Appenaix	ט		Sce	enario Outline	Form ES-D-1
Facility:	<u>Byron</u> S	cena	rio No: #	1 Op-Test No: <u>20</u>	<u>008-01</u>
Examin	ers:			Operators: _	
Initial C	onditions: Rx Pc	wer -	~ 100%.		
Turnove	er: Drain PRT pe	er BO	P RY-4 a	nd then ramp down to 95	% Rx Power.
Event No.	Malf. No.		Event Type*	Do	Event escription
1	NONE	N	RO SRO	Drain PRT per RY-4, Dr	aining the PRT
2	NONE	R	RO SRO	Ramp from 100% down	to 95%
3	ED11A	С	BOP SRO	Loss of Instrument Bus	111
4	Cv01a	С	RO SRO	Trip of 1A CV Pump	
5	RX01E0	I	BOP SRO	Failure of 1PT-525 low 1	1B S/G pressure transmitter
6	Rx15 2500	I	RO SRO	Failure of Pzr Master Prohigh	essure Controller 1PK-455A
7	MS07A 3 120	М	TEAM	1A Main Steam Line Bre	eak inside Cnmt.

Failure of Turbine to Auto Trip

Failure of 1B SI Pump to start

MSIV's A & D fail to close

TEAM

TEAM

BOP

Μ

Μ

С

TC03

SI01B

MS01A&D

100

8

9

10

⁽N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

TITLE: Respond to a Loss of Instrument Bus followed by a N	Main Steam Line Fault
TURNOVER INFORMATION:	
The unit is at full power, steady state, equilibrium xenon, MOL	
THIS SCENARIO CONTAINS: The following objectives/K/A	\a's:
004000A4.08 (3.8/3.4) IV.D.OA-06/VIII.D.OA-056-C 000040AA2.04 (4.5/4.7) IV.D.EP-06/VIII.D.EP-001-E /012-B/ VIII.D.EP-007-B	Drain PRT per BOP RY-4 Ramp Down from 100% to 95% Response to Instrument Bus 111 Failure Respond to 1A CV Pump Trip Respond to a Failed S/G Press Channel Fails Low Respond to Master Pzr Press Controller Failing high Respond to 'A' Main Steam Line Break inside Cnmt Respond to Failure of Turbine to Trip Respond to Failure of 'A'
COMPLETION CRITERIA:	Terminate SI injection flow
CRITICAL TASKS: 1. Manually Trip Turbine during immediate action steps of 1 2. Manually aligning 1A SI Pump prior to transitioning out of 3. Terminate SI to prevent Pzr overfill and eliminate PTS configuration.	of 1BEP-0.
Examinees: US RO	

BOP _____

SCENARIO NUMBER: #1___

Evaluator:	Date:	

10 CFR 55.45

X 1. Perform pre-startup procedures for the facility, including operation those controls associated with plant equipment that could affect reactivity. <u>X</u> 2. Manipulate the console controls as required to operate the facility between shutdown and designated power levels. X 3. Identify annunciators and condition-indicating signals and perform appropriate remedial action where appropriate. <u>X</u> 4. Identify the instrumentation systems and the significance of facility instrument readings. Observe and safely control the operating behavior characteristics of the facility. <u>X</u> 6. Perform control manipulations required to obtain desired operating results during normal, abnormal, and emergency situations. <u>X</u> 7. Safely operate the facility's heat removal systems, including primary coolant, emergency coolant, and decay heat removal systems and identify the relation of the proper operation of these systems to the operation of the facility. <u>X</u> 8. Safely operate the facility's auxiliary and emergency systems, including operation of those controls associated with plant equipment that could affect reactivity or the release of radioactive materials to the environment. <u>X</u> 9. Demonstrate or describe the use and function of the facility's radiation monitoring systems, including fixed radiation monitors and alarms, portable survey instruments, and personnel monitoring equipment. Demonstrate knowledge of significant radiation hazards, including permissible levels in excess of those authorized, and ability to perform other procedures to reduce excessive levels of radiation and to guard against personnel exposure. Demonstrate knowledge of the emergency plan for the facility, including, as appropriate, the operator's or senior operator's responsibility to decide whether the plan should be executed and the duties under the plan assigned. Demonstrate the knowledge and ability as appropriate to the assigned position to assume the X 12. responsibility associated with the safe operation of the facility. Demonstrate the applicant's ability to function within the control room team as appropriate to the assigned position, in such a way that the facility licensee's procedures are adhered to and that the limitations in its license and amendments are not violated.

Simulator Scenario Review Checklist

	1.	The scenario has clearly stated	Yes	Operato	ors have sufficient time to	Yes
		objectives in the scenario summary.			at expected activities without	
	2.	The initial conditions are realistic, in	Yes	undue ti	ime constraints. Cues are given.	
		that some equipment and/or		If time of	compression techniques are used,	
		instrumentation may be out of service, but it		scenario	summary clearly indicates.	
		does not cue crew into expected events.		The sim	ulator modeling is not altered.	Yes
	3.	The scenario consists mostly of	Yes	10.	All crew competencies can be	Yes
		related events.			evaluated.	
	4.	Each event description consists of:	Yes	11.	The scenario has been validated.	Yes
		• the point in the scenario when it		12.	The sampling plan indicates that	Yes
		is to be initiated			the scenario was not used for training	
		• the malfunction(s) that are entered			during the requal cycle. Evaluate the need	
		to initiate the event			to modify/replace scenario if used.	
		 the symptoms/cues that will be visible 		13.	Total malfunctions inserted: 4-8 <u>8</u>	
		to the crew		14.	Malfunctions that occur after	
		 the expected operator actions 			EOP entry: 1-4 <u>3</u>	
		(by shift position)		15.	Abnormal Events: 1-2 2	
		 the event termination point 		16.	Major Transients: 1-21	
	5.	No more than one non-mechanistic	Yes	17.	EOPs used beyond primary	
		failure (e.g., pipe break) is incorporated			scram response EOP: 1-3 4	
		into the scenario without a credible preceding		18.	EOP Contingency Procedures	
		incident such as a seismic event.			used: 0-3 <u>1</u>	
6	5.	The events are valid with regard to	Yes	19.	Approximate scenario run time:	No
		physics and thermodynamics.			45-60 minutes	
7	7.	Sequencing/timing of events is	Yes	20.	EOP run time:	Yes
		reasonable, and allows for the			40-70% of scenario run time	
		examination team to obtain complete		21.	Crew Critical Tasks: 2-53	
		evaluation results commensurate		22.	Technical Specifications are	Yes
		with the scenario objectives.			exercised during the test.	

SCENARIO OVERVIEW

The unit is at full power, steady state, equilibrium xenon, MOL.

The scenario begins with the PRT High Pressure alarm LIT and the operating crew draining the PRT per BOP RY-4. This action will account for the normal evolution. The reactivity requirement will be fulfilled with a unit ramp from 100% Rx power down to 95% Rx power in preparation for a Main Feedwater Pump swap from 1C to 1A MFP for maintenance work.

Once the power change has been completed Instrument Bus 111 will de-energize and 1BOA ELEC-2, Loss of an Instrument Bus will be entered. The instrument bus is damaged & will remain de-energized. Depending on the cause of the failure enter the applicable Tech Spec 3.8.7 & 3.8.9 and 1BOA Inst-1 for a loss of N41.

After the actions of 1BOA ELEC-2 are complete, the 1A CV pump trips. The crew should start the 1B CV pump per BAR 1-9-A3 and take actions to stabilize the plant. Technical Specification 3.5.2 and TRM 3.1.d apply.

S/G Pressure Channel 1PT-525 fails low. The crew enters BOA INST-2, the BOP needs to take manual control of the 1B Feed Reg Valve and manual control of the Feedwater ΔP controller to restore adequate feedwater flow to all S/G's. The applicable Tech Spec for this failure is 3.3.2.

Master Pressurizer Pressure Controller 1PK-455A output fails high. PORV 1RY455A does not open due to the previous Instrument Bus failure. However, Pressurizer Spray valves fully open and Pzr pressure begins to drop. The RO takes manual control of the Master Pressurizer Pressure controller and restores Pzr Pressure. The applicable Tech Specs are 3.3.1, 3.3.2, & 3.3.4.

The 1A Main Steam Line experiences a large break. Containment temperature and pressure increase and Main Steam Line Pressure decreases to the SI setpoint. 1BEP-0, Response to Reactor Trip of Safety Injection is entered. The Reactor automatically trips on the SI signal but the Turbine fails to trip automatically following the Reactor trip. A Manual Turbine trip is required to be actuated. 1B SI pump fails to start on the Safety Injection, and 1A SI pump must be manually started. Due to the Instrument Bus failure all Train A ESF loads fail to start on the SI signal. Train A ESF loads must be manually started.

The A & D MSIV's fail to automatically or manually close causing both S/G's to blow down. Upon completion of 1BEP-0 the crew transitions to 1BEP-2, Faulted S/G to

isolate the faulted S/G's then transitions to 1BEP-1 Loss of Reactor or Secondary Coolant, and then to 1BEP ES-1.1, SI Termination, completion criteria is stopping the RH pumps in 1BEP ES-1.1.

SCENARIO SETUP GUIDE

- Initialize IC-22: "full power, steady state, equilibrium xenon, MOL."
- Lineup control boards.
- Place simulator in run (allow simulator to run during board walkdowns and turnover).
- Open PW to the PRT and fill until the PRT High Pressure alarm is LIT, leave BAR open to page on desk
- Check computer points that are placed in test and removed from scan as part of this scenario

N0049A removed from scan N0041, N0042, U1144 placed in test

- Type **bat f:\BY-SIM1\BY-SIM1** and ensure the following insert:

ior zdi1hstg010 norm imf tc03 imf ms01a 100 imf ms01d 100 imf si01b cae f:\BY-SIM1\BY-SIM1.CAE

 Perform TQ-BY-201-0113, Appendix A, "Simulator Exam/ Scenario Reset Checklist"

EVEN	ΓTIME	MALF NO.	DESCRIPTION
1	0	N/A	Drain PRT per BOP RY-4
2	10	N/A	Ramp Unit from 100% to 95%
3	25	ed11a	Trip Instrument Bus 111
4	40	cv01a	Trip of 1A CV Pump (SDG CV5)
5	50	rx01e 0	Failure of 1PT-525 low 1B S/G pressure transmitter
6	65	rx15 2500	Failure of PZR Master Press controller 1PK-455A High

7	75	ms07a 3 120	1A Main Steam Line Break inside Cnmt.
8	75	tc03 (in batch)	Failure of Turbine to Auto Trip
9	75	ms01a & d 100 (in b	eatch) MSIV's A & D fail to close
10	75	si01b	Failure of 1B SI pump to start

Note 1: Events 7, 8 & 9 should be tied to Main Steam Line Break.

INSTRUCTOR/SIMULATOR RUN AID GUIDE

<u>EVENT</u>	ADDITIONAL INFORMATION
3	Report Bus 111 Inverter AC Output Bkr 4CB Open. Report Bus 111 Main Feed Bkr Open. Report Bus 111 is damaged, with smell of burned insulation and paint bubbled. Enters 1BOA ELEC-2, Loss of AC bus, and INST-1, NI malfunction. Need to trip
	bistables associated with N41. Operator needs to be dispatched to open 1AF005A-D. Allow ten minutes to accomplish this task.

Remote functions per event:

RP20	Open/Close	Protection Cabinet Door #	‡ 1	
	(SDC	6 RX10)		
RX013	TRIP	OTΔT Rx Trip	TB411C	C1-124
	BS-3 (SDC	6 RX4)		
RX135	TRIP	OT∆T Runback	TB411D	C1-124
	BS-4 (SDC	6 RX4)		

Acknowledge 1A CV pump trip/EP review request when contacted as SM (as required). When requested to investigate, report a Phase A overcurrent flag at the 1A CV pump breaker bus 141 cub 11. Report results 4 minutes after request.

Per the initial scenario cue. Annunciator PRT Press High alarm LIT (Response) Refer to BOP RY-4 Draining the Pressurizer Relief Tank CLOSE 1RY469 PRT to GW isol vlv VERIFY approximately 3 psig on 1PI469 (6# indicated) VERIFY/OPEN 1AOV-RY8033 N ₂ Supply to PRT isol vlv VERIFY/OPEN 1AOV-RE9170, RCDT Discharge header vlv U-1 VERIFY/OPEN 1AOV-RE1003, RCDT Discharge header vlv U-1 VERIFY/OPEN 1AOV-RE1003, RCDT Discharge header vlv U-1 VERIFY/START 1RE01PA/B, RCDT Pump 1A/B VERIFY/START 1RE01PA/B, RCDT Pump 1A/B VERIFY/START 1RE01PA/B, RCDT Pump 1A/B	EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT	
(Response) Refer to BOP RY-4 Draining the Pressurizer Relief Tank CLOSE 1RY469 PRT to GW isol vlv VERIFY approximately 3 psig on 1PI469 (6# indicated) VERIFY/OPEN 1AOV-RY8033 N ₂ Supply to PRT isol vlv VERIFY/OPEN 1AOV-RE9170, RCDT Discharge header vlv VERIFY/OPEN 1AOV-RE1003, RCDT Discharge header vlv VERIFY/OPEN 1AOV-RE1003, RCDT Discharge header vlv VERIFY/START 1RE01PA/B, RCDT Pump 1A/B CYCLE 1RY8031, PRT Drain vlv to control PRT pressure CLOSE 1AOV-RY8031, PRT Drain vlv at desired PRT level VERIFY/STOP 1RE01PA/B, RCDT Pump 1A/B U-1 VERIFY/STOP 1RE01PA/B, RCDT Pump 1A/B U-1 VERIFY/STOP 1RE01PA/B, RCDT Pump 1A/B U-1	Perform the action to lower PRT level per BOP RY-4			
 VERIFY/OPEN 1AOV-RE1003, RCDT Discharge header vlv U-1	Per the initial scenario cue. Annunciator PRT Press High alarm LIT (Response) Refer to BOP RY-4 Draining the Pressurizer Relief Tank CLOSE 1RY469 PRT to GW isol vlv VERIFY approximately 3 psig on 1PI469 (6# indicated) VERIFY/OPEN 1AOV-RY8033 N ₂ Supply to PRT isol vlv	U-1 U-1 U-1	<u> </u>	
	 VERIFY/OPEN 1AOV-RE1003, RCDT Discharge header vlv OPEN 1AOV-RY8031, PRT Drain vlv VERIFY/START 1RE01PA/B, RCDT Pump 1A/B CYCLE 1RY8031, PRT Drain vlv to control PRT pressure CLOSE 1AOV-RY8031, PRT Drain vlv at desired PRT level VERIFY/STOP 1RE01PA/B, RCDT Pump 1A/B 	U-1 U-1 U-1 U-1 U-1		
	COMMENTS _			

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT	
Re-activity manipulation for this scenario.			
(Cue) o Per the initial scenario cue.			
(Response) • Refer to BGP 100-4T3, Load Swing Instruction Sheet	CREW		
• Direct actions to lower Rx Pwr	US		
 PROGRAM Turbine Controls to ramp down Enter ramp rate (2 MW/min) into LOAD RATE Enter ~1175 into REF DEMAND 	U-1 AST		
 BORATE per BOP CV-6 to control Tave-Tref within desired limits Enter boration amount into AB totalizer Select BORATE Mode Select START on Makeup control switch 	U-1		
• ENERGIZE Second Pressurizer B/U Htr group.	U-1		
• INSERT Control Rods to maintain ΔI and Tave-Tref	U-1		
• STOP ramp at 95% Rx Pwr	US		

EVENT 3	
BRIEF DESCRIPTION Loss of Instrument Bus 111	
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT
Recognize symptoms and respond to a de-energized Instrume	nt Bus 111
 (Cue) Loss of Control and Instrument Power to: N31 Source Range Instrument N35 Intermediate Range Instrument N41 Power Range Instrument Annunciator (1-4-A5) "BUS 111 INVERTER TROUBLI Annunciator (1-4-A3) "PROCESS I & C CAB PWR SUI Annunciator (1-4-B3) "SOLID STATE PROT CAB GEN Annunciator (1-4-C2) "SEQUENCING CAB PWR FAIL Annunciator (1-13-A2) "RCP BUS UNDERVOLTAGE RX TRIP ALERT" and associated trip status light 	P FAILURE" NERAL WARNING"
 (Response) Implement 1BOA Elec-2, "Loss of AC Bus" Instrument control channels checked for operability: PZR pressure/level T_{AVE}/Delta T P_{IMP} 	US
 SG level, steam flow & feed flow Dispatch operator to investigate status of bus Do NOT attempt to energize Instrument Bus 111 from CVT (bus is damaged) 	CREW
 Dispatch operator to locally fail open 1AF005A-D Brief crew on effects of loss of inst. bus 111/Train A ESI equipment manual start requirements per Table A 	U-1 AST
Inform SM of unit status/loss of inst. bus 111/EP potential/shutdown required by Tech Specs	US
Implement 1BOA INST-1 "NUCLEAR INSTRUMENT MALFUNCTION" to establish the following conditions:	US
 Control rods placed in manual Plant conditions stabilized Rod stop bypassed Tave restored to Tref (+ 1 degree) 	U-1

SG levels stable

•	Bypass associated functions for PR N-41	U-1 AST		
	 Upper current comparator 			
	 Lower current comparator 			
	 Power mismatch 			
	 Rod stop 			
	Channel current comparator			
•	Trip bistables for PR N-41			
	• Pull control power fuses to trip:	U-1 AST		
	• Lo Rx trip			
	• Hi Rx trip			
	• Positive Rate trip			
	• Dispatch operator to locally trip:	CREW	<u></u>	
	• OTAT Trip			
	• OTAT Runback			
•	Select/verify operable channel to loop ΔT recorder	U-1		
•	Remove Point from Scan Input to PDMS	U-1	· · · · · · · · · · · · · · · · · · ·	·
•	Restore automatic rod control	U-1 U-1 US US	·	
•	Inform SM of unit status/potential EP event	US		
•	Review Tech Spec 3.8.7, Inverters - Operating & 3.8.9,	US	·	
	Dist. Systems - Operating (Restore power to inst. bus	· · · · · · · · · · · · · · · · · · ·	·	
	111 within 2 hours or be in hot standby in next 6 hours.)			
	•			
CO	MMENTS			

EVENT 4	
BRIEF DESCRIPTION 1A CV Pump Trip	
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT
Recognize symptoms and respond to a tripped CV pu (Cue) o Annunciator (1-7-B2) "RCP SEAL WTR INJ Fl	-
o Annunciator (1-9-A3) "CHG PUMP TRIP"o Annunciator (1-9-D3) "CHG LINE FLOW HIG	
o 1A CV pump trip light lit (Response)	
 Direct actions to restore charging flow Refer to BAR 1-9-A3 	US U-1 AST/U-1
 Isolate Letdown Ensure suction source to standby charging pump 	U-1 AST/U-1
Place 1CV121 in Manual at 10% OpenStart the 1B CV pump	U-1
EXAMINER'S NOTE: The crew may elect to perform actions for loss of sea "LOSS OF SEAL COOLING".	el injection per 1BOA RCP-2
Implement 1BOA RCP-2 "LOSS OF SEAL COOLING direct operator actions to:	G" to US
 Check RCP seal cooling Start 1B CV pump 	U-1 U-1 iection flow U-1
• Throttle 1CV182 and 1CV121 to control seal inj	jection flow U-1
EXAMINER'S NOTE: Bus 111 failure prevents auto make-up to VCT. Low action to Open make-up isolation valves per applicable.	•
o Restore Letdown per BOP CV-17Notify SM of unit status/EP potential	U-1 AST/U-1 US
 Refer to TRM 3.1.d, Charging Pumps - Operating, and Tech Spec 3.5.2, ECCS - Operat (return to operable status within 7 days). 	US
 Direct performance of 1BOSR 5.5.1-1 (Reference Dispatch operators to locally check 1A CV pum Notify EM Dept. to investigate 1A CV pump tri 	p/breaker CREW

- (Feedback)
 o 1B CV pump running
 o Charging flow restored

o Seal injection flows normal

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT	
Recognize symptoms and respond to 1B SG Pressure Transm	nitter failed LOW	
(Cue) Annunciator LOW MS PRESSURE Annunciator 1B SG SF – FF MISMATCH 1B SG Feed Flow lowering 1B SG Level lowering Master Turbine Driven Feed Pump (TDFPSC) DP deman	nd lowering	
Check SG pressure Take manual control to restore SG Feed Flow Place TDFPSC controller in manual Control Feedwater DP Implement 1BOA Inst-2, Failure of Instrument Channel, Transfer controlling SP channels Locally trip Bistables for failed channel Review Tech Spec 3.3.2 Contact SM/maintenance to investigate 1PT-525 failure Feedback) Appropriate annunciators clear Associated trip status lights and annunciators lit SG level restored to normal	U-1AST	
COMMENTS		

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT
Recognize symptoms and respond to a Master Pzr Press Controller fa	àilure
Cue)	
Annunciator (1-12-D2) "PZR PRESS CONT DEV HIGH"	
Annunciator (1-12-B2) "PZR PORV OR SAF VLV OPEN" Wo	on't Open due to Bus 111
failure	
PZR spray valves (1RY455B & C) open	40007
Master PZR pressure controller output indication (1PK-455A) a	at 100%
Actual PZR pressure dropping	
Response)	
Check PZR pressure	U-1
 Take manual control to restore PZR pressure 	
 Place Master PZR pressure controller in manual 	U-1
Control PZR pressure	
Check PZR PORVs, spray valves and heaters	U-1
• PZR PORVs – closed	U-1
 PZR spray valves – normal PZR heaters – normal 	U-1
(Pzr Htr's won't auto turn on due to Bus 111 failure	O-1
Review Tech Spec 3.4.1,	US
DNB Limits.	<u></u>
Contact SM/maintenance to investigate 1PK-455A failure	US
(Feedback)	
Appropriate annunciators clear	
Associated trip status lights and annunciators lit	
PZR pressure restored to normal	

EVENT7	
Brief Description: 1A Main Steam line break inside of	<u>Cnmt.</u>
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT
Recognize symptoms and respond to main steam line b	reak inside of cnmt.
 (Cue) o Cnmt temp and pressure increasing o Low S/G & Main Steam Line Pressure o Rx Trip and Safety Injection o Multiple Annunciator Alarms 	
 (Response) Automatic Rx Trip Implement 1BEP-0, Reactor Trip or Safety Injection Inform SM of unit status/potential EP event 	CREW
 Reactor trip verified Rod bottom lights Reactor trip/Bypass breakers Neutron flux dropping 	U-1
 * Turbine MANUALLY TRIPPED by fast action GV CLOSED from OWS panel G-5512 • GVs closed • TVs closed 	U-1 AST**
 4 KV ESF busses are energized Bus 141 energized Bus 142 energized 	U-1 AST
 SI Status SI is actuated/required SI ACTUATED permissive light lit SI first out annunciator lit SI equipment automatically actuated 	CREW
Actuate SI	U-1/U-1 AST
• Feedwater Status	CREW
COMMENTS	

(NOTE)

Loss of Instrument Bus 111 causes failure of Train A ESF load to automatically start on SI signal. Train A SI Pump must be manually aligned to complete critical task. RCP Trip criteria may also be met early during scenario.

•*	Verify ECCS Status	CREW		
	 Verify 1B CV Pp Running 	U-1 AST/U-1		
	•* START 1A SI Pp	U-1 AST/U-1 *	*	*
	• START 1A RH Pp	U-1 AST/U-1		
•	Verify RCFC's Running in Accident Mode			
	 Manually align A Train components 	U-1 AST/U-1		
•	Verify Cnmt Isolation Phase A			
	 Manually align A Train components 	U-1 AST/U-1		
•	Verify Cnmt Ventilation Isolation	U-1 AST/U-1		
•	Verify Aux. Feed System			
	 Manually align A Train components 	U-1 AST/U-1		
•	Verify CC Pp's Running			
	• Manually align A Train component Start 1A CC P	p U-1 AST/U-1		
•	Verify SX Pp's Running			
	• Manually align A Train component Start 1A SX P	p U-1 AST/U-1		
•	Main Steam Line Isolation	TI 4 A CODATI 4		
	 Manually attempt to close 1A & 1D MSIV's 	U-1 AS1/U-1		-
•	Check if Cnmt Spray is Required			
	 Manually align A Train if required 	U-1 AST/U-1		
•	Verify Total Aux. Feed System Flow			
	 Manually align A Train components 	U-1 AST/U-1		
•	Verify ECCS Valve alignment			
	• Manually align A Train components	U-1 AST/U-1		
•	Verify ECCS Flow	U-1 AST/U-1		
•	Verify one Pzr PORV relief path available	U-1		
•	Verify Main Generator Tripped	U-1 AST/U-1		

Verify D/G's RunningManually Start 1A D/G	U-1 AST/U-1
 Verify Ventilation Equipment aligned for Emerger Manually align A Train components 	u-1 AST/U-1
Verify Pzr Spray Valves Closed	U-1 AST/U-1
EXAMINER'S NOTE: Due to steam line break RCS temperature control will not be dumping steam and close all MSIV's. They should also stop at step 25 to Maintain RCS Temperature.	
Maintain RCS Temperature Control	U-1 AST/U-1
Check RCP Status	U-1 AST/U-1
 Check S/G Secondary Pressure boundaries intact Crew determines Secondary Pressure Bound and transitions to 1BEP-2, Faulted S/G Isola 	aries not intact
EXAMINER'S NOTE: US directs transition from 1BEP-0 to 1BEP-2, Faulted S/G I procedure is to isolate the Non-Faulted S/G's from the Fault closing the MSIV's on the Non-Faulted 1B & 1C S/G's.	
Announcement of faulted Steam Generator symptoms Transition to 1BEP-2 "FAULTED SG ISOLATION"	US
 Order STA function at transition from 1BEP-0 "RX TRIP OR SI" 	US
 Perform operator actions to establish the following condense of the second se	U-1 AST

•	Monitor AF pump suction pressure	U-1 AST
•	Check Secondary Radiation	
	 Identify NO ruptured SGs 	U-1 AST

EXPECTED OPERATOR/PLANT RESPONSE	SAT UNSAT	
N/A		
 Transition to 1BEP-1, Loss of Rx or Secondary Co 	olant IIS	
Transition to TBET-1, Loss of KX of Secondary Co	olant US	
Transition from 1BEP-2 to 1BEP-1, "LOSS OF	US	
REACTOR OR SECONDARY COOLANT" upon veri		
that a SGTR does not exist per 1BEP-2		
Perform operator actions of 1BEP-1 to establish the foll	owing	
conditions:	owing	
Check status of RCPs	II-1	
 Check status of RCPs Check SG pressure boundaries intact (1A & D SG Check intact SG levels 	faulted)U-1 AST	
• Check intact SG levels	U-1 AST	
• SG levels $> 10\%$ (31%)		
• Maintain SG NR levels between 10% (31%) -	50%	
SG levels not rising in an uncontrolled manner		
 Check secondary radiation trends normal 	U-1 AST	
 Check SJAE/gland steam exhaust and 		
MS line radiation normal for plant conditions		
 Check PZR PORVs and isolation valves 	U-1	
Transition from 1DED 1 to EC 1 1 UCL TEDMINIATION	US	
Transition from 1BEP-1 to ES-1.1 "SI TERMINATION when the following conditions are met:	US	
 Acceptable RCS subcooling 		
 Secondary heat sink 		
• SG NR levels > 10% (31%)		
• > 500 gpm total flow		
RCS pressure is stable/rising		
• PZR level > 12%		
Direct operator actions to	US	
establish the following conditions of ES-1.1:		
• Reset SI	U-1/U-1 AST	
Reset Containment Isolation	U-1 AST	
• Phase A		
o Phase B		
• Check SAC run light lit		
IA restored to Cnmt CV system realigned.	TT 1	
CV system realigned 1B CV pump already running	U-1	
1B CV pump already runningCheck RCS pressure stable or rising	U-1	
Terminate high-head ECCS	U-1 *	*
CV pump suction aligned to RWST	U-1	·
 SI recirc sump/CV miniflows reset 		
of rooms sumple villaminows reset		

- CV miniflow valves open 1SI8801A & B closed

 Establish charging flow 1CV182 controller at 0% 1CV8105 and 1CV8106 opened 1CV182 throttled (8-13 gpm seal injection) 	U-1	
Control charging to maintain PZR levelSI pumps stopped	U-1 II-1/II-1 AST	
 RH pumps stopped 	U-1/U-1 AST U-1/U-1 AST	
1 1 11		
TERMINATE SCENARIO		
COMMENTS		
COMMENTS		

Appenaix	Scenario Outline Form ES-D-1					
Facility: <u>Byron</u> Scenario No: #2				2	Op-Test No:	2008-01
Examiners:				Operators:		
Initial C	onditions: Rx Pc	wer	~ 100%.			
Turnove	er: Ramp down t	:o 95°	% Rx Pov	ver.		
Event No.	Malf. No.		Event Type*			Event Description
1	msv1ms004a =100	С	BOP SRO	1MS004A Fails Open. Rx Power increases > 100%		
1a		R	RO SRO	Ramp from 102% down to 98%		
2	Cv16 100	I	RO SRO	1LT-112 Failure High		
3	Rx18f 650	I	BOP SRO	1TE-445 Failure High		
4	rm061	С	BOP SRO	Failure of 0PR031J to actuate VC Emergency Make-L		
5	Eg03 96	С	BOP SRO	Failure of Main Generator Voltage Regulator High		
6	Th04a 50000	М	TEAM	Sesmic event results in Large Break LOCA		

1B CS Train fails to align, Manual action required

1B RH Pp Trips on OC at 49% RWST level

1SI8811A Fails to open on low RWST level

TEAM

TEAM

TEAM

С

С

С

Cs05

Ed053p open

Rh01b

7

8

9

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

TITLE:		
TURNOVER IN	NFORMATION:	
The unit is at 10	00% power, steady state, ed	quilibrium xenon, MOL.
THIS SCENAR	IO CONTAINS: The follo	owing objectives/K/A's
		1MS004A Steam Dump Valve Leak Ramp unit <100% power 1LT-112 Failed High 1TE-445 Failed High High Rad Alarm on 0PR31J with no Emergency Mode Actuation Signal – Manual alignment required Main Generator Voltage Regulator fails high Large Break LOCA Failure of 1B CS Train to properly align Failure of 1SI8811A Cnmt Recirc Sump Isolation Vlv to Open
		Failure of 1B RH Pp
COMPLETION 200°F	I CRITERIA:	Initiate RCS Cooldown to
200°F CRITICAL TAS		Initiate RCS Cooldown to
200°F CRITICAL TAS 1. Manually a	SKS:	Initiate RCS Cooldown to

BOP _____

	_		
Evaluator:	 Date:		

10 CFR 55.45

X 1. Perform pre-startup procedures for the facility, including operation those controls associated with plant equipment that could affect reactivity. <u>X</u> 2. Manipulate the console controls as required to operate the facility between shutdown and designated power levels. X 3. Identify annunciators and condition-indicating signals and perform appropriate remedial action where appropriate. <u>X</u> 4. Identify the instrumentation systems and the significance of facility instrument readings. Observe and safely control the operating behavior characteristics of the facility. <u>X</u> 6. Perform control manipulations required to obtain desired operating results during normal, abnormal, and emergency situations. <u>X</u> 7. Safely operate the facility's heat removal systems, including primary coolant, emergency coolant, and decay heat removal systems and identify the relation of the proper operation of these systems to the operation of the facility. <u>X</u> 8. Safely operate the facility's auxiliary and emergency systems, including operation of those controls associated with plant equipment that could affect reactivity or the release of radioactive materials to the environment. <u>X</u> 9. Demonstrate or describe the use and function of the facility's radiation monitoring systems, including fixed radiation monitors and alarms, portable survey instruments, and personnel monitoring equipment. Demonstrate knowledge of significant radiation hazards, including permissible levels in excess of those authorized, and ability to perform other procedures to reduce excessive levels of radiation and to guard against personnel exposure. Demonstrate knowledge of the emergency plan for the facility, including, as appropriate, the operator's or senior operator's responsibility to decide whether the plan should be executed and the duties under the plan assigned. Demonstrate the knowledge and ability as appropriate to the assigned position to assume the X 12. responsibility associated with the safe operation of the facility. Demonstrate the applicant's ability to function within the control room team as appropriate to the assigned position, in such a way that the facility licensee's procedures are adhered to and that the limitations in its license and amendments are not violated.

Simulator Scenario Review Checklist

1	The scenario has clearly stated objectives in the scenario summary.	Yes	Operators have sufficient time to carry out expected activities without		Yes
2	2. The initial conditions are realistic, in	Yes	undue time constraints. Cues are given.		
	that some equipment and/or			compression techniques are used,	
	instrumentation may be out of service, but it			o summary clearly indicates.	
	does not cue crew into expected events.			nulator modeling is not altered.	Yes
3	3. The scenario consists mostly of	Yes	10.	All crew competencies can be	Yes
	related events.			evaluated.	
۷	Each event description consists of:	Yes	11.	The scenario has been validated.	Yes
	 the point in the scenario when it 		12.	The sampling plan indicates that	Yes
	is to be initiated			the scenario was not used for training	
	 the malfunction(s) that are entered 			during the requal cycle. Evaluate the need	
	to initiate the event			to modify/replace scenario if used.	
	 the symptoms/cues that will be visible 		13.	Total malfunctions inserted: 4-88	
	to the crew		14.	Malfunctions that occur after	
	 the expected operator actions 			EOP entry: 1-4 <u>3</u>	
	(by shift position)		15.	Abnormal Events: 1-23	
	 the event termination point 		16.	Major Transients: 1-21	
4	No more than one non-mechanistic	Yes	17.	EOPs used beyond primary	
	failure (e.g., pipe break) is incorporated			scram response EOP: 1-3 3-4	
	into the scenario without a credible preceding		18.	EOP Contingency Procedures	
	incident such as a seismic event.			used: 0-3 <u>1</u>	
6.	The events are valid with regard to	Yes	19.	Approximate scenario run time:	No
	physics and thermodynamics.			45-60 minutes	
7.	Sequencing/timing of events is	Yes	20.	EOP run time:	Yes
	reasonable, and allows for the			40-70% of scenario run time	
	examination team to obtain complete		21.	Crew Critical Tasks: 2-53	
	evaluation results commensurate		22.	Technical Specifications are	Yes
	with the scenario objectives.			exercised during the test.	
	-				

SCENARIO OVERVIEW

The unit is at 100% Rx power, steady state, equilibrium xenon, MOL.

The scenario begins when Steam Dump valve 1MS004A fully opens. This pushes Rx Pwr above 100% and requires the unit to be ramped down and the valve isolated.

1LT-112 fails high, bringing in alarm 1-9-A2, VCT LEVEL HIGH-HIGH/LOW, resulting in diverting letdown to the HUT. The crew will respond to the BAR and take manual control of VCT level controller. Manual makeup to the VCT will be required since auto makeup will not work.

Loop B T_{hot} Channel will fail High. This failure may require the crew to place the Control Rods into Manual to stop outward rod motion. The crew enters 1BOA INST-2, stablizes the plant and trips the associated bistable. The applicable Tech Spec for this failure is 3.3.1.

Following initiation of bistable tripping for the T_{hot} Channel failure 0PR31J will go into High Rad Alarm without actuating the Emergency Make-Up Mode of Operation for the 0A VC Train. The crew will need to recognize the appropriate system response did not occur and will have to manually align the 0A VC Train into the Emergency Make-Up Mode.

The next failure to occur will be for the Main Generator Voltage Regulator output to fail high. This will result in increased excitation of the Main Generator which results in increased MVAR output and thus increased current flowing through the generator. The crew should recognize the failure respond to the applicable BAR procedures switch the voltage regulator to OFF and lower generator output to within limits.

The major failure for this scenario will be a seismic event that causes a Large Break LOCA. During 1BEP-0 it will be discovered that the 1B CS Train doesn't align properly. The crew must take manual actions to correctly align the 1B CS Train into operation.

When RWST water level reaches 49% the 1B RH Pp will trip due to a ϕ C overcurrent condition. When RWST water level reaches 46% the 1SI8811A Cnmt Sump Isolation Valve will not open. This results in a Loss of Emergency Recirculation capability and thus 1BCA-1.1 needs to be entered from 1BEP ES-1.3.

SCENARIO SETUP GUIDE

- Initialize IC-22: "full power, steady state, equilibrium xenon, MOL."
- Lineup control boards.
- Place simulator in run (allow simulator to run during board walkdowns and turnover).
- Type **bat f:\BY-SIM2\BY-SIM2** and ensure the following insert:

imf ms05a 100 imf rm05e imf cs01a mrf cs05 over trgset 1 "zao1li930.lt.0.49" trg 1 "imf rh01b" trgset 3 "zlo1si8811a(2).gt.0" trg 3 "mrf ed053p open" cae f:\BY-SIM2\BY-SIM2.CAE

 Perform TQ-BY-201-0113, Appendix A, "Simulator Exam/ Scenario Reset Checklist"

EVENT	TIME	MALF NO.	DESCRIPTION
1	0	set msv1ms004a=100	1MS004A Fails Open Ramp Unit to < 100% Rx Pwr
2	5	cv16 100	1LT-112 failure High
3	15	rx18f 650	Failure of 1TE-445 High
4	25	rm061	Failure of 0PR31J to actuate Emergency Make-Up Mode
5	40	eg03 96	Failure of Main Generator Voltage Regulator High
6	50	th04a 50000	Seismic event results in Large Break LOCA
7	70	cs05	1B CS Train fails to align, Manual action required

8	80	rh01b	1B RH Pp trips on φC overcurrent at 49% RWST level
9	85	ed053p open	1SI8811A fails to open on low RWST level 46.7%

Note 1: Events 6 &7 can be tied to the seismic event and events 8 & 9 should be tied to RWST Level.

EVENT1			
BRIEF DESCRIPTION 1MS004A Fails Open			
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT U	INSAT	
Perform the action to determine cause of over power condit reduce Rx Pwr to < 100%. The crew should also determine condition and take actions to isolate the failed open Steam I	cause of the overpow		
Reactivity manipulation for this scenario.			
 (Cue) Annunicator Bank D Rod Stop C-11 Noticeable RCS cooldown Rx Power > 100% Crew notices 1MS004A Open sugar cube light lit 			
(Response)			
• Refer to 1BGP 100-4, Power Descension	Crew		
• Direct actions to lower Rx Pwr	US		
 PROGRAM Turbine Controls to ramp down 	U-1 AST		
 BORATE per BOP CV-6 to control Tave-Tref within desired limits 	U-1		
• INSERT Control Rods to maintain ΔI and Tave-Tref	U-1		
• STOP ramp when < 100% Rx Pwr	US		
Direct NLO to Locally isolate 1MS004A	U-1 AST		
COMMENTS			_

BRIEF DESCRIPTION <u>1KT-112 failure high</u>		
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT
(Cue) • 1-9-A2, VCT LEVEL HIGH-HIGH/LOW LIT • 1LI-112 at 100%		
(Response) • Refer to BAR	CREW	
Direct actions to manually control VCT level	US	
 Manually makeup to VCT as required. 	U-1	
 Notify SM or WEC for IR 	US	
COMMENTS		

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT
Recognize symptoms and respond to a Failed High Tave Ch	annel	
Cue)		
Annunciator Tave CONTROL DEV HIGH		
Annunciator Tave CHANNEL DEV HIGH		
Annunciator DELTA-T CHANNEL DEV HIGH		
Response)		
	-1 AST/U-1	
Direct actions to enter 1BOA INST-2		
PLACE Rod Bank Select Switch in Manual	U-1	
Manually Defeat Failed RTD Channel		
• SELECT failed Tave channel with Tave Defeat Switch		
• SELECT failed ΔT channel with ΔT defeat switch	U-1	
SELECT an operable RTD channel to the ΔT Recorder	U-1	
Check if Rod Control can be placed in Auto		
• Turbine Low Power Interlock C5 Not Lit	U-1	
 Tave-Tref deviation Stable and within 1°F 	U-1	
 Place Rod Bank Select Switch in Auto if desired 	U-1	
Check Pzr Level normal and Stable	U-1	
	-1 AST/U-1	
Check P12 Interlock in correct state for current RCS ter		
Defeat affected channel input to PDMS (T0422 to Test)		
Enter Tech Spec 3.3.1, 3.3.2 for the failed channel	US	

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT
Recognize symptoms and respond to a 0PR31J High Rad	Alarm	
(Cue) o RM-11-2-0PR31J Annunciator (0PB131) "Main Con	trol Room Out Air Ir	n OA"
 (Response) Direct actions to respond to the RM-11 alarm Enter BAR RM-11-2-0PR31J Determines Auto actions have not occurred STARTS Make-Up Fan 0VC03CA, BOP VC-5 Aligns recirculation charcoal adsorber Direct NLO to STOP 0VV01CA & B at 0VV01J Notify RP to perform BRP 5820-13 Enter Tech Spec 3.3.7, spec 3.7.10 not required 	US U-1 AST/U-1 U-1 AST U-1 AST U-1 AST U-1 AST US US	
EXAMINER'S NOTE: The crew may elect to perform actions to swap trains of N but this is not required or anticipated.	Iain Control Room H	IVAC
	rs US	
Implement 1BOP-VC-17 "Swapping Control Room Chille and HVAC Trains"	rs US	
Implement 1BOP-VC-17 "Swapping Control Room Chille and HVAC Trains" • Secure electric heaters	<i>U-1 AST</i>	
 Implement 1BOP-VC-17 "Swapping Control Room Chille and HVAC Trains" Secure electric heaters Secure chiller 	<i>U-1 AST U-1 AST</i>	
 Implement 1BOP-VC-17 "Swapping Control Room Chille and HVAC Trains" Secure electric heaters Secure chiller Secure Ventilation system 	U-1 AST U-1 AST U-1 AST	
 Implement 1BOP-VC-17 "Swapping Control Room Chille and HVAC Trains" Secure electric heaters Secure chiller Secure Ventilation system Start standby Ventilation system 	U-1 AST U-1 AST U-1 AST U-1 AST	
 Implement 1BOP-VC-17 "Swapping Control Room Chille and HVAC Trains" Secure electric heaters Secure chiller Secure Ventilation system 	U-1 AST U-1 AST U-1 AST	

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UN	SAT
Recognize symptoms and respond to the Main Generator Voltage high.	Regulator's output faili	ng
(Cue) Annunciator (1-19-B6) "Generator Field Forcing"		
Response) implement BAR 1-19-B8		
PLACE Voltage Regulator Control Switch to OFF	U-1 AST	
 ADJUST Base Adjuster reduce exciter field current to LESS THAN 100 amps 	U-1 AST	
 Notify Electric Operations 	U-1 AST	
• Check U-2 for adverse Main Generator trends	U-1 AST	
• If Generator field current cannot be reduced to LESS THAN 109 amps THEN trip the Rx if above P-8	U-1 AST	

BRIEF DESCRIPTION <u>Large Break LOCA with failur</u> properly align.		
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UN	SAT
Recognize symptoms and respond to a Large Break LOCA		
(Cue) o Annunciator "Rx Trip" o Annunciator "Safety Injection"		
(Response) Automatic Rx Trip Implement 1BEP-0, Reactor Trip or Safety Injection	CREW	
• Inform SM of unit status/potential EP event	US/STA	
 Reactor trip verified Rod bottom lights Reactor trip/Bypass breakers Neutron flux dropping 	U-1	
Turbine trip verified byGVs closedTVs closed	U-1 AST	
 4 KV ESF busses are energized Bus 141 energized Bus 142 energized 	U-1 AST	
 SI Status SI is actuated/required SI ACTUATED permissive light lit SI first out annunciator lit SI equipment automatically actuated 	CREW	
• Actuate SI	U-1/U-1 AST	
Feedwater StatusFWI Monitor Lights LIT	CREW	

•	Verify ECCS Status	CREW	_		
	 Verify CV Pp's Running 	U-1 AST/U-1			
	 Verify SI Pp's Running 	U-1 AST/U-1			
	 verify RH Pp's Running 	U-1 AST/U-1	_		
•	Verify RCFC's Running in Accident Mode	U-1 AST/U-1	_		
•	Verify Cnmt Isolation Phase A	U-1 AST/U-1	_		
•	Verify Cnmt Ventilation Isolation	U-1 AST/U-1	_		
•	Verify Aux. Feed System	U-1 AST/U-1	_		
•	Verify CC Pp's Running	U-1 AST/U-1	_		
•	Verify SX Pp's Running	U-1 AST/U-1	_		
•	Main Steam Line Isolation	U-1 AST/U-1	_		
	P trip criteria will be met on both RCS Pressure ad SI flow, and because of Phase B containment i		High ———		
				*	*
Не	ad SI flow, and because of Phase B containment i Check if Cnmt Spray is Required • Manually align B Train Test Switch to 'Test Position'	solation		*	*
Не	ad SI flow, and because of Phase B containment i Check if Cnmt Spray is Required • Manually align B Train Test Switch to 'Test Position' OPEN 1CS019B	solation		*	*
Не	Check if Cnmt Spray is Required • Manually align B Train Test Switch to 'Test Position' OPEN 1CS019B Test Switch to 'Normal' Position	Solation U-1 AST/U-1	*	*	*
Не	Check if Cnmt Spray is Required • Manually align B Train Test Switch to 'Test Position' OPEN 1CS019B Test Switch to 'Normal' Position Verify Total Aux. Feed System Flow	U-1 AST/U-1	*		*
Не	Check if Cnmt Spray is Required • Manually align B Train Test Switch to 'Test Position' OPEN 1CS019B Test Switch to 'Normal' Position Verify Total Aux. Feed System Flow Verify ECCS Valve alignment	U-1 AST/U-1 U-1 AST/U-1 U-1 AST/U-1	*		
Не	Check if Cnmt Spray is Required • Manually align B Train Test Switch to 'Test Position' OPEN 1CS019B Test Switch to 'Normal' Position Verify Total Aux. Feed System Flow Verify ECCS Valve alignment Verify ECCS Flow	U-1 AST/U-1 U-1 AST/U-1 U-1 AST/U-1 U-1 AST/U-1	*	— —	
Не	Check if Cnmt Spray is Required • Manually align B Train Test Switch to 'Test Position' OPEN 1CS019B Test Switch to 'Normal' Position Verify Total Aux. Feed System Flow Verify ECCS Valve alignment Verify ECCS Flow Verify one Pzr PORV relief path available	U-1 AST/U-1 U-1 AST/U-1 U-1 AST/U-1 U-1 AST/U-1 U-1 AST/U-1	*		

for Emergency

	r Spray Valves & PORV's Closed	U-1 AST/U-1	
• Maintain	RCS Temperature Control	U-1 AST/U-1	
• Check RC	P Status	U-1 AST/U-1	
• Check S/C	G Secondary Pressure boundaries Intact	U-1 AST/U-1	
• Check S/C	G Tubes are Intact	U-1 AST/U-1	
	S NOTE: ition from 1BEP-0 to 1BEP-1, Loss of Reacte to 46% the Crew will transition to 1BEP ES-		
	RCS is Intact I transition to 1BEP-1, Loss of Rx or Coolant)	U-1 AST/U-1	
Transition to 1	BEP-1 "Loss of Rx or Secondary Coolan	t" US	
• Call for S'	TA function at transition from 1BEP-0 OR SI"	US	
	ns of 1BEP-1 establish the following cond		
.	ped (RCP should have been tripped earlie	er) U-1 U-1 AST	·
	nerator boundaries intact team Generator pressures stable	U-1 AS1	·
	maintained between 10%(31%) - 50%	U-1 AST U-1 AST	
• SG levels	ondary radiation normal	TI 1 A C/D	
• Check sec	3	U-1 AS1	
Check secAll se	econdary radiation trends normal for plan	t conditions	
Check secAll seAcceptabl	econdary radiation trends normal for plan e RCS subcooling	t conditions U-1	
Check secAll seAcceptablSecondary	econdary radiation trends normal for plan e RCS subcooling heat sink	t conditions	
 Check sec All se Acceptabl Secondary SG N 	econdary radiation trends normal for plan e RCS subcooling	t conditions U-1 U-1	_
 Check sec All sec Acceptable Secondary SG N > 500 RCS press 	econdary radiation trends normal for plan e RCS subcooling heat sink R levels > 10%(31%)	t conditions U-1 U-1	_

•	Check if CS should be Stopped	U-1
	CS Pumps running	
	• Reset CS signal	
	• Spray Add Tank Lo-2 level lights lit (answer should be no	
	• Cnmt pressure less than 15 psig (it may be)	
	• Sprays operating time greater than 8 hours (it will not be)	
•	Check if RH Pumps should be Stopped	U-1
	• Reset SI	
	 Verify SI ACTUATED permissive light NOT LIT 	
	 Verify AUTO SI BLOCKED permissive light LIT 	
	• Check RCS pressure greater than 325 psig (it won't be so	go to step 10)
•	Check if DG's should be stopped	U-1
	• Check 4 KV ESF busses energized by offsite power	
	• Check 4 KV Non-ESF busses energized by offsite power	
	Stop any unloaded DG	
•	Initiate evaluation of plant status	U-1
	Verify power to a least on RH pump	
	 Verify CNMT sump isol valve position lights LIT 	
	Check Aux Bldg radiation trends NORMAL for PLANT (CONDITIONS
	• Reset CNMT Isol Phase A	
	• Place Hydrogen Monitors in service per BOP PS-9	
	• Consult TSC for obtaining samples	
	Evaluate plant equipment	
	• Prepare Hydrogen Recombiners to run per BOP OG-10	
	 Align SX MDCT for long term cooling per BOP SX-T2 	
	 Shutdown all HD Pumps 	
	 Shutdown FW Pumps per BOP FW-2 	
	 Shutdown all unnecessary CD/CB Pumps per BOP CD/C 	CB-2
	Align NDCT for temperature and level control	
	• Shutdown all unnecessary CW Pumps per BOP CW-2	
	Shutdown chiller on non-operating VC train	
•	Check if RCS cooldown and depressurization is required	U-1
	• Check RCS pressure greater than 325 psig	
•	Check if transfer to cold leg recirculation is required	U-1
	Check ECCS in INJECTION MODE	
	• Check RWST level less than 46% - WHEN YES go to 1B	EP ES-1.3
CON	MMENTS	

EXAMINER'S NOTE:

US directs transition from 1BEP-0 or 1BEP-1 when RWST Level get to 46%. The Crew will transition to 1BEP ES-1.3, Cold Leg Recirculation.

Transition to 1BEP ES-1.3 "TRANSFER TO COLD LEG	G US	
RECIRCULATION"		
Direct operator actions of 1BEP ES-1.3 to establish	US	
the following conditions:		
• Establish CC flow to RH HXs	U-1/U-1 AST	
 Check U-0 CC HX aligned to U-1 		
Check 2 CC Pumps running		
• Open 1CC9412A/B		
• Check CC to RH HX flows > 5000 GPM		
• Check Cnmt floor water level > 8" (13")	U-1 AST	
Align RH pumps suction to Cnmt sumps	U-1 AST U-1 AST	
 Place Train A & B SVAG valve control switch 	ues to CLOSE	
Check RH Pumps both running (1B RH not run		
• 1SI8811A NOT open (GO TO Att. A)	mmg)	
• Check if RH Pump 1A needs to be aligned to CNM	Т Sump U-1 AST	
1SI8811A closed	1 Sump 6-1 AS1	
• Check Train A Recirc flowpath from CNMT Sump	available U-1 AST	
1A RH pump running	available U-I ASI	
)\	
1SI8811A energized (1SI8811A is NOT energy Transfer to cold be recipropletion and establish ECC		
 Transfer to cold leg recirculation and establish ECC recirculation flow before RWST level reaches 7% 	S U-1 AST/U-1	
Place 1A RH pump in PULL OUT		
• Close 1SI8812A		
o Place 1A CS pump in PULL OUT		
• Close 1CS001A		
• Open 1SI8811A (It will not Open)		
o Reopen 1CS001A		
o Restart 1A CS pump (will NOT start)		
 Check if RH Pump 1B needs to be aligned to CNM 	Г Sump U-1 AST	
• 1SI8811B closed (open)		
• Close 1SI8812B		
 Check at least one CNMT Sump Recirc flowpath Es 	stablished U-1 AST	
• 1SI8811A Not Open		
 1B RH Pp Not Running 		
• Close 1SI8812B		
• Close 1SI8812B		
• Trip SVAG valve C/S's and then transition to 1BCA	1.1	
*		
COMMENTS		

EXAMINER'S NOTE:

US directs transition to 1BCA-1.1 since neither train of Emergency Coolant Recirculation could be established.

Transition to 1BCA-1.1 "Loss of Emergency Coolant Recirculation	" US		
Direct operator actions of 1BCA-1.1 to establish the following conditions:	US		
 Check Emergency Coolant Recirc Equip Available At Least One Train Available 1A RH Pp 1SI8811A 1B RH Pp 1SI8811B 	U-1 AST		
 RWST level LO-2 Alarm Lit CNMT sump isol valves OPEN (Dispatch NLO's) CNMT Floor water level at least 8" (13" adverse) CHECK at least one train of Emergency Recirc Equipment Restored (Answer will be NO) 	U-1 AST U-1 AST U-1 AST U-1 AST		<u> </u>
 RESET SI DEPRESS both SI reset Pushbutton 	U-1 AST		
 RESET RWST Auto Swapover Depress SI recirc sump isol valve reset pushbuttons 	U-1 AST		
 * ADD Make-up to RWST as necessary • Initiate BOP SI-13, Filling the RWST 	U-1 AST*	*	*
 CHECK Intact S/G Levels Narrow range level > 10% (31% adverse) Feed flow maintaining BETWEEN 10-50% S/G level 	U-1 AST		
• Initiate RCS Cooldown to 200°F	U-1 AST		
TERMINATE SCENARIO			
COMMENTS			

Appendix D Scenario Outline Form ES-D-1				Form ES-D-1			
Facility:	Byron S	cena	rio No: #	3	Op-Test No:	2008-01	
Examin	ers:				Operators:		
Initial Co	onditions: Rx Po	wer	~ 100%.				
Turnove	er: Continue to o	pera	te at 100°	% Rx Po	wer.		
Event No.	Malf. No.		Event Type*			Event Description	
1	NONE	С	BOP SRO	Shutdown the 1A CW Pp			
2	NONE	R	RO SRO			account for secondary from CW Pp shutdown	
3	MF CV23B 20	С	RO SRO	Failure of 1B Letdown Hx			
4	MF FW22C	С	BOP SRO	1C CD	/CB Pp Trip wit	h Failure of STBY to Auto Start	
5	MF CV10 0 60	I	RO SRO	Failure	of 1CV121 Co	ntroller Low	
6	MF FW02A	С	BOP SRO	1FW01	2B Fails Open		
7	MF RP09B	М	TEAM	Rx Trip	switch on 1PN	105J Fails	

1A AF Pp Trips

Turbine Trip Fails requires Manual Turbine Trip

1A AF Pp trips after 1 minute run

TEAM

TEAM

TEAM

С

С

С

MF TC03

MF FW44

MF FW43

8

10

⁽N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

SCENARIO NUMBER	R: <u>#3</u>
TITLE: Loss of Heat Sink requiring Bleed and Fe	eed
TURNOVER INFORMATION:	
The unit is at 100% power, steady state, equilibrium	n xenon, MOL.
THIS SCENARIO CONTAINS: The following ob	jectives/K/A's
	Shutdown the 1A CW Pp per BOP CW- 2 Load Ramp from 100% down 50 MW Letdown Hx tube leak (Tech. Spec.) 1C CD/CB Pump trips standby fails to start 1CV121 Controller Fails Low 1B TDFWP Trips, 1A MDFWP and SU FWP won't start Rx Fails to trip from 1PM05J Turbine Fails to trip - must use manual trip pushbutton 1A & 1B AFW Pumps fail to start
COMPLETION CRITERIA: hot dry S/G's	Re-initiate feed flow to the
CRITICAL TASKS:	
1. Manually Trip Rx from 1PM06J.	
2. Manually Trip Turbine with manual trip pushl	outton.
3. Initiate Bleed and Feed during 1BFR - H.1 wh	nen required.
Examinees: US	
RO	

ВОР _____

T	.		
Evaluator:	 Date:		

10 CFR 55.45

X 1. Perform pre-startup procedures for the facility, including operation those controls associated with plant equipment that could affect reactivity. <u>X</u> 2. Manipulate the console controls as required to operate the facility between shutdown and designated power levels. X 3. Identify annunciators and condition-indicating signals and perform appropriate remedial action where appropriate. <u>X</u> 4. Identify the instrumentation systems and the significance of facility instrument readings. Observe and safely control the operating behavior characteristics of the facility. <u>X</u> 6. Perform control manipulations required to obtain desired operating results during normal, abnormal, and emergency situations. <u>X</u> 7. Safely operate the facility's heat removal systems, including primary coolant, emergency coolant, and decay heat removal systems and identify the relation of the proper operation of these systems to the operation of the facility. <u>X</u> 8. Safely operate the facility's auxiliary and emergency systems, including operation of those controls associated with plant equipment that could affect reactivity or the release of radioactive materials to the environment. <u>X</u> 9. Demonstrate or describe the use and function of the facility's radiation monitoring systems, including fixed radiation monitors and alarms, portable survey instruments, and personnel monitoring equipment. Demonstrate knowledge of significant radiation hazards, including permissible levels in excess of those authorized, and ability to perform other procedures to reduce excessive levels of radiation and to guard against personnel exposure. Demonstrate knowledge of the emergency plan for the facility, including, as appropriate, the operator's or senior operator's responsibility to decide whether the plan should be executed and the duties under the plan assigned. Demonstrate the knowledge and ability as appropriate to the assigned position to assume the X 12. responsibility associated with the safe operation of the facility. Demonstrate the applicant's ability to function within the control room team as appropriate to the assigned position, in such a way that the facility licensee's procedures are adhered to and that the limitations in its license and amendments are not violated.

Simulator Scenario Review Checklist

	1.	The scenario has clearly stated	Yes	Operato	ors have sufficient time to	Yes
		objectives in the scenario summary.			nt expected activities without	
	2.	The initial conditions are realistic, in	Yes	undue t	ime constraints. Cues are given.	
		that some equipment and/or		If time	compression techniques are used,	
		instrumentation may be out of service, but it		scenario	summary clearly indicates.	
		does not cue crew into expected events.		The sim	ulator modeling is not altered.	Yes
	3.	The scenario consists mostly of	Yes	10.	All crew competencies can be	Yes
		related events.			evaluated.	
	4.	Each event description consists of:	Yes	11.	The scenario has been validated.	Yes
		• the point in the scenario when it		12.	The sampling plan indicates that	Yes
		is to be initiated			the scenario was not used for training	
		• the malfunction(s) that are entered			during the requal cycle. Evaluate the need	
		to initiate the event			to modify/replace scenario if used.	
		 the symptoms/cues that will be visible 		13.	Total malfunctions inserted: 4-8 <u>6</u>	
		to the crew		14.	Malfunctions that occur after	
		 the expected operator actions 			EOP entry: 1-4 <u>3</u>	
		(by shift position)		15.	Abnormal Events: 1-2 3	
		 the event termination point 		16.	Major Transients: 1-21	
	5.	No more than one non-mechanistic	Yes	17.	EOPs used beyond primary	
		failure (e.g., pipe break) is incorporated			scram response EOP: 1-3 2	
		into the scenario without a credible preceding		18.	EOP Contingency Procedures	
		incident such as a seismic event.			used: 0-3 <u>0</u>	
6	ó.	The events are valid with regard to	Yes	19.	Approximate scenario run time:	No
		physics and thermodynamics.			45-60 minutes	
7	7.	Sequencing/timing of events is	Yes	20.	EOP run time:	Yes
		reasonable, and allows for the			40-70% of scenario run time	
		examination team to obtain complete		21.	Crew Critical Tasks: 2-53	
		evaluation results commensurate		22.	Technical Specifications are	Yes
		with the scenario objectives.			exercised during the test.	

SCENARIO OVERVIEW

The unit is at 100% Rx power, steady state, equilibrium xenon, MOL.

The scenario begins with the outside EO reporting the 1A CW pump making a lot of noise and vibrating badly, getting worse. EO recommends immediately shutting down the 1A CW pump.

When notified of the 1A CW pump shutdown, the SM will direct a load reduction of 50 MW to maintain <100% reactor power. The crew will reduce turbine load and control reactor parameters during the load reduction.

After the actions to ramp down 50 MW have been completed the online Letdown Heat Exchanger will develop a tube leak into the CC system. The crew will be alerted to the failure by the CC surge tank hi rad alarm. The crew will implement BOA PRI-6, CC Malfunction procedure to address the failure. The SRO should realize that the crew needs to enter the RCS Leakage Tech Spec 3.4.13 for excessive (> 10 gpm) RCS leakage. Once the crew identifies the online Letdown Heat Exchanger as the problem the crew will swap to the standby Letdown Heat Exchanger and isolate both the CV and CC sides of the leaking Letdown Heat Exchanger.

The next failure to occur will be for the 1C CD/CB pump to trip with a failure of the standby pump to automatically start. The crew will take actions to manually start the standby pump while entering BOA SEC-1, Secondary Pump Trip procedure to address any other plant concerns.

Following the CD/CB pump trip the output of the controller for 1CV121 will fail low causing 1CV121 to drift closed reducing charging header flow. The reactor operator will need to take manual control of the controller and restore charging flow so that pressurizer level can be restored.

The 1B TDFWP Trips and the 1A MDFWP doesn't start to provide replacement feedwater flow. The crew will take actions per BOA SEC-1 to runback the Unit to 780 MW however the runback function on the turbine generator will not work so the crew will have to manually reduce turbine load.

The crew will not be able to maintain SG water levels and will have to trip the unit. This is the start of the major transient in which the reactor will not trip from the front panel 1PM05J. The reactor operator will have to use the alternate reactor trip switch located on 1PM06J. The turbine will also fail to trip and must be manually tripped using the trip pushbutton.

The reduction in feedwater results in lowering SG levels and a signal to automatically start the Aux. Feedwater Pumps. The 1A AFW pump starts but trips on overcurrent immediately after its start. The 1B AFW will start and run for 1 minute and then trip. With no AFW pumps running once the crew transitions out of 1BEP-0, the crew should enter 1BFR - H.1.

The crew will need to initiate an RCS bleed and feed once SG wide range levels fall below 27% (43% adverse). The scenario will be terminated when feedwater flow is re-initiated to a hot dry SG.

SCENARIO SETUP GUIDE

- Initialize IC-22: "full power, steady state, equilibrium xenon, MOL."
- Lineup control boards.
- Place simulator in run (allow simulator to run during board walkdowns and turnover).
- Type **bat f:\BY-SIM3\BY-SIM3** and ensure the following insert:
- Perform TQ-BY-201-0113, Appendix A, "Simulator Exam/ Scenario Reset Checklist"

<u>EVENT</u>	TIME	MALF NO.	DESCRIPTION		
1	0	N/A	Shutdown Standby CW Pp		
2	4	N/A	Ramp Unit from 100% down 50 MW		
3	20	MF CV23B 20	Failure of 1B Letdown Hx		
4	35	MF FW22C	1C CD/CB Pp trip with Failure of STBY to auto start		
5	45	MF CV10 0 60	Failure of 1CV121 Controller Low		
6	60	MF FW02A	1B TDFWP Trip with 1A MDFWP failure to start		
7	65	MF RP09B	Rx Trip switch on 1PM05J failure		
8	65	MF TC03	Turbine trip fails requires trip via manual trip pushbutton		
9	70	MF FW43	1A AFW Pp trips on φC overcurrent immediately after starting		
10	71	MF FW44	1B AFW Pp trips after 1 minute run		

INSTRUCTOR/SIMULATOR RUN AID GUIDE

<u>EVENT</u>	ADDITIONAL INFORMATION
1	After the crew has taken the watch, call as the Outside EO and report the 1A CW pump is vibrating badly and making a lot of noise, rapidly getting worse. Recommend immediate shutdown of 1A CW pump.
3	As directed by the crew when switching from the 1B LD HX to the 1A LD HX: 1CC9452B OPEN mrf cc40 100 1CC9452A OPEN mrf cc37 100 1CV8467A OPEN mrf cv63 100 1CV9452C CLOSE mrf cc39 0 1CV8467A OPEN mrf cv63 100 1CC9452C CLOSE mrf cc39 0 1CC9452C CLOSE mrf cc39 0 1CC9452D CLOSE mrf cc43 0 1CV8467B CLOSE mrf cv64 0 to drain CC surge tk mrf cc16 100
4	Report OC trip on 1C CD/CB pump when dispatched to check bus 159 cubicle 3.
6	To pull FWI fuses mrf fw150 removed mrf fw151 removed to start s/u fw pp l.o. pp mrf fw149 start to start 1B AF pump locally mrf fw196 local
	mrf fw147 on

mrf fw148 on dmf fw44

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT
(Cue)	
• Initial cue is to 1A Circulating Water pump is vibrating b	oadly and making noise.
(Response) Refer to 1BOP CW-2, Circulating Water System Sh	utdown Crew
• Direct actions to shutdown 1A CW Pp	US
 BOP refers to 1BOP CW-2 step F.1 Shutdown 1CW01PA CW Pp Verify discharge valve 1CW001A is closing 	U-1 AST
COMMENTS	

BRIEF DESCRIPTION Ramp Unit down 50 MW			
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT	
Re-activity manipulation for this scenario.			
(Cue) o Per SM, lower power 50 MW to maintain Reactor Power <100%.	er		
(Response) Refer to BGP 100-4, Power Descension	CREW		
• Direct actions to lower Rx Pwr	US		
PROGRAM Turbine Controls to ramp down	U-1 AST		
 BORATE per BOP CV-6 to control Tave-Tref within desired limits 	U-1		
• INSERT Control Rods to maintain ΔI and Tave-Tref	U-1		
• STOP ramp after 50 MW ramp is complete.	US		
COMMENTS			

Cue) Annunciator RM-11 alarm CC Surge Tank Hi-Rad CC Surge Tank level increasing Indicated letdown line flow decreases Response) Refer to BAR's Check CC Surge Tank level Surge tank level > 13% Surge tank level stable (is rising) Check for inleakage from RCP Thermal Barrier RCP Therm Barr CC Wtr Flow alarms Lit OR Seal inj flow abnormally Hi (Both will be no - go to step 6) Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CC side of 1B L/D Hx	EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT U	NSAT	
Cue) Annunciator RM-11 alarm CC Surge Tank Hi-Rad CC Surge Tank level increasing Indicated letdown line flow decreases Response) Refer to BAR's Check CC Surge Tank level Surge tank level > 13% Surge tank level stable (is rising) Check for inleakage from RCP Thermal Barrier RCP Therm Barr CC Wtr Flow alarms Lit OR Seal inj flow abnormally Hi (Both will be no - go to step 6) U-1 Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx				
Annunciator RM-11 alarm CC Surge Tank Hi-Rad CC Surge Tank level increasing Indicated letdown line flow decreases Response) Refer to BAR's U-1 AST/U-1 Check CC Surge Tank level Surge tank level > 13% U-1 Surge tank level stable (is rising) U-1 RCP Therm Barr CC Wtr Flow alarms Lit OR U-1 Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) U-1 Notify Chem to sample CC system for activity U-1 Locate and Isolate inleakage from 1B L/D Hx U-1 Locally Isolate CC side of 1B L/D Hx U-1 Locally Isolate CC side of 1B L/D Hx U-1	Recognize symptoms and respond to a tube leak in the 1B	Letdown Heat Exchar	nger.	
CC Surge Tank level increasing Indicated letdown line flow decreases Response) Refer to BAR's Check CC Surge Tank level Surge tank level > 13% Surge tank level stable (is rising) Check for inleakage from RCP Thermal Barrier RCP Therm Barr CC Wtr Flow alarms Lit OR Seal inj flow abnormally Hi (Both will be no - go to step 6) Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx	Cue)			
Indicated letdown line flow decreases Response) Refer to BAR's Check CC Surge Tank level Surge tank level > 13% Surge tank level stable (is rising) Check for inleakage from RCP Thermal Barrier RCP Therm Barr CC Wtr Flow alarms Lit OR Seal inj flow abnormally Hi (Both will be no - go to step 6) Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx				
Response) Refer to BAR's Check CC Surge Tank level Surge tank level > 13% Surge tank level stable (is rising) Check for inleakage from RCP Thermal Barrier RCP Therm Barr CC Wtr Flow alarms Lit OR Seal inj flow abnormally Hi (Both will be no - go to step 6) Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locatly Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Refer to BAR's U-1 Locally Isolate CC side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Check in Index				
• Check CC Surge Tank level • Surge tank level > 13% • Surge tank level stable (is rising) • Check for inleakage from RCP Thermal Barrier • RCP Therm Barr CC Wtr Flow alarms Lit OR • Seal inj flow abnormally Hi (Both will be no - go to step 6) • Isolate CC System Inleakage • Check inleakage from RCS (cc rad trends rising) • Notify Chem to sample CC system for activity • Locate and Isolate inleakage from 1B L/D Hx • Locally Isolate CV side of 1B L/D Hx • Locally Isolate CC side of 1B L/D Hx • Locally Isolate CC side of 1B L/D Hx	Indicated letdown line flow decreases			
 Check CC Surge Tank level Surge tank level > 13% Surge tank level stable (is rising) Check for inleakage from RCP Thermal Barrier RCP Therm Barr CC Wtr Flow alarms Lit OR Seal inj flow abnormally Hi (Both will be no - go to step 6) Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx 				
 Surge tank level > 13% Surge tank level stable (is rising) Check for inleakage from RCP Thermal Barrier RCP Therm Barr CC Wtr Flow alarms Lit OR Seal inj flow abnormally Hi (Both will be no - go to step 6) Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx 		U-1 AST/U-1		
 Check for inleakage from RCP Thermal Barrier RCP Therm Barr CC Wtr Flow alarms Lit OR Seal inj flow abnormally Hi (Both will be no - go to step 6) U-1 Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx 				
 Check for inleakage from RCP Thermal Barrier RCP Therm Barr CC Wtr Flow alarms Lit OR Seal inj flow abnormally Hi (Both will be no - go to step 6) U-1 Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx U-1 		U-1		
RCP Therm Barr CC Wtr Flow alarms Lit OR Seal inj flow abnormally Hi (Both will be no - go to step 6) Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Representations U-1 U-1 U-1 V-1 V-1 V-1 V-1 V-1		U-1		-
Seal inj flow abnormally Hi (Both will be no - go to step 6) U-1 Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Solution Locally Isolate CC side of 1B L/D Hx Solution Sol				
 Isolate CC System Inleakage Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx U-1 U-1 U-1 U-1 				
Check inleakage from RCS (cc rad trends rising) Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx U-1 U-1 U-1 U-1 U-1 U-1 U-1 U-		to step 6) U-1		
Notify Chem to sample CC system for activity Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx U-1 U-1 U-1 U-1 U-1 U-1 U-1 U-	, and the second			
Locate and Isolate inleakage from 1B L/D Hx Locally Isolate CV side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx Locally Isolate CC side of 1B L/D Hx U-1 U-1 U-1 U-1 U-1 U-1 U-1 U-	` ` ` ` ` ` ` ` ` ` ` ` ` ` ` ` ` ` ` `	U-1		
		U-1		
		U-1		-
		U-1		-
COMMENTS	 Locally Isolate CC side of 1B L/D Hx 	U-1		
COMMENTS				
	COMMENTS			

EVENT4			
BRIEF DESCRIPTION 1C CD/CB Pp Trips with failure of St	andby Pp to auto st	art	
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UN	ISAT	
Recognize symptoms and respond to a trip of the 1C CD/CB P standby Pp to auto start.	p with a failure of t	he	
(Cue) o Annunciator 1PM03J "CD/CB Pp Trip"			
(Response)			
 Direct actions to respond to the tripped CD/CB Pp Enter 1BOA SEC-1, Secondary Pump Trip 	US Crew		
Check Turbine Load			
• Check Turbine Load > 700 MW	U-1 Ast		
 Check Standby CD/CB Pp Running (failed to auto start) 			
• Start Aux Oil Pp for Standby CD/CB Pp	U-1 Ast		
• Start Standby CD/CB Pp	U-1 Ast		
 Check at least 3 CD/CB Pp's running Check FW flow > or = to Steam Flow 	U-1 Ast		
• Check F w How > or – to Steam Flow	U-1 Ast		
 Check FW Pp's NOT Cavitating 			
• Close recirc valve on tripped CD/CB Pp	U-1 Ast		
• Check FW Pp discharge flows Oscillating (shouldn't be)			
Check CD/CB Pp Flow Restored			
• Check Alarm Not Lit CB Pp Dsch Flow Hi (1-17-B11)	U-1 Ast		
• Check Alarm Not Lit FW Pp NPSH Low (1-16-E1)	U-1 Ast		
Check Plant Status			
• Check Alarm Not Lit PDMS Inop (1-10-E8)	U-1		
• Check 1BOL 3.h Pwr Dist Monitoring Sys Not Entered	U-1 Ast		
• Check Alarm Not Lit PDMS Limit Exceeded (1-10-D7)	U-1 Ast		
 Check Delta I near target 	U-1 Ast		
 Check Alarm Not Lit Rod Bank Lo Insert Limit (1-10-B 			
Check Alarm Not Lit Turbine runback	U-1 Ast		
• Check Alarm Not Lit Loss of Turb Load C7 (1-BP-4.6)	U-1 Ast		
Restore Plant Conditions			
Adjust Boron Conc. as necessary	U-1		
• Verify control in Auto	U-1 Ast		
• Review BOP CD/CB-1 & CD/CB-2	U-1 Ast		
 Adjust SG blowdown flows and calorimetric inputs 	U-1 Ast		

 Verify DEHC feedback loop In Service Notify Chemistry to monitor secondary systems	U-1 Ast U-1 Ast	
• Return to Procedure and Step in Effect	US	

BRIEF DESCRIPTION The controller for 1CV121 fails low	<i>I</i> .
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT
Recognize symptoms and respond to the 1CV121 control valve co	ontroller output failing low.
 (Cue) Charging Flow starts lowering. RCP seal injection low starts lowering. VCT level starts increasing. 	
(Response)	
• PLACE Flow Controller 1FK-121 in Manual	U-1
ADJUST Controller Output to re-open 1CV121	U-1
EXAMINER'S NOTE: Depending on how fast the Reactor Operator catches the	e failure the crew may not
EXAMINER'S NOTE: Depending on how fast the Reactor Operator catches the enter a procedure to correct the failure. The SRO may defailed controller to manual and re-establish charging flow may enter the BAR for low charging low BAR 1-9-D3, 11 Primary Plant Leakage, or 1BOA RCP-2 Loss of Seal Inc.	irect the RO to place the w. If the crew delays they BOA PRI-1 Excessive
Depending on how fast the Reactor Operator catches the enter a procedure to correct the failure. The SRO may defailed controller to manual and re-establish charging flow may enter the BAR for low charging low BAR 1-9-D3, 11	irect the RO to place the w. If the crew delays they BOA PRI-1 Excessive
Depending on how fast the Reactor Operator catches the enter a procedure to correct the failure. The SRO may difailed controller to manual and re-establish charging flow may enter the BAR for low charging low BAR 1-9-D3, 11	irect the RO to place the w. If the crew delays they BOA PRI-1 Excessive
Depending on how fast the Reactor Operator catches the enter a procedure to correct the failure. The SRO may defailed controller to manual and re-establish charging flow may enter the BAR for low charging low BAR 1-9-D3, 11	irect the RO to place the w. If the crew delays they BOA PRI-1 Excessive
Depending on how fast the Reactor Operator catches the enter a procedure to correct the failure. The SRO may defailed controller to manual and re-establish charging flow may enter the BAR for low charging low BAR 1-9-D3, 11	irect the RO to place the w. If the crew delays they BOA PRI-1 Excessive
Depending on how fast the Reactor Operator catches the enter a procedure to correct the failure. The SRO may defailed controller to manual and re-establish charging flow may enter the BAR for low charging low BAR 1-9-D3, 11	irect the RO to place the w. If the crew delays they BOA PRI-1 Excessive
Depending on how fast the Reactor Operator catches the enter a procedure to correct the failure. The SRO may defailed controller to manual and re-establish charging flow may enter the BAR for low charging low BAR 1-9-D3, 11	irect the RO to place the w. If the crew delays they BOA PRI-1 Excessive
Depending on how fast the Reactor Operator catches the enter a procedure to correct the failure. The SRO may defailed controller to manual and re-establish charging flow may enter the BAR for low charging low BAR 1-9-D3, 11	irect the RO to place the w. If the crew delays they BOA PRI-1 Excessive

EVENT 6	
BRIEF DESCRIPTION 1B TDFWP Trips and the 1A MD	FWP does not start.
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT
Recognize symptoms and respond Main Feedwater Pump trip.	
 (Cue) Annunciator (1-16-B1) "FW Pump 1B Trip" Annunciator (1-15-A4) "SG 1A Flow Mismatch FW Flow Annunciator (1-15-B4) "SG 1B Flow Mismatch FW Flow Annunciator (1-15-C4) "SG 1C Flow Mismatch FW Flow Annunciator (1-15-D4) "SG 1D Flow Mismatch FW Flow 	Low" Low"
 (Response) Implement 1BOA SEC-1, Secondary Pump Trip Inform SM of unit status/potential EP event 	US/STA
Close FW Pp Recirc ValveClose 1FW012B	U-1 Ast
 Check Turbine Load Check Turbine Load > 700 MW Check At Least One FW Pp Running 	U-1 Ast
 Restore Feed Flow Check FW Pp 1A Available Start Aux Oil Pp for 1A FW Pp Verify 1FW016 controller in Manual at 20% demand Start 1A FW Pp (Pp Trips on OC immediately) 	U-1 Ast
 Reduce Turbine Load Initiate CD/FW Runback Runback Pushbutton (Will not work) OWS panel G-5512 Check Turbine Load Dropping Verify Rod Control in AUTO Initiate Boration as necessary 	U-1 Ast

EXAMINER'S NOTE:

US directs transition from 1BOA SEC-1 to 1BEP-0, Reactor Trip Response or Safety Injection when he realizes SG levels can not be maintained.

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT
Implement 1BEP-0, Reactor Trip Response or Safety Injection	on
 Manually actuate Reactor Trip from 1PM05J (Trip switch does not work on 1PM05J) 	U-1
*• Manually actuate Reactor Trip from 1PM06J (Trip switch works on 1PM06J)	U-1
 Reactor trip verified Rod bottom lights Reactor trip/Bypass breakers Neutron flux dropping 	U-1
 Turbine trip verified by (Auto trip fails) GVs closed TVs closed Must use Manual trip pushbutton to actuate Turbine 	U-1 ASTe trip
 4 KV ESF busses are energized Bus 141 energized Bus 142 energized 	U-1 AST
 SI Status SI is not actuated or required SI ACTUATED permissive light lit (Not Lit) SI first out annunciator (Not Lit) SI equipment automatically actuated (Not runn 	CREWning)
Transition to 1BEP ES-0.1, Reactor Trip Response	CREW

COMMENTS			
_			
=			

EXA	MIN	JER'S	NOTE:

1BEP ES-0.1, Reactor Trip Response

The 1A AFW Pp trips on Overcurrent after initially starting. The 1B AFW Pp trips after a two minute run. US directs transitions to 1BEP ES-0.1 because SI has not actuated and is not required. The Crew will transition to 1BFR H-1 when <u>all</u> SG narrow range levels are < 10% (31% Adverse).

	, 1		
•	Verify Generator Trip • Main Transformer Output Breakers Open • Verify GCB 3-4 • Verify OCB 4-5	U-1 AST	
•	Maintain RCS Temperature Control • RCP Running RCS Temp stable at or trending	U-1	
•	Determine if Boration is Required • RCP's Running	U-1 AST/U-1	
•	Check FW Status • FW isol monitor lights LIT • Trip all HD Pp's • Total Feed Flow to SG's > 500 gpm • SG Blowdown isol valves closed 1SD002A-H	U-1 AST/U-1	
•	Verify all Control Rods Fully Inserted	U-1	
•	Check PZR Level > 17% • Check Charging and Letdown in Service • PZR Level Trending to Program Level	U-1	
•	Check PZR Pressure Control • PZR Pressure > 1829 psig • PZR Pressure Stable at or Trending to 2235 ps	U-1	
•	Check SG Level • Check Narrow range SG Level > 10% in any S	U-1	

• Control Feed flow to maintain between 10%-50% SG lvl

 Transition to 1BFR H-1, Loss of Heat Sink when all narrow range levels drop below 10% and AF flow is < 500 gpm (AS DIRECTED BY CSF MONITORING 	
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT
EXAMINER'S NOTE: US will direct transitions to Step 13 and initiate Bleed and Fee levels are < 27% (43% Adverse).	d when three SG wide range
 Check if Secondary Heat Sink is Required RCS Pressure > any Non-Faulted SG Pressure RCS Temperature > 350F 	U-1
 Check Cent Chg Pp Status Cent Chg Pp at least one Running 	U-1
 Check if Bleed and Feed is Required SG Wide range level in any three SG's < 27% PZR pressure > than 2335 psig due to Loss of I 	
 Try to Establish AF to at least one SG Check SG Blowdown isol Valves Closed 1SD0 Check SG Blowdown sample isol Valves Close Prior to initiating feed flow review Att B Check AF Pp SX Suct Vlvs ARMED alarm No Check AF Pp's Both Running (Attempt manual Check AF test valves OPEN 1AF004A&B Check AF isol Valves OPEN 1AF013A-H Check AF Throttle Valves OPEN 1AF005A-H Check Total AF flow > 500 gpm (It won't be) 	ed 1SD005A-D ot Lit il start)
Reduce RCP Heat Input Stop all RCP's	U-1
Prepare FW System for Restoration	

- Check at least one CD/CB Pp Running
 Place FW Reg valve in Manual at Zero Demand
 Place FW Bypass Reg Valve in Manual at Zero Demand
 Place Tempering flow control Valve in Manual at Zero Demand

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT
 Reset FW Isolation Signal Check FW isolation Aux Relay lights Lit Check SI Not Actuated Depress both FW Isolation Reset Pushbuttons Check FW Isol Actd relay lights Not Lit Dispatch an operator to pull FW Isol Relay Fuses 	U-1
 Try to Establish Main FW flow to at least one SG Check Startup FW Pp or 1A MDFWP available Open FW tempering isol valves 1FW035A-D Check at least two CD/CB Pp's running Check Bus 159 Energized Verify Startup FW Pp Aux Oil Pp Running Verify Discharge Valve 1FW059 Open Open recirc valve 1FW076 and place c/s to Modulate Close Main FW Pp recirc valves 1FW012A-C Start Startup FW Pp (Pp won't start) 	U-1
• Try to establish 1A MDFWP (Unavailable previously)	U-1
 Try to establish Condensate Booster Pp flow Close FW Pp recirc valves Start Aux Oil Pp for selected Pp Open selected FW Pp discharge valve 1FW002A-C, 1FV Check SI Not Actuated Check RCS Pressure < 1930 psig P-11 Lit (It won't be) Depressurize RCS to 1880 psig by Opening one PZR PC pressure)

$\mathbf{F}\mathbf{X}\mathbf{\Lambda}$	MIN	JFR'S	NOTE.

US will direct transitions to Step 13 and initiate Bleed and Feed when three SG wide range levels are < 27% (43% Adverse). Step 13 through 16 must be perform without delay to ensure adequate core cooling is maintained.

•*	Establish RCS Feed Path • Stop all RCP's • Actuate SI	U-1	*	*	*
•*	Verify RCS Feed Path Check Cent Chg Pp's at least one running or Check SI Pp's at least one running Check ECCS Valve Alignment Group 2 Injection I	U-1 AST/U-1_	*	*	*
•*	Establish RCS Bleed Path • Verify PZR PORV isol Valves Energized and OPE • Open PZR PORV's 1RY455A &1RY456	U-1 AST/U-1 En	*	*	*
•*	Verify Adequate RCS Bleed Path • Verify PZR PORV Both Open • Verify PZR PORV's isolation valves Both Open	U-1 AST/U-1	*	*	*

EXAMINER'S NOTE:

Once RCS Bleed and Feed has been established the 1B AF Pp will become available. The scenario will end when the crew re-establishes feed flow to a Hot Dry SG per Att B.

Attachment B Feed Flow Limitations:

- 3 After Initiation of Bleed and Feed with Stable or Dropping Core Exit Temperatures
 - A. Feed any non-dry SG's as follows:
 - If SG feedlines are not voided, then feed non-dry SG at desired rate
 - If SG feedlines are voided, then feed the non-dry SG between 60-80 gpm for 10 minutes prior to restoring desired feedwater flow.
 - B. If all SG's are dry, then feed one SG as follows:
 - Feed associated SG between 60-80 gpm for 10 minutes
 - After the 15 minutes, feed at a rate NOT to exceed 100 gpm
 - When associated SG Wide Range level is > 10% feed flow may be raised as desired

ΓERMINATE SCENARIO		
COMMENTS		

Appendix	D		Sco	enario Outline	Form ES-D-1
Facility:	Byron S	cena	rio No: <u>#</u>	4 Op-Test No: 2	008-01
Examiners: Operators:					
Initial Co	onditions: Rx Pc	wer ·	~ 28%.		
Turnove	er: Continue to s	hutdo	own the F	Rx.	
Event No.	Malf. No.		Event Type*	С	Event Description
1	NONE	N	BOP SRO	Swap into Shutdown El	ectrical Lineup
2	NONE	R	RO SRO	Ramp down 50 MW per	r turnover
3	IFM RX10A	_	RO SRO	1PT-505 Fails High	
4	IMF CC01B	С	BOP SRO	1A CC Pp Trip, 1A CC	Pp Fails to auto start
5	IMF CV05 600	I	RO SRO	Failure of 1CV131 Conf	troller
6	IOR ZDI1FW012C OPEN	C	BOP SRO	1FW012C Fails Open	
7	IMF FW19B 3.5	M	TEAM	1B SG Feed Line Break	k inside Cnmt.
8	IMF TH03B	М	TEAM	1B SGTR 400 gpm	

MSIV's Fail to Auto Close Manual action required

TEAM

С

IRF RP 60, 61, 34, 35 OUT

9

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

SCENARIO NUMBER: #4

TURNOVER INFORMATION:

The unit is at 28% power, steady state equilibrium xenon, BOL, 24 EFPH Unit was raising power following a refueling outage when the 1A AFP was found to be INOPERABLEA shutdown is in progress due to 1A AFP inoperability, BOL 7.5 condition A has been in effect for 68 hours, and the repairs will not be completed for 12 hours. Currently at step 14 of 1BGP 100-4 to switch to the shutdown electrical lineup. Continue in 1BGP 100-4 to ramp Unit 1 off line.

THIS SCENARIO CONTAINS: The following objectives/K/A's:

004000A4.01	(3.8/3.9)	IV.B.GP-04/VIII.B.GP-04-B	Perform a Power Descension
008000A2.01	(3.3/3.6)	IV.D.OA-10/VIII.D.OA-16-B	Respond to a Loss of the CC
			System
012000A2.05	(3.1/3.2)	IV.D.OA-36/VIII.D.OA-10-B	Respond to a 1PT-505
			Failure
000040AA2.04	(4.5/4.7)	IV.D.EP-06/VIII.D.EP-001-E	Respond to a Feedline Break
			Inside Cnmt
000038EA2.02	(4.5/4.8)	4D.EP-05/8D.EP-001-E/013-B	Respond to a SGTR
000040AA2.01	(4.2/4.7)	IV.D.CA-05/VIII.D.CA-007-B	Respond to a SGTR with
			LOCA-Subcooled Recovery

COMPLETION CRITERIA: Completion of the 1BCA-3.1 cooldown determination step.

CRITICAL TASK(S):

Isolate 1B Steam Generator before transition out of 1BEP-2.
 (ERG Critical Task number - E-2--A) (K/A number - 000040AA1.10 importance - 4.1/4.1)

Examinees:	US	U-1	
	U-1AST		
Evaluator:			Date:

SCENARIO OVERVIEW

The unit is at 28% power, steady state, equilibrium xenon, BOL. Unit one is ramping down to take the unit offline to comply with a Tech Spec LCO 3.7.5 condition A and B.

The crew will perform actions of 1BGP 100-4 to swap to the shutdown electrical lineup.

After electrical lineup is switched, a failure of the turbine impulse pressure transmitter 1PT-505 fails high. The crew will implement 1BOA INST-2 to address the failure. Technical Specification 3.3.1 applies.

After the actions of 1BOA Inst-2 are finished, the 1A CC pump trips. The 1B CC pump will fail to auto start on low CC pressure and the crew will manually start the 1B CC pump.

After 1B CC pump is started and the failure has been addressed, 1CV131 auto controller fails to 0 demand. The will take control letdown pressure in manual.

After actions are finished for 1CV131 failure, 1FW012C, 1C FW pump recirc valve, fails open the crew will perform actions of BAR or 1BOA SEC-1 to restore FW flow, or trip the reactor.

After the plant has been stabilized after 1FW012C failure, or if the plant is manually tripped in response to the failure, a FW line break inside CNMT occurs requiring the crew to trip the reactor (if not previously tripped.).

The crew should manually initiate a Main Steam Isolation and SI and complete the steps 1BEP-0. If the conditions requiring an SI are not identified before the completion of 1BEP-0 step 4 the crew may transition to 1BEP ES-0.1. When the crew determines that a SI is required they will manually initiate an SI and return to step 1 of 1BEP-0. The MSIVs will not close from an auto signal but will respond to a manual Main Steam Isolation. The crew should identify the 1B SG is faulted and transition to 1BEP-2. 1B DG doesn't automatically start on the SI signal, but can be manually started.

After the 1B SG has blown down to 25#, and RCS pressure is rising, a SGTR will occur on the 1B SG. If conditions for the SGTR are not present while in 1BEP-2 the crew will transition to 1BEP-1. When 1BEP-1 is entered the crew will transition to 1BEP-3 from the OAS. While in 1BEP-3 the crew will determine the need to transition to 1BCA-3.1 to stabilize conditions based on 1B SG pressure <320#.

Completion criteria is Completion of the 1BCA-3.1 cooldown determination step (1BCA 3.1 step 10.)

SCENARIO SETUP GUIDE

- Initialize IC 180, 28% power, steady state, increasing xenon, BOL.
- Verify 1A CC pump is in operation
- Set Xenon to 0 (zero) on Control Panel Fast Speed box.
- Place C/O tag on 1A AFP control switch, with switch in PTL.
- Place simulator in run (allow simulator to run during board walkdowns and turnover).
- Provide turnover forms and marked up copy of 1BGP 100-4T1 with exceptions marked to start at step 14.
- Provide ReMa for plant rampdown.
- Run batch file **f:\by-sim4\by-sim4** and verify the following:
 - IRF RP 60 OUT MSI Failure
 - IRF RP 61 OUT MSI Failure
 - IRF RP 34 OUT MSI Failure
 - IRF RP 35 OUT MSI Failure
 - IMF CC02B 200 1B CC Pump auto start failure
 - IMF FW43 1A AFP Trip (to prevent start of 1A AFP)
 - MRF RP73 1B DG auto start failure
- EVENT 1 "YCP0422.LT.25"
- Command "IMF TH03B 400 120"

EVENT	TIME	MALF NO.	DESCRIPTION
A			Swap Shutdown Electrical lineup
В	2	IFM RX10A	1PT-505 fails high (SDG RX11)
C	12	IMF CC01B	1A CC pump trip, 1B CC pump fails to auto start
D	20	IMF CV05 600	1CV131 controller failure
E	28	IOR ZDI1FW012C O	PEN Fails 1FW012C open
F	35	IMF FW19B 3.5 Note 1	1B SG feed line break inside CNMT
G	Note 2	IMF TH03B	1B SGTR (400 gpm) (SDG TH4)

- **Note 1:** MSIVs fail to auto close inserted in batch file.
- **Note 2:** Inserted by trigger 1 at 120 seconds after the 1B SG pressure drops to 25 psig.

INSTRUCTOR/SIMULATOR RUN AID GUIDE

<u>EVENT</u>	ADDITIONAL INFORMATION
A	Acknowledge power descension when contacted as Electric Operations/SM (as required).
В	Acknowledge Turbine Impulse pressure channel failure/EP review request when contacted as SM (as required). Trip bistables using appropriate RFs when requested (in cae). Procedure is 1BOA Inst-2, Att D for 1PT-505.
С	Acknowledge 1A CC pump trip/EP review request when contacted as SM (as required). When requested to investigate, report a Phase A overcurrent flag at the 1A CC pump breaker. Report results 4 minutes after request.
D	Acknowledge report of failed 1CV131 control failure as SM (as required)
E	Acknowledge report of failure of 1FW012C as SM (as required)
F	Acknowledge Rx Trip & SI/EP review request when contacted as the SM (as required).
	Acknowledge procedure transitions when contacted as the SM (as required).

REMOTE FUNCTIONS/OVERRIDES PER EVENT

B IRF RP20 OPEN/CLOSE Cabinet 1 Door Open/Close (SDG RX19)

RX143	P13 input to P7	MRF RX143 (TRIP) PB505A; C1-742
BS-1 (SDG RX1	1)	
RX149	AMS MRF RX149	(TRIP) to place Operating
	(SDG RX11)	
		Bypass Switch (SW12) in the TIP 1
position.		

RP91 AMS MRF RP91 (ACTION)

(TEST-TRIP) Operating Bypass

Test Input switch (SW11) to test-trip.

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT	
Continue with actions to ramp the unit offline.			
Cue)			
From Turnover: Continue with 1BGP 100-4 Step 14 to switch to the shutdown electrical lineup.			
Response)			
Direct U-1 AST to continue with 1BGP 100-4 Step 14	US		
Turn on synchroscope for SAT feed to Bus 143	U-1AST		
Close SAT feed 1432			
Open UAT feed 1431			
Turn off synchroscope			
Turn on synchroscope for SAT feed to Bus 144	U-1AST		
Close SAT feed 1442			
Open UAT feed 1441			
Turn off synchroscope			
Turn on synchroscope for SAT feed to Bus 157	U-1AST		
Close SAT feed 1572			
Open UAT feed 1571			
Turn off synchroscope			
Turn on synchroscope for SAT feed to Bus 156	U-1AST		
Close SAT feed 1562			
Open UAT feed 1561			
Turn off synchroscope			
Feedback)			
Annunciator (1-20-E1) UAT/SAT PARALLEL FEED alarm whenever both breakers to a bus are shut			
All busses aligned to SAT			
7 III ousses anglied to DAT			

EXPECTED OPERATOR/PLANT RESPONSE	SAT	UNSAT	
N/A			
Recognize symptoms and respond to 1PT-505 failure			
(Cue)			
o 1PI-505 decreasing			
Annunciator (1-14-D1) "TAVE CONT DEV HIGH" Control rod inward motion			
Control rod inward motion $T_{REF} ext{ decreasing on } 1TR-0412$			
REF decreasing on TTR 0.12			
(Response)			
Implement 1BOA INST-2 "OPERATION WITH A FAIL	LED US		
INSTRUMENT CHANNEL" Att. D and direct operator			
actions to establish the following conditions: Place rod control in MANUAL	TT 1		
	U-1 U-1 AST		
 Restore steam dumps Place MS HDR PRESS Controller in MANUAL 	U-1 AS1		
and set to 0			
 Place MODE Switch to STEAM PRESS Mode 			
 Place MS HDR PRESS Controller in AUTO 			
• 1PT-505 channel defeated	U-1/U-1 AST		
• Operator dispatched to trip bistable for 1PT-505	U-1		
• Check T_{AVE} - T_{REF} deviation	U-1		
■ Insert CB D rods as needed to match Tave-Tref	** 4		
Rod control returned to AUTO	U-1		
• Check ATWS Mitigation System	U-1		
Trip AMS bistableCheck P-13 interlock NOT lit	U-1 AST		
Review Tech Spec 3.3.1, Rx Trip Instrumentation	US US		
Condition A & D			
(Verify or place P-13 in required state in 1 hour)			
 Inform SM of unit status/EAL potential 	US		
Notify IM Dept. to investigate 1PT-505 failure	US		
(Feedback)			
o Annunciator (1-14-E6) "TURB IMPULSE PRESS (OOS"		
o T _{REF} returned to normal value via 1PT-506			

BRIEF DESCRIPTION 1A CC Pump Trip/1B CC Pump Fails to	o Start		
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT	
(Cue) o No CC pump run lights lit o 1A CC pump trip light lit o Annunciator (1-2-A4) "CC PUMP TRIP" o Annunciator (1-2-B5) "CC PUMP DSCH PRESS LOW"			
(Response)			
EVALUATOR'S NOTE: Crew may take actions using BAR alone, or enter 1BOA Pri-6.			
 Respond to BAR 1-2-A4 "CC PUMP TRIP" Manually start 1B CC Pump Verify adequate CC flow to affected components Place 1A CC Pump Switch to Pull To Lock (PTL) 	U-1 AST U-1 AST U-1 AST U-1 AST	<u></u>	
 Implement 1BOA PRI-6 "COMPONENT COOLING MALFUNCTION" & direct actions to establish the following conditions: 	US		
 Verify CC surge tank level > 13% and stable Manually start 1B CC pp Verify annunciator 1-2-B5 NOT lit 	U-1 AST U-1 AST U-1 AST	<u> </u>	
 Check CC system/component temperatures normal CC heat exchanger outlet temp < 105°F Check RCP cooling Letdown HX temperature alarms NOT lit 	U-1 AST		
 Check CC system status Check CC system rad monitors normal Ensure CC surge tank level between 50% - 65% Ensure CC system pressure > 85 psig 	U-1 AST		
 Dispatch operators to investigate cause of CC pump failures 	U-1 AST/U-1		
• Review Tech Spec 3.7.7, CC System; while only 1CC pump is available (Restore 1 pump in 7 days)			
 Direct Crew to restore U-0 CC Pump to A/T Verify U-0 CC pump is mechanically aligned Inform SM of unit status/potential EAL event (Feedback) 	US U-1 AST US		
1B CC pump runningCC annunciators clear			
COMMENTS			

EVENT C

SAT	UNSAT	
of taking manua	l control	
CREW		
U-1		
U-1		
U-1		
US	. <u> </u>	
	CREW U-1 U-1	SAT UNSAT

EV	YENT <u>E</u>		
BR	RIEF DESCRIPTION <u>1FW012C fails open</u>		
EX	RPECTED OPERATOR/PLANT RESPONSE N/A	SAT UNSAT	
Cı tri _l NC	VALUATOR'S NOTE: rew may respond to this using either the listed BAR response pped FW pump. OTE that a possible response to 1BOA Sec-1 is to consider N e Reactor.	Ç	
(C) 0 0 0 0	Annunciators (1-15-A4 thru D4) "S/G 1_ FLOW MISMA Annunciators (1-15-A9 thru D9) "S/G 1_ LEVEL DEVIA Steam flow/feed flow mismatch on all SGs All FRVs opening/open All SG levels dropping		
(R	esponse)		
O.D.	Respond to BAR 1-15-A4-D4 to "Take manual control of Feed flow and Balance Feed flow with Steam flow to stabilize S/G level." by adjusting 1C FW pump flow.	U-1 AST	
OR	Implement 1BOA SEC-1 "SECONDARY PUMP TRIP" and direct recovery actions:	US	
•	Attempt to close 1C FW pump recirc valve, 1FW012C Check NPSH Low Alarm (1-16-E1) not lit Restore plant conditions Adjust boron concentration as necessary Adjust 1C FW pump flow OR Start 1A FW pump Verify HD, CD, CB, GS valves in auto	U-1 AST	
•	Check feed flow restored	US/U-1 AST	
•	Operator is dispatched to locally investigate 1FW012C	US	
•	Inform SM of unit status/potential EAL event	US	
OR	Implement 1BOA SEC-1 "SECONDARY PUMP TRIP" and direct Reactor Trip.	US	
CC	DMMENTS		

EVENT F			
BRIEF DESCRIPTION <u>1B SG Feedline Break Inside Cl</u>	NMT		
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT	
Recognize symptoms and respond to conditions requiring a I	Rx trip		
(Cue)			
o Cnmt pressure increasing			
o 1B SG level decreasing	IIIGII I ONIII		
o Annunciator (1-15-B9) "S/G 1B LEVEL DEVIATION o 1B SG FW flow > steam flow	HIGH LOW"		
o IB SG FW flow > steam flow o Annunciator (1-1-A2) "CNMT DRAIN LEAK DETEC	T FI OW HIGH"		
. Annunciator (1-1-A2) CIVIVI DRAIN LEAR DETEC	1 I'LOW IIIOII		
(Response)			
Initiate a manual Rx trip/announce a Rx trip	CREW		
Implement 1BEP-0 "REACTOR TRIP OR SI"	IIS		
Immediate operator actions of 1BEP-0 establish	<u> </u>		
the following conditions:			
Reactor is tripped	U-1		
All rod bottom lights lit	~ <u></u>		
Rx Trip and Bypass breakers open			
Neutron flux decreasing			
Turbine is tripped	AST		
• TVs closed			
 GVs closed 			
 4 KV ESF busses are energized 	AST		
• Bus 141 energized			
• Bus 142 energized			
(Feedback)			
o All rod bottom lights lit			
o Neutron flux decreasing			
o Rx Trip and Bypass breakers open			
o Ita IIIp und Bypuss steakers open			
COMMENTS			
COMMENTS			

SX pumps running

	BRIEF DESCRIPTION	1B SG Feedline Break In	nside CNMT
--	-------------------	-------------------------	------------

EXPECTED OPERATOR/PLANT RESPONSE	SAT UNSAT
N/A	
EVALUATOR'S NOTE:	GI L NOT
Depending on the timing of the event and the crew's response	
MANUALLY actuated, or AUTOMATICALLY actuated. If n MANUALLY actuate SI.	ot yet actuatea, the crew should
IMINOTELLI uctuute 51.	
The crew MAY manually actuate Main Steam Line Isolation	to limit the fault before CNMT HI-
2 (8.2#) is reached.	
CI Ctatas	
• SI Status	CDEW
SI is actuated/required SI A CTI A TED page issue light lit	CREW
o SI ACTUATED permissive light lit o SI first out annunciator lit	
 SI equipment automatically actuated Actuate SI 	U-1/AST
Actuate 51	U-1/AS1
Continue with implementation of 1BEP-0	
to establish the following conditions:	
• FW isolated	AST
 FW pumps tripped 	
FWI monitor lights lit	-
• FW pump discharge valves closed (1FW002A	
ECCS pumps running	U-1
CV pumps running	
RH pumps running SI running	
• SI pumps running	AST
 RCFCs running in accident mode RCFC accident mode status lights lit 	AS1
• Refre accident mode status lights in	
CNMT Phase A valves closed	AST
• Group 3 monitor lights lit	
 CNMT VENT isolation valves closed 	AST
• Group 6 monitor lights lit	
AF pumps running	AST
AF pumps running	
• 1AF013 valves open	
• 1AF005 valves throttled	
 CC pumps running 	AST

AST____

COMMENTS		

BRIEF DESCRIPTION <u>1B SG Feedline Break Inside CNMT</u>

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT	
Main Steamlines are isolated (MSIV do NOT auto close)	U-1/AST		
• Close MSIVs via MSLI switch or individual switches	A CITE		
• Containment Spray is not required	AST		
• Cnmt pressure < 20 psig			
Examiner's Note: AF to the 1B SG may be throttled to zero or isolated at the next ste	<i>p</i> .		
	-		
• Verify total AF flow	AST		
• > 500 gpm			
• Control feed flow to maintain SG NR level between 10% (31%) - 50%			
 No SG level increasing in an uncontrolled manner 			
The 20 to the more and an encountered manner			
 ECCS valves aligned with flow 	U-1		
• Group 2 cold leg injection monitor lights lit			
• > 100 gpm on indicator 1FI-917			
• Check at least 1 PZR PORV relief path available	U-1		
Generator tripped Spritch and handress areas	AST		
Switch yard breakers openPMG breaker open			
Diesel Generators running	AST		
• 1A Diesel Generator running	1101		
• Start 1B Diesel Generator			
 Verify 1SX169A and B open 			
Operator dispatched			
• Ventilation aligned for emergency operation U-2	AST /AST		
 PZR PORVs and spray valves closed 	U-1		
• RCS temperature < 557°F	U-1		
RCPs running	U-1		
 Inform SM of unit status/potential EP event 	US		
COMMENTS			
COMMENTS			

BRIEF DESCRIPTION <u>1B SG Feedline Break Inside CNMT</u>

Recognize symptoms and respond to a faulted steam generator in accordance with 1BEP-0 "REACTOR TRIP OR SAFETY INJECTION" (Cue) 0	EXPECTED OPERATOR/PLANT RESPONSE	SAT	UNSAT	
"REACTOR TRIP OR SAFETY INJECTION" (Cue) 1 B SG depressurizing in an uncontrolled manner 1 B SG is completely depressurized. (Response) Announcement of faulted steam generator symptoms CREW	N/A			
(E-2A) (000040AA1.10 4.1/4.1) • AF valves closed (1AF013B/F) • Main feedwater isolated • 1B SG PORV closed (1MS018B) • Blowdown isolation/sample valves closed (1SD002E/F, 1SD005C) • Monitor AF pump suction pressure • Informs SM of unit status US Examiner's Note: • If Secondary Radiation is indicated, the crew will transition to 1BEP-3. (Steps continued on page 15.) Transition to 1 BEP-1 "LOSS OF REACTOR OR SECONDARY COOLANT" US (Feedback) • MSIVs closed • 1AF013B and 1AF013F closed	Recognize symptoms and respond to a faulted steam generate "REACTOR TRIP OR SAFETY INJECTION" (Cue) o 1B SG depressurizing in an uncontrolled manner o 1B SG is completely depressurized. (Response) Announcement of faulted steam generator symptoms Transition to 1BEP-2 "FAULTED SG ISOLATION" • Request STA function at transition from 1BEP-0 • Identify at least 1 non-faulted SG	CREW US US		- - -
Informs SM of unit status Examiner's Note: O If Secondary Radiation is indicated, the crew will transition to 1BEP-3. (Steps continued on page 15.) Transition to 1 BEP-1 "LOSS OF REACTOR OR SECONDARY COOLANT" US (Feedback) MSIVs closed 1AF013B and 1AF013F closed	 (E-2A) (000040AA1.10 4.1/4.1) AF valves closed (1AF013B/F) Main feedwater isolated 1B SG PORV closed (1MS018B) Blowdown isolation/sample valves closed (1SD002) 	2E/F, 1SD005C)		-
Examiner's Note: o If Secondary Radiation is indicated, the crew will transition to 1BEP-3. (Steps continued on page 15.) Transition to 1 BEP-1 "LOSS OF REACTOR OR SECONDARY COOLANT" US (Feedback) • MSIVs closed • 1AF013B and 1AF013F closed	Monitor AF pump suction pressure	U-1 AST		-
o If Secondary Radiation is indicated, the crew will transition to 1BEP-3. (Steps continued on page 15.) Transition to 1 BEP-1 "LOSS OF REACTOR OR SECONDARY COOLANT" US	• Informs SM of unit status	US		_
 MSIVs closed 1AF013B and 1AF013F closed 	o If Secondary Radiation is indicated, the crew will transit on page 15.)	ONDARY COOLANT"	ntinued	_
COMMENTS	 MSIVs closed 			
	COMMENTS			

EVENT <u>G</u>			
BRIEF DESCRIPTION Ruptured - Faulted SG – 1B SG			
EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT	
Recognize and diagnose/respond to a SGTR in accordance values. "LOSS OF REACTOR OR SECONDARY COOLANT" Of			
(Cue)			
o 1B SG radiation monitor trends rising o PZR pressure dropping			
o PZR pressure dropping o PZR level dropping			
o 1B SG pressure rising			
(Response)			
Announce indications of a SGTR	US/U-1AST		
Transition from 1BEP-1 " LOSS OF REACTOR OR SECONDARY COOLANT " to 1BEP-3 "SGTR"	US		
• Inform SM of unit status/potential EP event	US		
COMMENTS			

BRIEF DESCRIPTION Ruptured - Faulted SG – 1B SG

EXPECTED OPERATOR/PLANT RESPONSE	SAT	UNSAT	
N/A			
Recognize and diagnose/respond to a SGTR			
Transition to 1BEP-3 "SGTR"	US		
• Inform SM of unit status/potential EP event	US		
Coordinate/direct operator actions in accordance with 1BEP-3 "SGTR" to isolate a ruptured SG			
 RCPs running 	U-1 U-1/U-1AST		
• Identify ruptured SG (1B)	U-1/U-1AST		
• Isolate flow from ruptured SG	TI 1 A COD		
o 1B SG PORV in auto and closed (1MS018B) o 1B SG blowdown isolation valves closed	U-1AST U-1/U-1AST		
o 1B SG blowdown isolation valves closed (1SD002E & F)	U-1/U-1AS1		
o 1B MSIV and MSIV bypass valves closed (1MS001B and 1MS101B)	U-1AST		
o Check intact SG PORVs available	US/U-1AST		
for RCS cooldown (1MS018A, C, and D)	CS/C 11151		
o 1B SG NR level < 10% (do not feed 1B SG)	U-1AST		
 Aux Feedwater isolation valves 1AF013B & F closed 	U-1AST U-1AST		
(may have been previously isolated)			
EXAMINER'S NOTE: o If 1B SG pressure is > 320 psig, go to page 16. o If 1B SG pressure is < 320 psig, go to page 17.			
• Check ruptured SG pressure > 320 psig	U-1 AST		
COMMENTS			

BRIEF DESCRIPTION	Ruptured -	- Faulted SG -	- 1B SG
-------------------	------------	----------------	---------

EXPECTED OPERATOR/PLANT RESPONSE N/A	SAT	UNSAT	
 Initiate RCS cooldown Specify RCS temperature to which the RCS must be cooled down to allow depressurization of the RCS to ruptured SG pressure Dump steam at maximum rate via: Intact SG PORVs 	US/U-1AST		
 Crew performs steps 7 through 11 of 1BEP-3 to establish the following conditions: Check intact SG levels > 10%, control 18-50% Check PZR PORVs and isolation valves Reset SI Reset Containment Isolation Restore IA to Cnmt Verify AC buses energized by offsite power 	CREW		
 Check if RH pumps should be stopped RCS pressure > 325 psig, stop RH pumps Check if RCS cooldown should be stopped Maintain CETCs < required temperature Check ruptured SG pressure stable or rising 1B SG pressure within 250 psig of the intact SGs 	U-1AST U-1 AST/U-1 U-1AST	· ——	
COMMENTS			

BRIEF DESCRIPTION Ruptured - Faulted SG – 1B SG

EXPECTED OPERATOR/PLANT RESPONSE	SAT	UNSAT	
N/A			
December of the second of the			
Recognize symptoms and respond to a ruptured/faulted SG			
(Cue)			
o 1B SG pressure < 320 psig			
o 1B SG pressure within 250 psig of the intact SGs			
(Response)			
Announcement of faulted/ruptured SG symptoms	CREW		
Transition to 1BCA-3.1 "SGTR WITH LOSS OF	US		_
RX COOLANT - SUBCOOLED RECOVERY DESIRE			
• Direct operator actions of 1BCA-3.1 to establish the	US		
following conditions:			
o Reset SI	U-1AST		
o Reset Cnmt Isolation	U-1AST		
o Reset Phase A			
o IA restored to Cnmt			
 AC busses energized by offsite power 	U-1AST		
 Cnmt spray not running 	U-1AST U-1AST		
 Check ruptured SG level 	U-1AST		
 Verify 1AF013B and F are closed 			
o Check if RH pumps should be stopped	U-1AST		
• RCS pressure > 325 psig, stop RH pumps			
Plant status evaluated	CREW		
Aux. Bldg. radiation			
Place hydrogen monitors in service	. 1		
RCS boron and Cnmt recirc sump pH samples requ	ested		
Evaluate CC and RH systems			
o Prepare hydrogen recombiners• 1B SG identified as faulted	TI 1ACT		
 Check intact SG levels >10%, control 18-50% 	U-1AST U-1AST		
	US/U-1/U-1AST		
 RCS cooldown initiated at < 100°F/hr Do not initiate cooldown if cooldown rate 	US/U-1/U-1AS1		
is > 100°F in any 1 hour period			
is > 100 1° in any 1 nour period			
COMMENTS			