UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 5, 2009

Vice President, Operations
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
450 Broadway, GSB
P.O. Box 249
Buchanan, NY 10511-0249

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 - CORRECTION TO TECHNICAL SPECIFICATIONS (TAC NOS. MA4359 AND MC5033)

Dear Sir or Madam:

The purpose of this letter is to issue the following typographical corrections to the Indian Point Nuclear Generating Unit No. 3 (IP3) Technical Specifications (TSs).

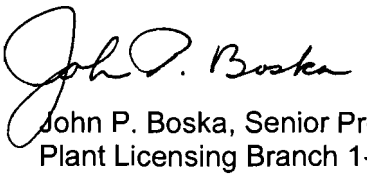
By letter dated April 6, 2005, Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML050680066 and ML050970205, the Nuclear Regulatory Commission (NRC) issued License Amendment No. 226 to Facility Operating License (FOL) No. DPR-64 for IP3. Although not a part of the amendment request, a typographical error was introduced on TS Page 3.5.3-1 by eliminating the word "testing" in the note for Limiting Condition for Operation 3.5.3.

Also, by letter dated April 14, 2005, ADAMS Accession Nos. ML050660112 and ML051080034, the NRC issued License Amendment No. 228 to FOL No. DPR-64 for IP3. The amendment revised the TSs by eliminating the requirements associated with hydrogen recombiners and hydrogen monitors. The marked-up TS pages correctly showed the change to remove footnote (d) from TS Table 3.3.3-1 by moving it to be footnote (c). However, there was a typographical error on the final TS Page 3.3.3-4 issued with the amendment, in which the Required Channels column for the Core Exit Thermocouples did not have footnote (d) replaced by footnote (c), as was approved by the amendment. There is currently no footnote (d) to this table, although it is still referenced on Page 3.3.3-4.

Please replace the corresponding pages from the TSs with the corrected TS pages 3.3.3-4 and 3.5.3-1 enclosed with this letter.

Please contact me at (301) 415-2901 if you have any questions on this issue.

Sincerely,



John P. Boska, Senior Project Manager
Plant Licensing Branch 1-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure:
Corrected TS Pages 3.3.3-4 and 3.5.3-1

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## 3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)

### 3.5.3 ECCS–Shutdown

LCO 3.5.3 One ECCS residual heat removal (RHR) subsystem and one ECCS recirculation subsystem shall be OPERABLE.

---NOTE---
An RHR train may be considered OPERABLE during alignment and operation for decay heat removal, and during valve testing, if capable of being manually realigned to the ECCS mode of operation.
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APPLICABILITY: MODE 4.

ACTIONS

---NOTE---
LCO 3.0.4.b is not applicable to the ECCS residual heat removal and ECCS recirculation subsystems.
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| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|---|---|---|
| A. Required ECCS residual heat removal (RHR) subsystem inoperable. | A.1 Initiate action to restore required ECCS RHR subsystem to OPERABLE status. | Immediately |
| B. Required ECCS Recirculation subsystem inoperable. | B.1 Restore required ECCS recirculation subsystem to OPERABLE status. | 1 hour |
| C. Required Action and associated Completion Time of Condition B not met. | C.1 Be in MODE 5. | 24 hours |

Table 3.3.3-1 (page 1 of 2)
Post Accident Monitoring Instrumentation

| | FUNCTION | REQUIRED CHANNELS | CONDITION REFERENCED FROM REQUIRED ACTION D.1 | SR 3.3.3.2 FREQUENCY |
|---|---|---|---|---|
| 1. | Neutron Flux | 2 | F | 24 months |
| 2. | RCS Hot Leg Temperature (Wide Range) ($T_{hot}$) | 1 per loop | E | 24 months |
| 3. | RCS Cold Leg Temperature (Wide Range) ($T_{cold}$) | 1 per loop | E | 24 months |
| 4. | RCS Pressure (Wide Range) | 2 | E | 24 months |
| 5. | Reactor Vessel Water Level | 2 | E | 24 months |
| 6. | Containment Water Level (Wide Range) | 2 | E | 24 months |
| 7. | Containment Water Level (Recirculation Sump) | 2 | E | 24 months |
| 8. | Containment Pressure | 2 | E | 18 months |
| 9. | Automatic Containment Isolation Valve Position | 2 per penetration flow path[(a)(b)] | F | 24 months |
| 10. | Containment Area Radiation (High Range) | 2 | F | 24 months |
| 11. | NOT USED | | | |
| 12. | Pressurizer Level | 2 | E | 24 months |
| 13. | SG Water Level (Narrow Range) | 2 per SG | E | 24 months |
| 14. | SG Water Level (Wide Range) and Auxiliary Feedwater Flow | 1 each per SG | E | 24 months, SGL<br>18 months, AFF |
| 15. | NOT USED | | | |
| 16. | Steam Generator Pressure | 2 per SG | E | 24 months |
| 17. | Condensate Storage Tank Level | 2 | F | 24 months |
| 18. | Core Exit Thermocouples-Quadrant 1 | 2 (c) | E | 24 months |
| 19. | Core Exit Thermocouples-Quadrant 2 | 2 (c) | E | 24 months |
| 20. | Core Exit Thermocouples-Quadrant 3 | 2 (c) | E | 24 months |
| 21. | Core Exit Thermocouples-Quadrant 4 | 2 (c) | E | 24 months |
| 22. | Main Steam Line Radiation | 1 per steam line | F | 24 months |
| 23. | Gross Failed Fuel Detector | 2 | F | 24 months |
| 24. | RCS Subcooling Margin | 2 | E | 24 months |

See NOTES, next page.

Please contact me at (301) 415-2901 if you have any questions on this issue.

Sincerely,

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John P. Boska, Senior Project Manager
Plant Licensing Branch 1-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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Enclosure:
Corrected TS Pages 3.3.3-4 and 3.5.3-1

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