



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 26, 2009

Mr. Joseph N. Jensen
Senior Vice President and
Chief Nuclear Officer
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2 – CORRECTION
LETTER FOR LICENSE AMENDMENT NOS. 309 AND 291, REGARDING
TECHNICAL SPECIFICATION CHANGE RELATING TO DIESEL GENERATOR
STEADY-STATE PARAMETERS (TAC NOS. MD8773 AND MD8774)

Dear Mr. Jensen:

By letter dated April 30, 2009 (Agencywide Documents Access and Management System Accession No. ML090630245), the U.S. Nuclear Regulatory Commission (NRC) staff issued Amendment Nos. 309 and 291 to Renewed Facility Operating License Nos. DPR-58 and DPR-74 for the Donald C. Cook Nuclear Plant, Units 1 and 2, respectively. The amendments changed the Technical Specifications in response to your application dated June 27, 2007, as supplemented by letters dated April 28, September 4, and December 17, 2008.

The NRC staff recently identified administrative errors on Page 3 of the Renewed Facility Operating License for Unit 1 and Unit 2 associated with issuance of the aforementioned amendments.

For Unit 1, two lines (bold and italicized below, for emphasis) were inadvertently deleted from the bottom of the page in Section 2.C.(4). The partial paragraph should read as follows:

- (4) Indian Michigan Power Company shall implement and maintain, in effect, all Provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for the facility and as approved in the SERs dated ***December 12, 1977, July 31, 1979, January 30, 1981, February 7, 1983, November 22, 1983, December 23, 1983, March 16, 1984, August 27, 1985***

For Unit 2, the license condition information provided in the amendment was that for Unit 1.

The following corrected pages are provided as enclosures to this letter:

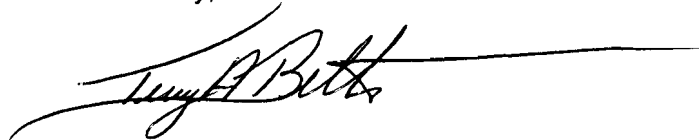
- Page 3, Amendment No. 309, for Renewed Facility Operating License No. DPR-58
- Page 3, Amendment No. 291, for Renewed Facility Operating License No. DPR-74

J. Jensen

- 2 -

The incorrect information was editorial. The corrections do not change the approval conveyed by the amendments, or the conclusions reached by the associated safety evaluation. We regret any inconvenience this error may have caused you.

Sincerely,

A handwritten signature in black ink, appearing to read "Terry A. Beltz", is written over a long horizontal line that extends across the page.

Terry A. Beltz, Senior Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures:
As stated

cc: Distribution via Listserv

and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components; and
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not to exceed 3304 megawatts thermal in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A and Appendix B, as revised through Amendment No. 309 are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Less than Four Loop Operation

The licensee shall not operate the reactor at power levels above P-7 (as defined in Table 3.3.1-1 of Specification 3.3.1 of Appendix A to this renewed operating license) with less than four reactor coolant loops in operation until (a) safety analyses for less than four loop operation have been submitted, and (b) approval for less than four loop operation at power levels above P-7 has been granted by the Commission by amendment of this license.

- (4) Indiana Michigan Power Company shall implement and maintain, in effect, all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for the facility and as approved in the SERs dated December 12, 1977, July 31, 1979, January 30, 1981, February 7, 1983, November 22, 1983, December 23, 1983, March 16, 1984, August 27, 1985,

radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components; and
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not to exceed 3468 megawatts thermal in accordance with the conditions specified herein and in Attachment 1 to the renewed operating license. The preoperational tests, startup tests and other items identified in Attachment 1 to this renewed operating license shall be completed. Attachment 1 is an integral part of this renewed operating license.

(2) Technical Specifications

The Technical Specifications contained in Appendix A and Appendix B, as revised through Amendment No. 291 are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Additional Conditions

(a) Deleted by Amendment No. 76

(b) Deleted by Amendment No. 2

(c) Leak Testing of Emergency Core Cooling System Valves

Indiana Michigan Power Company shall prior to completion of the first inservice testing interval leak test each of the two valves in series in the

June 26, 2009

J. Jensen

- 2 -

The incorrect information was editorial. The corrections do not change the approval conveyed by the amendments, or the conclusions reached by the associated safety evaluation. We regret any inconvenience this error may have caused you.

Sincerely,

/RA/

Terry A. Beltz, Senior Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures:
As stated

cc: Distribution via Listserv

DISTRIBUTION:

Public

LPL3-1 R/F

RidsNrrDorLpl3-1 Resource

RidsNrrLABTully Resource

RidsNrrPMTBeltz Resource

RidsAcrcsAcnw_MailCTR Resource

RidsRgn3MailCenter Resource

RidsOgcRp Resource

RidsNrrDirsltsb Resource

ADAMS Accession No. **ML091750036**

OFFICE	NRR/LPL3-1/PM	NRR/LPL3-1/LA	NRR/LPL3-1/BC
NAME	TBeltz	BTully	LJames
DATE	06/26/09	06/24/09	06/26/09

OFFICIAL RECORD COPY