



Entergy Operations, Inc.
1448 S.R. 333
Russellville, AR 72802
Tel 479-858-3110

Kevin T. Walsh
Vice President, Operations
Arkansas Nuclear One

2CAN060906

June 24, 2009

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

SUBJECT: License Amendment Request Supplemental
Technical Specification Change To Modify the Mode of Applicability for
Emergency Feedwater Actuation Signal
Arkansas Nuclear One, Unit 2
Docket No. 50-368
License No. NPF-6

REFERENCE 1: Entergy Letter to NRC dated March 2, 2009, "License Amendment Request
related to Technical Specification Change To Modify the Mode of
Applicability for Emergency Feedwater Actuation Signal" (2CAN030901)

Dear Sir or Madam:

In letter dated March 2, 2009 (Reference 1), Entergy Operations, Inc. (Entergy) proposed changes to the Arkansas Nuclear One, Unit 2 (ANO-2) Technical Specifications (TSs) associated with various Reactor Protective System (RPS) and Engineered Safeguard Features (ESF) setpoint modes of applicability.

Recently, the NRC notified ANO-2 Licensing that, while discussion of changes to TS Table 3.3-1 (TS Page 3/4 3-2) relating to Logarithmic (Log) Power were included in the aforementioned Entergy letter, there appeared to be no markup of the TS page in this regard. Therefore, a new markup page is included in attachment to this letter with the appropriate changes illustrated. Note that this page already illustrated changes to the Pressurizer Pressure – Low and Steam Generator Pressure – Low functions and are included in the attachment for consistency with the original letter (Reference 1). Therefore, the attached page may be used to replace the like page in the original letter (Reference 1).

As communicated above, changes to the Log Power function within TS Table 3.3-1 were discussed in the March 2, 2009, Entergy letter. However, to provide ease of review, a short discussion of the changes associated with this function is included below.

The "and *" under the Functional Unit 3.a column is deleted. The "*" refers to conditions when the reactor trip circuit breakers are closed and the control rods are, therefore, capable of withdrawal. This is redundant to the "*" next to the modes of applicability shown in the Applicable Modes column. Note that Mode 2 does not contain a "*" because Mode 2 (reactor critical) cannot be physically achieved with reactor trip circuit breakers open (power source to control rods). Therefore, it is not necessary for the "*" to be tied to Mode 2 conditions.

In addition, a "*" is added to the modes of applicability shown in the Applicable Modes column for Functional Unit 3.b. When reactor trip circuit breakers are open (control rods inserted), boron dilution monitors are used to monitor changes in reactivity. In addition, significant shutdown margin is maintained during shutdown operations by the elevated boron concentration of the Reactor Coolant System (RCS) up to the point an approach to criticality is to be initiated. Prior to initiating an approach to criticality, reactor trip circuit breakers are closed and control rod Shutdown Banks A and B are withdrawn ("cocked rod protection") as backup to the Boron Dilution Monitors. The proposed change to Functional Unit 3.b will ensure Log Power channels are operable for reactivity monitoring purposes any time cocked rod protection is established.

The proposed change does not result in a change to the current TS Action required to be performed if sufficient Log Power channels are not operable. In addition, the proposed change does not result in an impact to the original No Significant Hazards Determination contained in the March 2, 2009, Entergy letter (Reference 1).

The proposed change does not include any new commitment.

If you have any questions or require additional information, please contact David Bice at 479-858-5338.

I declare under penalty of perjury that the foregoing is true and correct. Executed on June 24, 2009.

Sincerely,

Original signed by Brad Berryman for Kevin Walsh

KTW/dbb

Attachment: Proposed Technical Specification Changes (mark-up)

cc: Mr. Elmo E. Collins
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
612 E. Lamar Blvd., Suite 400
Arlington, TX 76011-4125

NRC Senior Resident Inspector
Arkansas Nuclear One
P. O. Box 310
London, AR 72847

U. S. Nuclear Regulatory Commission
Attn: Mr. Kaly Kalyanam
MS O-8 B1
One White Flint North
11555 Rockville Pike
Rockville, MD 20852

Mr. Bernard R. Bevill
Arkansas Department of Health
Radiation Control Section
4815 West Markham Street
Slot #30
Little Rock, AR 72205

Attachment to

2CAN060906

Proposed Technical Specification Changes (markup)

TABLE 3.3-1

REACTOR PROTECTIVE INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
1. Manual Reactor Trip	2 sets of 2 2 sets of 2	1 set of 2 1 set of 2	2 sets of 2 2 sets of 2	1,2 3*,4*,5*	5 8
2. Linear Power Level – High	4	2	3	1,2	2,3
3. Logarithmic Power Level – High					
a. Startup and *	4	2(a)(d)	3	2,3*,4*,5*	2,3
b. Shutdown	4	0	2	3 *, 4 *, 5 *	4
4. Pressurizer Pressure – High	4	2	3	1,2	2,3
5. Pressurizer Pressure – Low	4	2(b)	3	1,2, 3 *, 4 *, 5 *	2,3
6. Containment Pressure – High	4	2	3	1,2	2,3
7. Steam Generator Pressure – Low	4/SG	2/SG	3/SG	1,2, 3 *, 4 *, 5 *	2,3
8. Steam Generator Level – Low	4/SG	2/SG	3/SG	1,2	2,3
9. Local Power Density – High	4	2(c)(d)	3	1,2	2,3