



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 6, 2009

Mr. Dave Baxter
Vice President, Oconee Site
Duke energy Carolians, LLC
7800 Rochester Highway
Seneca, SC 29672

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (OCONEE),
PRESSURIZED-WATER REACTOR OWNERS GROUP TOPICAL REPORT
BAW-2374, REVISION 2, "RISK-INFORMED ASSESSMENT OF
ONCE-THROUGH STEAM GENERATOR TUBE THERMAL LOADS DUE TO
BREAKS IN REACTOR COOLANT SYSTEM UPPER HOT LEG LARGE-BORE
PIPING" (TAC NO. ME1816)

Dear Mr. Baxter:

On June 25, 2009, a meeting was held between representatives of the U.S. Nuclear Regulatory Commission (NRC) staff and the Pressurized-Water Reactor Owners Group regarding the proposed resolution of items related to BAW-2374, Revision 2, "Risk-Informed Assessment of Once-Through Steam Generator Tube Thermal Loads Due to Breaks in Reactor Coolant System Upper Hot Leg Large-Bore Piping." A summary of the meeting can be found at Agencywide Documents Access and Management System Accession No. ML091820001. During this meeting, representatives of Duke Energy Carolinas, LLC (Duke), participated in a presentation relating to the status of this matter as it relates to Oconee.

The NRC staff requests that Duke provide a written response to the enclosed list of actions taken from the meeting summary of the June 25, 2009, meeting. This response should describe Duke's plans to address these items and will enable the staff to evaluate the need for any further regulatory action. A response is requested within 30 days from the date of this letter.

If you have any questions, please call me at 301-415-1345.

Sincerely,

A handwritten signature in cursive script that reads "John Stang" with "for" written below it.

John Stang, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, and 50-287

Enclosure:
As stated

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ACTIONS RESULTING FROM JUNE 25, 2009, MEETING
BETWEEN U.S. NUCLEAR REGULATORY COMMISSION
AND PRESSURIZED-WATER REACTOR OWNERS GROUP

The U.S. Nuclear Regulatory Commission (NRC) requests that within 30 days from the date of this letter each Babcock and Wilcox licensee submit a letter providing plans to address the following items resulting from the June 25, 2009, public meeting regarding once-through steam generator (SG) tube loads under conditions resulting from postulated breaks in reactor coolant system upper hot leg large bore piping:

- 1) Confirmation that its justification for continued operation (JCO) for addressing tube integrity following a large break loss-of-coolant accident (LBLOCA) remains valid.
- 2) Confirmation that compensatory measures, such as changes to emergency operating procedures, have been incorporated into plant procedures and operator training has been performed.
- 3) Confirmation that Title 10 of the *Code of Federal Regulations*, Part 50, Section 50.46(a)(3) reporting requirements have been satisfied.
- 4) Confirmation that all LBLOCAs (including those in the candy-cane region) are considered as design-basis accidents in the assessments of SG tube integrity following each SG tube inspection.
- 5) Confirmation that the design of the replacement SGs is sufficient to withstand the loads associated with an LBLOCA including the thermal loads associated with an LBLOCA in the candy-cane region of the reactor coolant system.
- 6) Commitment to provide the structural limit associated with the most-limiting LBLOCA for the replacement SGs as part of the next SG tube inspection report (required by the technical specifications) following completion of the next inspection of the tubes in the replacement SGs, unless previously submitted.

Enclosure

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Sincerely,

/RA J. Thompspon for/

John Stang, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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