/31/2009

South Texas Project Units 3 and 4
South Texas Project Nuclear Operating Co
Docket No. 52-012 and 52-013
SRP Section: 12.03-12.04 - Radiation Protection Design Features
Application Section: 12.3

QUESTIONS for Health Physics Branch (CHPB)

12.03-12.04-6

Section 12.3 of the COL FSAR references Turbine Building Radiation Area Zone maps, Figures 12.3-49 thru 12.3-53, 12.3-55, 12.3-70 thru 12.3.73, and 12.3-75 thru 12.3-77, and Table 12.3-7, Area Radiation Monitors Turbine Building, for the location and sensitivity ranges of the turbine building area radiation monitors. Additional information is needed by the staff concerning placement and monitor sensitivity ranges. In accordance with NUREG-0800 and RG-1.206 C.I.12.3.4, please provide the following additional information concerning the monitors:

- 1) How placement of the monitors was determined.
- 2) How the specified sensitivity ranges were determined.

12.03-12.04-7

ABWR DCD Section 12.3.1.4, Implementation of ALARA, states that the Reactor Water Cleanup (CUW) System backwash tank is vented through a charcoal filter canister to reduce the emission of radioiodines into the plant atmosphere. STD DEP 12.3-2 states that the text was revised to delete the ABWR DCD description of a charcoal filter canister on the backwash tank vent line. The applicant states that the current design intent is for the CUW backwash tank to be vented to the Reactor Building HVAC exhaust.

Operating experience has shown that venting tank and demineralizer vents directly into HVAC systems with no filtration, or moisture or resin traps, could result in intrusion of moisture or particulate matter into ventilation systems resulting in contamination of facilities or the environment. There is no discussion in the STP COL FSAR or in the Departures Report (Part 7 of the STP COL) to address this operating experience. In accordance with 10 CFR 20.1406(a), the COL applicant is responsible for documenting in the application how facility design and procedures for operation will minimize, to the extent practicable, contamination of the facility and the environment, facilitate eventual decommissioning, and minimize, to the extent practicable, the generation of radioactive waste.

Please describe what design feature(s) will be included in the backwash tank vent line that will prevent or mitigate contamination of the ductwork and facility. If design features are not used, then provide the specific alternate approaches used and the associated justification.

12.03-12.04-8

NUREG-0800 12.3-3 and Regulatory Guide 1.206 C.I.12.3.5 10, CFR 50.34(f)(2)(vii) and NUREG-0737 II.B.2, note that vital areas which require access by operators aiding, mitigating or recovering from the accident, need to be identified. NUREG-0737 II.B.2 notes that this is applicable to vital areas and equipment, other than the Control Room, such as the radwaste control stations, emergency power supplies, motor control centers, and instrument areas. These criteria are applicable even if the areas are accessed on an irregular basis and not continuously occupied for the duration of the event. Some of the areas that may reasonably be expected to require access include but are not limited to:

- Radwaste Control Panel
- Safety Injection Pumps
- SFP area
- Residual Heat Removal Pumps

Section 12.3 of the COL FSAR contains insufficient location, exposure rate, occupancy time information, and the associated mission doses, for those areas that plant operators or radiation protection personnel may need to access as noted above. In order to make a determination of reasonable assurance that the post-accident dose limit of 0.05 Sv (5 rem) for the duration of the accident can be met, additional information is needed by the staff.

In accordance with 10 CFR 50 GDC-19, NUREG-0800 and RG-8.8 C.I.12.3.5, please include figures, tables, and discussion in FSAR Section 12.3 that clearly identifies likely post-accident locations that will require access, travel pathways, occupancy times, and the exposure rate values associated with plant operation and monitoring, for the duration of the event, or provide the specific alternative approaches used and the associated justification.