



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 31, 2009

Mr. Stewart B. Minahan
Vice President-Nuclear and CNO
Nebraska Public Power District
72676 648A Avenue
Brownville, NE 68321

SUBJECT: COOPER NUCLEAR STATION - REQUEST FOR ADDITIONAL INFORMATION
RE: RELIEF REQUEST NOS. RI-21 AND RI-22 (TAC NOS. ME0687 AND
ME0688)

Dear Mr. Minahan:

By application dated February 16, 2009, to the U.S. Nuclear Regulatory Commission (NRC) (Agencywide Documents Access and Management System (ADAMS) Accession No. ML090540420), Nebraska Public Power District submitted relief request nos. RI-21 and RI-22, in accordance with Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.55a. RI-21 requests to use an alternative examination volume defined in Code Case N-613-1, Figure 1, instead of the 100 percent volumetric examination of the reactor nozzle-to-vessel welds defined by the American Society of Mechanical Engineers *Boiler and Pressure Vessel Code* (ASME Code). RI-22 requests to perform examinations of accessible portions of the reactor nozzle-to-safe-end welds to the extent practical instead of the examinations required by the ASME Code.

The NRC staff has reviewed the information provided in your application and determined that additional information is required in order to complete its review. The enclosed questions were provided to Mr. D. Van Der Kamp of your staff on July 28, 2009. Please provide a response to the questions by August 31, 2009.

The NRC staff considers that timely responses to requests for additional information help ensure sufficient time is available for NRC staff review and contribute toward the NRC's goal of efficient and effective use of NRC staff resources. If circumstances result in the need to revise the requested response date, please contact me at (301) 415-2296.

Sincerely,

A handwritten signature in black ink that reads "Michael C. Markely for F. Lyon".

Carl F. Lyon, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosure:
Request for Additional Information

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION

RELIEF REQUEST NOS. RI-21 AND RI-22

COOPER NUCLEAR STATION

DOCKET NO. 50-298

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the information provided by Nebraska Public Power District (the licensee) for Cooper Nuclear Station in its letter dated February 16, 2009, and determined that additional information is necessary to complete the review of relief request nos. RI-21 and RI-22. Please provide a response which addresses the following questions.

RI-21

1. Provide a more detailed description of the physical interferences (i.e., limitations that make increasing ultrasonic examination coverage impractical.
2. Provide a more detailed description of the design changes necessary to obtain the required degree of non-destructive examination (NDE) coverage.
3. Explain how the proposed partial examination, alternative, or additional examinations provide reasonable assurance of the continued structural integrity of the components.
4. Provide a description of all inspections performed on the subject components during the fourth inspection interval, including volumetric, surface, and visual inspections. Provide all results of those examinations.
5. Describe how imposing the applicable regulatory requirement from which relief is being sought would be a burden on the licensee.

RI-22

1. Provide a more detailed description of the design changes necessary to obtain the required degree of NDE coverage.
2. Explain how the proposed partial examination, alternative, or additional examinations provide reasonable assurance of the continued structural integrity of the components.
3. Provide a description of all inspections performed on the subject components during the fourth inspection interval, including volumetric, surface, and visual inspections. Provide all results of those examinations.
4. The American Society of Mechanical Engineers (ASME) Code Section XI, 2001 Edition, 2003 Addenda, Table IWB-2500-1, Examination Category B-F, Item B5.10, requires 100 percent volumetric and a surface examination of the pressure retaining dissimilar metal welds in vessel nozzles as defined by Figure IWB-2500-8. Discuss the surface examination and provide the results of this examination.
5. Describe how imposing the applicable regulatory requirement from which relief is being sought would be a burden on the licensee.

Enclosure

July 31, 2009

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Sincerely,
/RA by Michael T. Markley for/

Carl F. Lyon, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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ADAMS Accession No.: ML092090065

*memo dated

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