

#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

August 5, 2009

Mr. Charles G. Pardee President and Chief Nuclear Officer Exelon Nuclear 4300 Winfield Road Warrenville, IL 60555

SUBJECT: THREE MILE ISLAND, UNIT NO. 1 - INDIVIDUAL PLANT ACTIONS RE: PRESSURIZED-WATER REACTOR OWNERS GROUP TOPICAL REPORT BAW-2374, REVISION 2, "RISK-INFORMED ASSESSMENT OF ONCE-THROUGH STEAM GENERATOR TUBE THERMAL LOADS DUE TO BREAKS IN REACTOR COOLANT SYSTEM UPPER HOT LEG LARGE-BORE PIPING" (TAC NO. ME1797)

Dear Mr. Pardee:

On June 25, 2009, a meeting was held between representatives of the U.S. Nuclear Regulatory Commission (NRC) staff and the Pressurized-Water Reactor Owners Group regarding the proposed resolution of items related to BAW-2374, Revision 2, "Risk-informed Assessment of Once-Through Steam Generator Tube Thermal Loads due to Breaks in Reactor Coolant System Upper Hot Leg Large-Bore Piping." A summary of the meeting can be found at Agencywide Documents Access and Management System Accession No. ML091820001. During this meeting, representatives of Exelon Generating Company, LLC participated in a presentation relating to the status of this matter as it relates to Three Mile Island, Unit 1 (TMI-1).

The NRC staff requests that you provide a written response to the enclosed list of actions taken from the meeting summary of the June 25, 2009, meeting. This response should describe your plans to address these items and will enable the staff to evaluate the need for any further regulatory action. A response is requested within 30 days from the date of this letter.

Sincerely,

the Barroford

Peter Bamford, Project Manager Plant Licensing Branch I-2 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosure: Actions from June 25, 2009, Meeting

cc w/encl: Distribution via Listserv

# ACTIONS RESULTING FROM JUNE 25, 2009

# MEETING BETWEEN U.S. NUCLEAR REGULATORY COMMISSION

# AND PRESSURIZED-WATER REACTOR OWNERS GROUP

The U.S. Nuclear Regulatory Commission (NRC) requests that within 30 days from the date of this letter, Exelon Generation Company, LLC, the licensee for Three Mile Island, Unit 1, submit a letter providing plans to address the following items resulting from the June 25, 2009, public meeting regarding once-through steam generator (SG) tube loads under conditions resulting from postulated breaks in reactor coolant system (RCS) upper hot leg large bore piping:

- 1) Provide confirmation that its justification for continued operation for addressing tube integrity following a large break loss-of-coolant accident (LBLOCA) remains valid.
- Provide confirmation that compensatory measures, such as changes to emergency operating procedures, have been incorporated into plant procedures and operator training has been performed.
- 3) Provide confirmation that Title 10 of the *Code of Federal Regulations* paragraph 50.46(a)(3) reporting requirements have been satisfied.
- Provide confirmation that all LBLOCAs (including those in the candy-cane region) are considered as design-basis accidents in the assessments of SG tube integrity following each SG tube inspection.
- 5) Provide a commitment that an analysis will be performed to confirm that the design of the replacement SGs is sufficient to withstand the loads associated with a LBLOCA including the thermal loads associated with a LBLOCA in the candy-cane region of the RCS and to provide the results of that analysis to the NRC.
- 6) Provide a commitment to include the structural limit associated with the most limiting LBLOCA for the replacement SGs as part of the next SG tube inspection report (required by the technical specifications) following completion of the next inspection of the tubes in the replacement SGs, unless previously submitted.

Mr. Charles G. Pardee President and Chief Nuclear Officer Exelon Nuclear 4300 Winfield Road Warrenville, IL 60555

SUBJECT: THREE MILE ISLAND, UNIT NO. 1 - INDIVIDUAL PLANT ACTIONS RE: PRESSURIZED-WATER REACTOR OWNERS GROUP TOPICAL REPORT BAW-2374, REVISION 2, "RISK-INFORMED ASSESSMENT OF ONCE-THROUGH STEAM GENERATOR TUBE THERMAL LOADS DUE TO BREAKS IN REACTOR COOLANT SYSTEM UPPER HOT LEG LARGE-BORE PIPING" (TAC NO. ME1797)

Dear Mr. Pardee:

On June 25, 2009, a meeting was held between representatives of the U.S. Nuclear Regulatory Commission (NRC) staff and the Pressurized-Water Reactor Owners Group regarding the proposed resolution of items related to BAW-2374, Revision 2, "Risk-informed Assessment of Once-Through Steam Generator Tube Thermal Loads due to Breaks in Reactor Coolant System Upper Hot Leg Large-Bore Piping." A summary of the meeting can be found at Agencywide Documents Access and Management System Accession No. ML091820001. During this meeting, representatives of Exelon Generating Company, LLC participated in a presentation relating to the status of this matter as it relates to Three Mile Island, Unit 1 (TMI-1).

The NRC staff requests that you provide a written response to the enclosed list of actions taken from the meeting summary of the June 25, 2009, meeting. This response should describe your plans to address these items and will enable the staff to evaluate the need for any further regulatory action. A response is requested within 30 days from the date of this letter.

Sincerely,

/ra/

Peter Bamford, Project Manager Plant Licensing Branch I-2 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosure: Actions from June 25, 2009, Meeting cc w/encl: Distribution via Listserv <u>DISTRIBUTION</u>: PUBLIC LPL1-2 r/f RidsAcrsAcnw\_MailCTR Resource RidsNrrDorlLp11-2 Resource RidsNrrDorlDpr Resource RidsNrrDciCsgb Resource RidsNrrDraAadb Resource

RidsNrrPMPBamford Resource RidsNrrLAABaxter Resource RidsOgcRp Resource RidsRgn1MailCenter Resource RidsNrrDprPspb Resource

### ADAMS Accession No. ML092120053

OFFICE	DORL/LPL1-2/PM	DORL/LPL1-2/LA	DRA/AADB/BC	DPR/PSPB/BC	DCI/CSGB/BC	DORL/LPL1-2/BC
NAME	PBamford	ABaxter	RTaylor	SRosenberg	MGavrilas (w/ comments)	HChernoff
DATE	7/31/09	8/5/09	8/5/09	8/6/09	8/5/09	8/5/09