

August 5, 2009

Mr. Scott Head, Manager
Regulatory Affairs
STP Nuclear Operating Company
P. O. Box 289
Wadsworth, TX 77483

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION LETTER NO. 200 RELATED TO
SRP SECTION 09.03.03 FOR THE SOUTH TEXAS PROJECT COMBINED
LICENSE APPLICATION

Dear Mr. Head

By letter dated September 20, 2007, STP Nuclear Operating Company (STP) submitted for approval a combined license application pursuant to 10 CFR Part 52. The U. S. Nuclear Regulatory Commission (NRC) staff is performing a detailed review of this application to enable the staff to reach a conclusion on the safety of the proposed application.

The NRC staff has identified that additional information is needed to continue portions of the review. The staff's request for additional information (RAI) is contained in the enclosure to this letter.

To support the review schedule, you are requested to respond within 30 days of the date of this letter. If changes are needed to the safety analysis report, the staff requests that the RAI response include the proposed wording changes.

S. Head

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If you have any questions or comments concerning this matter, I can be reached at 301-415-8484 or by e-mail at Tom.Tai@nrc.gov or you may contact George Wunder at 301-415-1494 or George.Wunder@nrc.gov.

Sincerely,

/RA/

Tom M. Tai, Senior Project Manager
ABWR Projects Branch
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-012
52-013

eRAI Tracking No. 2554

Enclosure:
Request for Additional Information

cc: William Mookhoek
James Agles

S. Head

-2-

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NRO-002

OFFICE	CHPB/TR	CHPB/BC	NGE2/PM	OGC	NGE2/L-PM
NAME	SWilliams	TFrye	TTai	SKirkwood	GWunder
DATE	4/14/09	4/17/09	8/5/09	4/30/09	5/12/09

***Approval captured electronically in the electronic RAI system.**

OFFICIAL RECORD COPY

Request for Additional Information No. 2554 Revision 2

**South Texas Project Units 3 and 4
South Texas Project Nuclear Operating Co
Docket No. 52-012 and 52-013
SRP Section: 09.03.03 - Equipment and Floor Drainage System
Application Section: 10 CFR 20 and 10 CFR 50 Appendix I**

QUESTIONS for Health Physics Branch (CHPB)

09.03.03-5

9.3.3.2.3 states, "The non-radioactive drain system collects waste water from plant buildings (Reactor, Turbine, Control, Service, and other buildings). A system composed of collection piping, curbs, and pumps is provided. Non-radioactive waste water from the Turbine Building, Reactor Building, hot machine shop and the Control Building is routed to a dedicated oil/water separator where oil and settled solids are removed for off-site disposal. The non-oily, non-radioactive effluent is sent to dual settling basins. Nonradioactive waste water from the Service Building and other buildings is sent directly to the dual settling basins. Means are provided to perform any required tests or analyses required by the discharge permit. The non-radioactive liquid effluent is discharged to the Main Cooling Reservoir through permitted outfall(s). If radioactivity levels exceed the limits for discharge, the flow from the non-radioactive drains has the capability to be diverted to the radioactive effluent portion of the radwaste system. Normally, if low levels of radioactivity are detected, it is quantified and discharged via the normal outfall. Higher levels of radioactivity may require a permitted "batch" discharge via the radwaste effluent radiation monitor. The non-radioactive drainage system is illustrated in Figure 9.3-12."

This drain system has the potential to carry radioactive contamination to the environment. Please describe in detail how the applicant intends to comply with 10 CFR 20.1501, 10 CFR 50.34a, 10 CFR 50.36a, and GDC 60, 64 and 10 CFR 50 Appendix I for this system. This description should include:

- a.) How will radioactive effluent release quantities and criteria be determined?
- b.) How will non-radioactive waste water and radioactive waste water be segregated in these potentially contaminated areas, e.g., Hot Machine Shop, Reactor Building, etc., listed in the FSAR?
- c.) How is the discharge permit related to the Offsite Dose Calculation Manual (ODCM) when any levels of radioactivity are detected? How is the associated dose and quantities of radioactive material accounted for in accordance with 10 CFR 20 and 10 CFR 50 Appendix I?
- d.) How is the radiation monitor, shown on drawing 9.3-12, calibrated, and set points established? Is this a required radiation monitor?
- e.) What are the means to collect samples to perform any required tests or analyses for the discharge permit? How will samples for radioactive analyses be obtained for any effluent releases?
- f.) Please describe the analyses to be performed prior to releasing any radioactive materials to the environment. Are these analyses required?
- g.) How will radioactive "batch" discharges to the environment be performed?
- h.) What are the radioactivity levels that exceed the "limits for discharge"? What are the limits for discharge that are needed to divert the radioactivity levels to the radwaste system for treatment?

Enclosure