



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 27, 2009

Mr. David A. Heacock
President and Chief Nuclear Officer
Dominion Nuclear Connecticut, Inc.
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: MILLSTONE POWER STATION, UNIT NO. 3 – REQUEST FOR ADDITIONAL
INFORMATION REGARDING RELIEF REQUEST IR-3-04, ALTERNATIVE
BRAZED JOINT METHODOLOGY (TAC NO. ME1256)

Dear Mr. Heacock:

By letter dated April 28, 2009 (Agencywide Document Access and Management System Accession No. ML091310666), Dominion Nuclear Connecticut, Inc. (DNC or the licensee) submitted Relief Request IR-3-04 for Millstone Power Station, Unit No. 3. The April 28, 2009, letter requested authorization to allow the deferral of repair/replacement activities for degraded brazed joints in the Class 3 Service Water System in lieu of the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, Section XI, 2004 Edition, No Addenda. To complete its review, the Nuclear Regulatory Commission staff requests the enclosed additional information.

The draft questions were sent to Mr. Bill Bartron, of your staff, to ensure that the questions were understandable, the regulatory basis for the questions was clear, and to determine if the information was previously docketed. On July 23, 2009, Mr. Mohamed Elmaghrabi, of your staff, agreed that you would provide a response by August 25, 2009.

If you have any questions regarding this matter, please contact me at 301-415-1603.

Sincerely,

A handwritten signature in black ink, appearing to read "Carleen J. Sanders".

Carleen J. Sanders, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosure:
As stated

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION (RAI)

RELIEF REQUEST IR-3-04

DOMINION NUCLEAR CONNECTICUT, INC.

MILLSTONE POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

By letter dated April 28, 2009 (Agencywide Document Access and Management System Accession (ADAMS) No. ML091310666), Dominion Nuclear Connecticut, Inc. (DNC or the licensee) submitted Relief Request IR-3-04 for Millstone Power Station, Unit No. 3. The April 28, 2009, letter requested authorization to allow the deferral of repair/replacement activities for degraded brazed joints in the Class 3 Service Water System in lieu of the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, (ASME Code) Section XI, 2004 Edition, No Addenda. To complete its review, the Nuclear Regulatory Commission (NRC) staff requests the following additional information.

1. Relief Request IR-3-04 contains many similarities in regards to the precedent relief request, IR-2-38 (ADAMS Accession No. ML051610101). It is unclear to the NRC staff why relevant information from the precedent relief request, contained in letters dated September 14, 2006 (ADAMS Accession No. ML062580234) and January 2, 2007 (ADAMS Accession No. ML070030092), was not incorporated into the current submittal. Please provide clarification as to why this information was not included and provide responses to the RAI's enclosed in the letter dated September 14, 2006, and the supplemental information from the January 2, 2007, letter, as applicable to Relief Request IR-3-04. If a question from the RAI is considered not applicable, please provide detailed justification as to why not.
2. In Section 5.3.4 of Relief Request IR-3-04, the licensee discusses the use of safety factors from ASME Code, Section XI, Appendix C paragraph C-3320(b) and C-3420(a), with these being the same safety factors permitted in Code Case N-513-1. Code Case N-513-1 has been superseded by Code Case N-513-2, as stated in Regulatory Guide 1.147, Revision 15. Please provide a discussion as to why the safety factors as listed in Code Case N-513-2 are not used.
3. Having employed an alternative to ASME Code requirements for the assessment of degraded brazed joints in the Class 3 service water system, via Relief Request IR-2-38, please provide a discussion on when this methodology was applied to degraded brazed joints during the last inservice inspection interval. Use an actual example from the plant and include the engineering evaluation used to assess the degraded joint; ultrasonic examination (UT) data gathered under MP-UT-45 Revision 000-01, Ultrasonic exam procedure in attachment E of the submittal, supporting the engineering evaluation; UT data from the monitoring of the degraded joint; UT data from augmented examinations; and work orders performing the code repair.

Enclosure

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Sincerely,
/RA/

Carleen J. Sanders, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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ADAMS Accession No. ML091940159

*By Memo Dated

Office	LPL1-2/PM	LPL1-2/LA	DCI/CPNB/BC	LPL1-2/BC
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Date	07/20/2009	07/15/2009	07/07/2009*	7/27/09

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