PMVogtleCOLPEm Resource

From: Joshi, Ravindra
Sent: Thursday, June 11, 2009 10:11 AM
To: Aughtman, Amy G.
Cc: VogtleCOL Resource
Subject: FW: Bellefonte chapter 4 SER with open items
Attachments: bellefonte ser chapter 4 cover letter.doc; bellefonte ser chapter 4.doc

For your info and use. Sorry for mix-up with the name AMY
Ravi

From: Joshi, Ravindra
Sent: Wednesday, June 10, 2009 4:38 PM
To: Wes Sparkman; Snyder, Amy
Cc: VogtleCOL Resource
Subject: FW: Bellefonte chapter 4 SER with open items

Attached is Bellefonte Chapter 4 SER with open items.

Sincerely

Ravi Joshi
Senior Project Manager
NRO/DNRL/NWE1
301-415-6191

From: Sebrosky, Joseph
Sent: Wednesday, June 10, 2009 4:10 PM
To: 'alsterdis@tva.gov'; 'Spink, Thomas E'
Cc: Nguyen, John-Chau; BelCol Resource; Wade, Tony; Joshi, Ravindra
Subject: Bellefonte chapter 4 SER with open items

Attached is Bellefonte chapter 4 SER with open items.

Sincerely,

Joe Sebrosky
Senior Project Manager
NRO/DNRL/NWE1
301-415-1132

Hearing Identifier: Vogtle_COL_Public
Email Number: 93

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Created By: Ravindra.Joshi@nrc.gov

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Tracking Status: None
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Tracking Status: None

Post Office: HQCLSTR01.nrc.gov

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Recipients Received:

June 10, 2009

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SUBJECT: BELLEFONTE UNITS 3 AND 4 SAFETY EVALUATION REPORT WITH OPEN ITEMS FOR CHAPTER 4, "REACTOR"

Dear Ms. Sterdis and Mr. Miller:

By letter dated October 30, 2007 (ML073110527), as supplemented by letters dated November 2, 2007 (ML073090248), January 8, 2008 (ML080100104), and January 14, 2008 (ML080160252), the Tennessee Valley Authority (TVA) submitted its application to the U.S. Nuclear Regulatory Commission (NRC) for a combined license (COL) for two AP1000 advanced passive pressurized water reactors in accordance with the requirements of 10 CFR 52, "Licenses, Certifications and Approvals for Nuclear Power Plants." The NRC formally docketed the application on January 18, 2008. These reactors are identified as Bellefonte (BLN) Units 3 and 4 and would be located near the town of Scottsboro in Jackson County, Alabama. The docket numbers established for Units 3 and 4 are 52-014 and 52-015, respectively. Subsequent to the original BLN COL application, TVA has updated and revised the application by letters dated October 10, 2008 (ML083100262), November 18, 2008 (ML083250490), and January 21, 2009 (ML090290406).

Based on our review of your application, the staff prepared the enclosed Safety Evaluation Report (SER) with Open Items for Chapter 4, "Reactor." Unless otherwise stated in the SER with Open Items, the staff's review was based on revisions and updates made to the BLN COL application through January 21, 2009. The SER with Open Items is being provided to support the upcoming meeting of the subcommittee of the Advisory Committee on Reactor Safeguards (ACRS) scheduled to be held on July 23-24, 2009. The ACRS Full Committee meeting will be held at a later date. Issuance of this SER is an important milestone in the staff's review to determine whether TVA's application meets the Commission's regulations.

In a letter dated April 28, 2009, the NuStart Energy Development, LLC, consortium informed the NRC that it had changed the Reference COL designation for the AP1000 Design Center from BLN Nuclear Plant, Units 3 and 4 to the Vogtle Electric Generating Plant (VEGP), Units 3 and 4. The NRC staff is taking steps necessary to implement this change. During this transition, the BLN Units 3 and 4 docket will continue to be the vehicle of standard content for the ACRS subcommittee reviews. It is the staff's understanding that Southern Nuclear Operating Company will be responsible for responding to open items related to standard content within a 45-day response period.

The staff concludes that the enclosed SER with Open Items does not contain any information for which exemption from public disclosure has been sought or approved. However, the NRC will withhold the enclosed SER from public disclosure for ten calendar days from the date of this letter to allow the opportunity to verify the staff's conclusion that the SER contains no such exempt information. If within that time, you do not request that all or portions of the SER be withheld from public disclosure in accordance with 10 CFR 2.90, "Public Inspections, Exemptions, Requests for Withholding," the enclosure will be made available for public inspection through the NRC's Public Document Room and the Publicly Available Records component of the NRC's Agencywide Documents Access and Management System (ADAMS). ADAMS is accessible from the Public Electronic Reading Room section of the NRC Web site at http://www.nrc.gov/reading-rm/adams.html.

Should you have any questions, please contact Mr. Ravindra Joshi, the NRC project manager at (301) 415-6191, ravindra.joshi@nrc.gov or Mr. Joseph Sebrosky, the lead project manager for the BLN COL application at (301) 415-1132, joseph.sebrosky@nrc.gov .

Sincerely,

/RA/

Stephanie Coffin, Branch Chief
AP1000 Projects Branch 1 (NWE1)
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-014
52-015
52-025
52-026

Enclosure:
As stated

cc w/ encl: see next page

In a letter dated April 28, 2009, the NuStart Energy Development, LLC, consortium informed the NRC that it had changed the Reference COL designation for the AP1000 Design Center from BLN Nuclear Plant, Units 3 and 4 to the Vogtle Electric Generating Plant (VEGP), Units 3 and 4. The NRC staff is taking steps necessary to implement this change. During this transition, the BLN Units 3 and 4 docket will continue to be the vehicle of standard content for the ACRS subcommittee reviews. It is the staff's understanding that Southern Nuclear Operating Company will be responsible for responding to open items related to standard content within a 45-day response period.

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Should you have any questions, please contact Mr. Ravindra Joshi, the NRC project manager at (301) 415-6191, ravindra.joshi@nrc.gov or Mr. Joseph Sebrosky, the lead project manager for the BLN COL application at (301) 415-1132, joseph.sebrosky@nrc.gov.

Sincerely,

/RA/

Stephanie Coffin, Branch Chief
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Office of New Reactors

Docket Nos. 52-014
52-015
52-025
52-026

Enclosure:
As stated

cc w/ encl: see next page

PKG: ML091330694
SER: ML091190476
Accession No.: ML091330732

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4.0 REACTOR

4.1 Introduction

This chapter describes the mechanical components of the AP1000 reactor and reactor core, including the reactor internals, control rod drive and core support structural materials, fuel system design (fuel rods and fuel assemblies), the nuclear design, and the thermal-hydraulic design. It also specifies the principal design criteria with which the mechanical design, the physical arrangement of the reactor components, and the capabilities of reactor control, protection, and emergency cooling systems (when applicable) must comply.

4.2 Summary of Application

Chapter 4 of the Bellefonte (BLN) Combined License (COL) Final Safety Analysis Report (FSAR), Revision 1, incorporates by reference Chapter 4 of the AP1000 Design Control Document (DCD), Revision 17.

In addition, in BLN COL FSAR Section 4.4, the applicant provided the following:

AP1000 COL Information Item

- STD COL 4.4-2

The applicant provided additional information in Standard (STD) COL 4.4-2 to address COL Information Item 4.4-2. This item states that, upon selection of the actual instrumentation, the COL holder shall calculate the instrumentation uncertainties of the operating parameters and confirm the validity of the design-limit departure from nucleate boiling ratio (DNBR) values.

License Condition

Part 10, License Condition 2, Item 4.4-2

The license condition will require the completion of the actions described in STD COL 4.4-2 prior to initial fuel load.

4.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is addressed within the Final Safety Evaluation Report (FSER) related to the DCD.

In addition, the relevant requirements of the Commission regulations for the thermal-hydraulic design and the associated acceptance criteria are given in SRP section 4.4 of NUREG-0800.

4.4 Technical Evaluation

The NRC Staff reviewed Chapter 4 of the BLN COL FSAR, Revision 1 and checked the referenced DCD to ensure that the combination of the DCD and the information in the COL

represent the complete scope of information relating to this review topic.[1] The NRC staff's review confirmed that the information contained in the application and incorporated by reference addresses the required information relating to the reactor internals, control rod drive and core support structural materials, fuel system design (fuel rods and fuel assemblies), the nuclear design, and the thermal-hydraulic design. Chapter 4 of the AP1000 DCD is being reviewed by the staff under Docket Number 52-006. The NRC staff's technical evaluation of the information incorporated by reference related to the reactor internals, control rod drive and core support structural materials, fuel system design (fuel rods and fuel assemblies), the nuclear design, and the thermal-hydraulic design will be documented in the staff SER on the DC application for the AP1000 Design.

The staff reviewed the information contained in the BLN COL FSAR:

AP1000 COL Information Item

- STD COL 4.4-2

The NRC staff reviewed STD COL 4.4-2 related to COL Information Item 4.4-2 and related COL Action Item 4.4-1 (from Appendix F of the NRC staff's FSER for the AP1000 DCD (NUREG-1793)), included under Section 4.4 of the BLN COL FSAR, Revision 1. STD COL 4.4-2 states:

> Following selection of the actual plant operating instrumentation and calculation of the instrumentation uncertainties of the operating plant parameters as discussed in DCD Subsection 7.1.6, the design limit DNBR values will be calculated. The calculations will be completed using the revised thermal design procedure (RTDP) with these instrumentation uncertainties and confirm that either the design limit DNBR values as described in DCD Section 4.4 remain valid or that the safety analysis minimum DNBR bounds the new design limit DNBR values plus DNBR penalties, such as rod bow penalty. This will be completed prior to fuel load.

License Condition

> Part 10, License Condition 2, Item 4.4-2

The applicant provided a license condition in Part 10 of the BLN COL application, "Proposed Combined License Conditions," which will require the completion of the actions described in STD COL 4.4-2 prior to initial fuel load.

As reported in FSER Section 4.4 related to the DCD, expected instrument uncertainties are included in the methodology used by the applicant in calculating the design limit DNBR values. The final validation of the design limit DNBR values will be based on the actual uncertainties for instrumentations not yet procured. The quantification of instrument uncertainties includes activities that require procurement and installation of the instruments, including evaluation of changes in sensor design and location, and that can only be completed after installation of the

[1] See Section 1.2.2 for a discussion of the staff's review related to verification of the scope of information to be included within a COL application that references a design certification.

instruments. Confirmation of instrument uncertainties after completion of the installation does not alter the methods of evaluation used to establish setpoints in the technical specifications, since the design limit DNBR values were based on the plant specifications for instrumentation uncertainties. The design limit DNBR values are expected to remain valid through plant procurement.

The NRC staff concluded in FSER Section 4.4 that the methodology for calculating the design limit DNBR values complied with the relevant regulatory requirements. The staff further concluded that it was acceptable to complete the final verification of the design limit DNBR values when the as-built specifications are available.

Therefore, the staff concludes that the supplemental information described in FSAR Section 4.4 meets COL Information Item 4.4-2 described in AP1000 DCD Subsection 4.4.7.2, complies with COL Action Item 4.4-1, and is acceptable.

The staff also finds the applicant's proposed license condition that will require completing this analysis prior to fuel load acceptable, since the applicant has committed to confirm that either the design limit DNBR values remain valid, or that the safety analysis minimum DNBR bounds the new design DNBR values plus DNBR penalties, such as rod bow penalty.

Conformance to Regulatory Guide 1.133, Revision 1

In BLN COL FSAR Section 1.9, "Compliance with Regulatory Criteria," Subsection 1.9.1, "Regulatory Guides," the applicant adds Appendix 1AA, which provides an evaluation of the degree of compliance with Division 1 regulatory guides as applicable to the content of this FSAR, or to the site-specific design, construction and/or operational aspects, and Table 1.9-201, which identifies the appropriate regulatory guide to FSAR cross-reference. In Appendix 1AA, the applicant provides an evaluation of its loose-part detection program for compliance with RG 1.133, Revision 1, May 1981, "Loose Part Detection Program for the Primary System of Light-Water-Cooled Reactors." It states that conformance of the design aspects is as stated in the DCD. It also documents conformance with the programmatic and/or operational aspects described in paragraphs C.3a and C.6 of RG 1.133, Revision 1.

RG 1.133, Revision 1, describes a method acceptable to the NRC staff for implementing regulatory requirements with respect to detecting a potentially safety-related loose part in light-water-cooled reactors during normal operation. The AP1000 design includes a digital metal impact monitoring system, which is a non-safety related system provided for monitoring the reactor coolant system for metallic loose parts. AP1000 DCD Subsection 4.4.6.4 documents the conformance of this monitoring system to RG 1.133. BLN COL FSAR Appendix 1AA documents its conformance to the design aspects described in DCD Subsection 4.4.6.4, and also states it conforms to Regulatory Position C.3a, regarding manual mode of data acquisition for detection of loose parts and Regulatory Position C.6, regarding notification to NRC of confirmation of the presence of a loose part.

The NRC staff noted that RG 1.133, Revision 1, was not included in Revision 0 of FSAR Table 1.9-201 for a cross-reference to the appropriate FSAR section, although an evaluation of compliance with RG 1.133 is provided in Appendix 1AA. In response to Request for Additional Information (RAI) 1-7, the applicant added RG 1.133, Revision 1, to Table 1.9-201, as part of Revision 1 to the FSAR. In addition, the response to RAI 1-7 was supplemented by adding a conformance discussion for regulatory guide positions related to the procedures and training program (positions 4g, 4h, 4i and 4j) in the proposed revision to BLN FSAR Appendix 1AA, "A

Conformance with Regulatory Guides." The proposed change to BLN FSAR is acceptable subject to a formal revision to BLN FSAR. Accordingly, this is **Confirmatory Item 4.4-1**. With the conformance of the programmatic and operational aspects of regulatory positions, the staff concludes that the applicant's loose parts detection program will conform to RG 1.133, Revision 1.

4.5 Post Combined License Activities

The following item was identified as the responsibility of the COL license holder:

> License Condition 2, Item 4.4-2, which requires that, prior to fuel load, the instrumentation uncertainties of the actual plant operating instrumentation will be calculated to confirm that either the design limit DNBR values remain valid or that the safety analysis minimum DNBR bounds the new design limit DNBR values plus DNBR penalties, such as rod bow penalty. The associated license condition requires this action be completed prior to fuel load.

4.6 Conclusion

The NRC staff reviewed the application and checked the referenced DCD. The NRC staff's review confirmed that the application addressed the required information relating to the reactor internals, control rod drive and core support structural materials, fuel system design (fuel rods and fuel assemblies), the nuclear design, and the thermal-hydraulic design and there is no outstanding information expected to be addressed in the BLN COL FSAR related to this chapter.

The Westinghouse application to amend Appendix D to 10 CFR Part 52 includes changes to Chapter 4 of the AP1000 DCD, as stated in Revision 17 of the AP1000 DCD. The staff is reviewing this information on Docket Number 52-006. The results of the NRC staff's technical evaluation of the information incorporated by reference in the BLN COL FSAR will be documented in a supplement to NUREG-1793. The supplement to NUREG-1793 is not yet complete, and this is being tracked as part of Open Item 1-1. The staff will update Chapter 4 of this SER to reflect the final disposition of the DC application.

In addition, the staff concludes that the relevant information presented within the COL FSAR is acceptable once Confirmatory Item 4.4-1, related to RG 1.133, is resolved. The staff based its conclusion on the following:

- STD COL 4.4-2 is acceptable because it specifies a commitment on the part of the applicant to confirm the validity of the calculations of the design limit DNBR values, which are based on the plant specifications for instrumentation uncertainties. The confirmation of plant instrument uncertainties will be completed when the as-built specifications are available. The methodology for this calculation was previously approved by the staff in NUREG-1793.

- The staff finds acceptable the applicant's proposed license condition that will require completing the analysis discussed in STD COL 4.4-2 prior to fuel load, since the applicant has committed to confirm that either the design limit DNBR values remain valid, or that the safety analysis minimum DNBR bounds the new design DNBR values plus DNBR penalties, such as rod bow penalty.

- The staff finds acceptable the applicant's proposed license condition that will require completing the analysis discussed in STD COL 4.4-2 prior to fuel load, since the applicant has committed to confirm that either the design limit DNBR values remain valid, or that the safety analysis minimum DNBR bounds the new design DNBR values plus DNBR penalties, such as rod bow penalty.

Accession No.: ML091190476

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