MITSUBISHI HEAVY INDUSTRIES, LTD.

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TOKYO, JAPAN

June 29, 2009

Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555-0001

Attention: Mr. Jeffrey A. Ciocco

Docket No. 52-021 MHI Ref: UAP-HF-09353

Subject: MHI's Response to US-APWR DCD RAI No. 393-2978

References: 1) "Request for Additional Information No. 393-2978 Revision 1, SRP Section: 05.04.02 .02– Steam Generator Program Application Section: 5.4" dated June 15, 2009.

With this letter, Mitsubishi Heavy Industries, Ltd. ("MHI") transmits to the U.S. Nuclear Regulatory Commission ("NRC") a document entitled "Responses to Request for Additional Information No. 393-2978 Revision 1".

Enclosure 1 provides the responses to the questions that are contained within Reference 1.

Please contact Dr. C. Keith Paulson, Senior Technical Manager, Mitsubishi Nuclear Energy Systems, Inc. if the NRC has questions concerning any aspect of the submittals. His contact information is below.

Sincerely,

y. Cynta

Yoshiki Ogata, General Manager- APWR Promoting Department Mitsubishi Heavy Industries, LTD. Enclosures:

1. Responses to Request for Additional Information No. Revision

CC: J. A. Ciocco C: K. Paulson

> DOB! NRD

Contact Information

C. Keith Paulson, Senior Technical Manager Mitsubishi Nuclear Energy Systems, Inc. 300 Oxford Drive, Suite 301 Monroeville, PA 15146 E-mail: ck_paulson@mnes-us.com Telephone: (412) 373-6466

Docket No. 52-021 MHI Ref: UAP-HF-09353

Enclosure 1

UAP-HF-09353 Docket No. 52-021

Responses to Request for Additional Information No. 393-2978 Revision 1

June 2009

RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION

6/30/2009

US-APWR Design Certification

Mitsubishi Heavy Industries

Docket No. 52-021

RAI NO.:	NO.393-2978 REVISION 1
SRP SECTION:	05.04.02.02 - STEAM GENERATOR PROGRAM
APPLICATION SECTION:	5.4
DATE OF RAI ISSUE:	6/15/2009

QUESTION NO.: 05.04.02.02-9

The response to RAI 05.04.02.02-8 (RAI 293-2173, dated March 30, 2009; MHI response dated April 17, 2009) described the size and shape of local degradation of steam generator tubes attributed to support structures (tube support plates and antivibration bars). For degradation at the tube support plates, the response stated that the length is 1.38 inches, which is equal to the plate thickness. Please also provide the circumferential extent of the degradation assumed in the analysis for the tube support

plate locations.

ANSWER:

The circumferential extent of the degradation is the fill circumference (360°) of the tube.

Impact on DCD

There is no impact on DCD.

Impact on COLA

There is no impact on COLA.

Impact on PRA

There is no impact on PRA.