

Reprocessing in the U.S.: Just Say No

Edwin S. Lyman
Senior Staff Scientist
Union of Concerned Scientists

Presentation at the U.S. NRC
Fuel Cycle Information Exchange
Rockville, MD, June 25, 2009

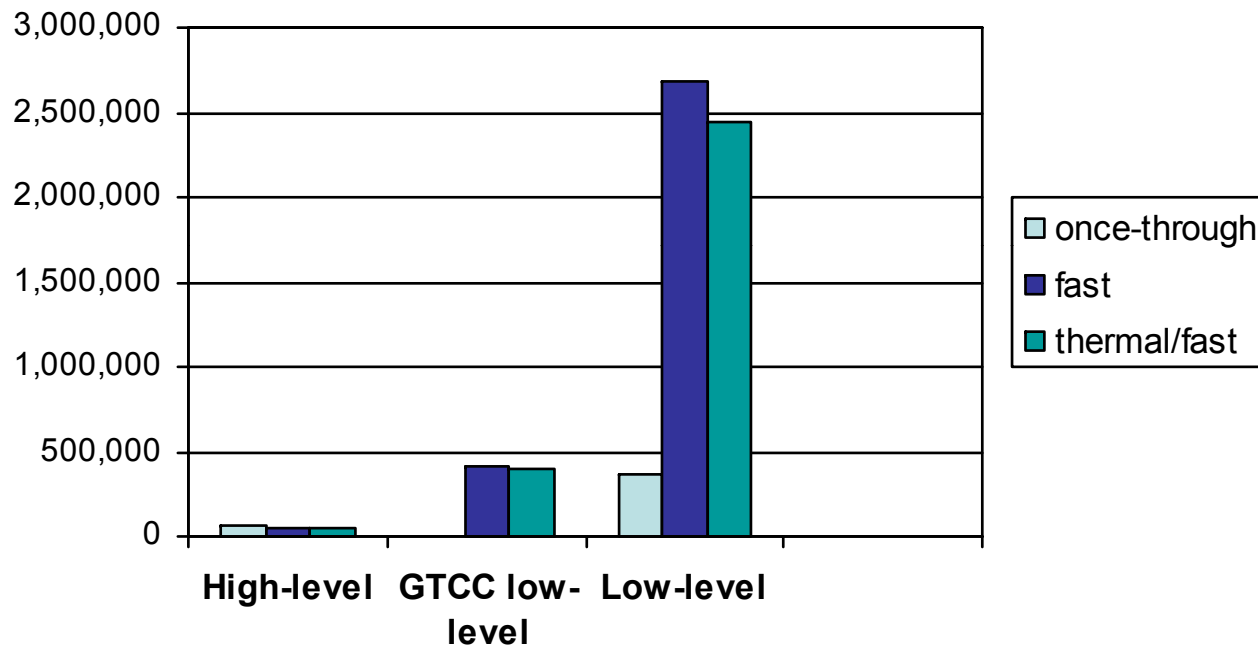
CONTEXT

- 2006: Global Nuclear Energy Partnership (GNEP)
 - Near-term, government-financed construction of commercial-scale fuel cycle facilities with reprocessing and fuel fabrication capabilities, as well as reactors to utilize reprocessed fuel
- Congress: scales back GNEP to R&D only
- April 2008: AREVA letter expressing interest in building a “used fuel recycling center” in the United States
- 2009: Redirection:
 - Obama Administration cancels GNEP program and NEPA process; refocuses on long-term, “science-based” R&D
 - But various approaches to facilitate near-term reprocessing are attempted in Congress, spurred on by lobbying by industry and national labs

REPROCESSING: DIRTY, DANGEROUS AND EXPENSIVE

- UCS opposes any approach that would move the US away from a strategy of direct disposal of spent fuel
- Reprocessing is
 - **Dirty**: according to Argonne National Laboratory data, for the fast reactor “recycle” option with Cs/Sr removal, after 50 years
 - Cumulative volume of all waste 7 times that of direct disposal option
 - Cumulative volume of greater-than-class C low-level waste is about 160 times greater than that of direct disposal option
 - Volume of reprocessed uranium comparable to volume of spent fuel
 - High-level waste volume only 25% less than initial spent fuel volume
 - **Dangerous**: can separate thousands of bombs’ worth of strategic special nuclear material per year; so-called “proliferation-resistant” modifications of reprocessing have been shown to be essentially worthless
 - **Expensive**: Argonne estimates reprocessing alone could cost 4-6 mils per kWh per year, or about \$4 billion per year to reprocess the current U.S. spent fuel output; DOE estimated that a reprocessing and “recycle” system would cost hundreds of billions more than direct disposal

50-YR CUMULATIVE WASTE GENERATION

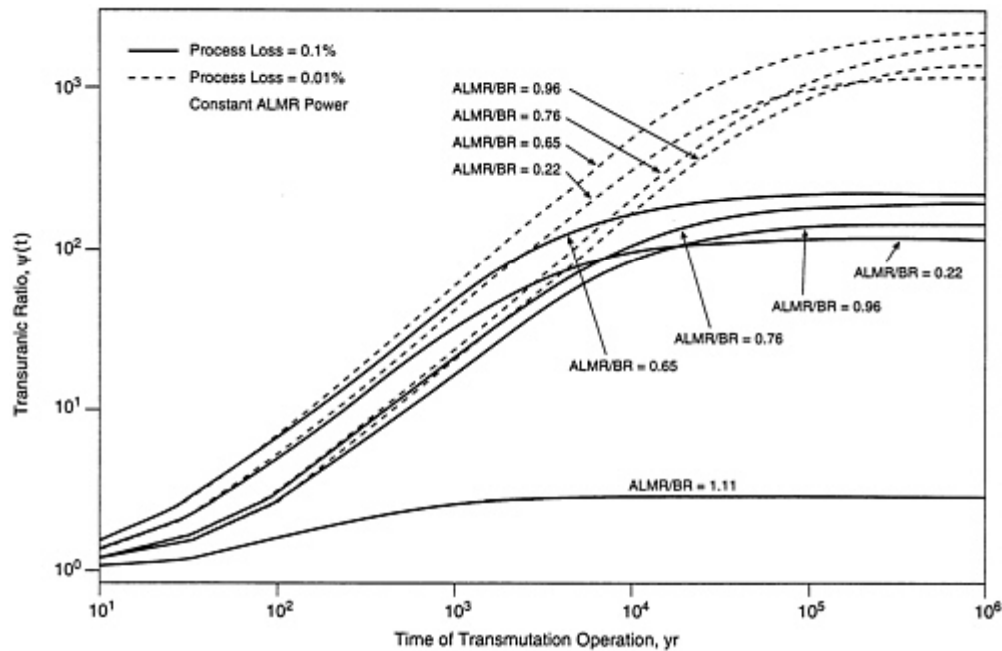


WHAT ABOUT THE TRANSURANICS (TRUs)?

- Inventories of separated plutonium and other transuranic elements (TRUs) must also be regarded as waste if they cannot be effectively fissioned in reactors
- In 1996, the National Academy of Sciences assessed the feasibility of “separations and transmutation systems” to significantly reduce the quantity of TRUs in the nuclear fuel cycle relative to the once-through cycle
 - $\psi(t)$ = “TRU ratio”
= ratio of once-through TRU inventory at time t to total TRU inventory in transmutation system (transmuter, fuel fabrication plants, spent fuel storage, process wastes)
- Report calculated $\psi(t)$ for a variety of transmutation scenarios, as a function of breeding ratio (BR) and process loss to waste

TRU INVENTORY REDUCTIONS (from NAS STATS Report, 1996)

- Constant power scenario



“RECYCLING”: TOO EXPENSIVE AND TIME-CONSUMING

- NAS conclusions:
 - A fast reactor system with 0.65 breeding ratio [0.5 TRU conversion ratio] to reprocess 62,000 t of LWR spent fuel and transmute the resulting TRUs under a declining nuclear power scenario would cost some \$500 billion and require approximately 150 years to accomplish the transmutation
 - “Merely developing, building and operating the individual components of the system would give little or no benefit. To have a real effect, an entire system of many facilities would be needed in which all the components operate with high reliability in a synchronized fashion for many decades or centuries ... the magnitude of the concerted effort and the institutional complexity ... are comparable to large military initiatives that endure for much shorter periods than would be required ...”
- UCS conclusion: reprocessing and transmutation schemes are not consistent with the intergenerational equity principle

DOE GNEP ECONOMIC ASSESSMENT

- A report by a DOE economist confirms the NAS picture (Matthew Crozat, “Evaluating the Economics of GNEP Deployment, January 8, 2007)
 - Assumptions:
 - DOE growth scenario (6-fold increase in nuclear power by 2100)
 - TRU conversion ratio of 0.5
 - Results:
 - “49% reduction in the mass of transuranics at year 100”
 - “over a third of the transuranics are tied up in the ... fuel cycle”
 - Additional cost of \$750 billion relative to direct disposal
- If operation of the system cannot be assured after 100 years, then the residual transuranics should be regarded as waste
 - Would require 6 Yucca Mountain-type repositories

REPROCESSING DOESN'T NEED A HELPING HAND

- New reprocessing regulations
 - should ensure that no reprocessing plant is built in the US unless there is sufficient protection against its exceptionally high safety, security and proliferation risks
 - should not facilitate licensing of inadequately safe and secure reprocessing technologies just because the industry can't do better

“GAP ANALYSIS”

- In SECY-09-0082, NRC has identified a number of high priority issues related to regulation of reprocessing, including
 - Definition of “reprocessing” and “recycling”
 - “Risk-informing” physical protection and material control and accounting (MC&A) rules for reprocessing plants...but NOT addressing the risks of other weapon-usable materials (Am, Np)

“RECYCLING” vs. REPROCESSING

- Reprocessing alone is NOT “recycling”
 - Generates separated plutonium and reprocessed uranium, most of which is not re-used today
- Applicants seeking a license for a “recycling” facility
 - Should be required to demonstrate that there are committed customers for all SNM that is separated
 - should have strict limits imposed on the amount of separated SNM that can be stored on site
- Will avoid large decommissioning liabilities associated with disposition of unwanted plutonium and reprocessed uranium
- Lack of “waste confidence” for all reprocessing effluents

“RISK-INFORMING” CAT I MPC&A STANDARDS?

- Gap analysis: reprocessing plants should be classified as Cat I facilities ...
 - But Category I standards themselves should be weakened because they “may pose an undue regulatory burden for reprocessing facility licensees”
- There is no technical basis for a weakening of Cat I standards for any reprocessing technology presently identified

“PROLIFERATION RESISTANCE” DOESN’T EXIST

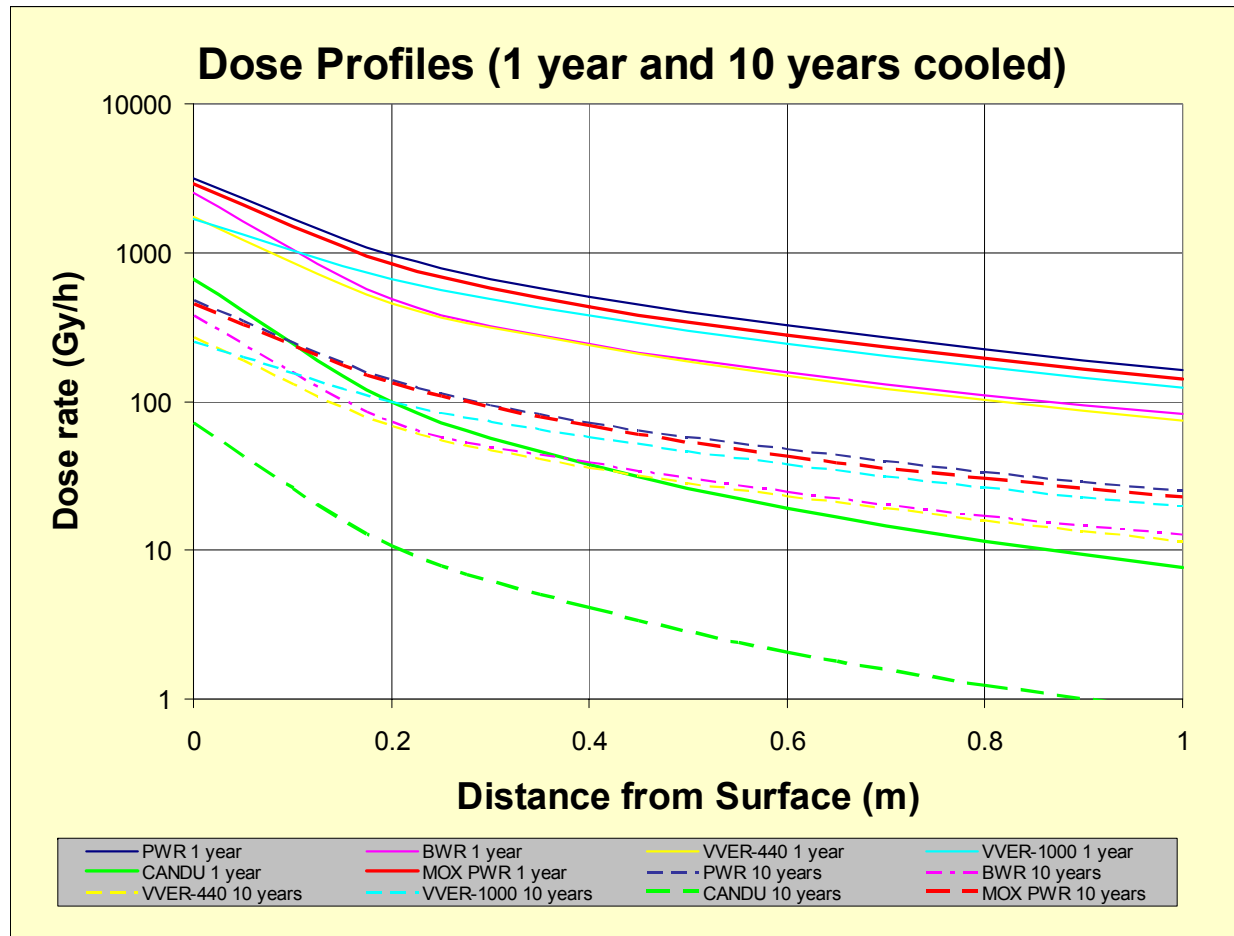
- Some argue that modified reprocessing techniques that do not separate “pure plutonium” such as COEX (50 Pu – 50 U) or pyroprocessing (Pu + U minor actinides + certain lanthanides) are “proliferation-resistant” and therefore do not need to be protected as Category I
- However, the December 2008 NNSA GNEP Nonproliferation Impact Assessment has determined that all of these alternatives “present comparable proliferation risks” and are considered to be Category I nuclear material under current DOE and NRC guidelines
- This determination should not to change even if NRC adopts an SNM categorization table and grades materials according to “attractiveness”
 - Pu will have to be diluted in U well below the 50-50 COEX mixture to reduce its attractiveness level
 - Np and Am must be considered strategic SNM

SELF-PROTECTION OF PYROPROCESSING

Electro-refining: The Best for Proliferation Resistance

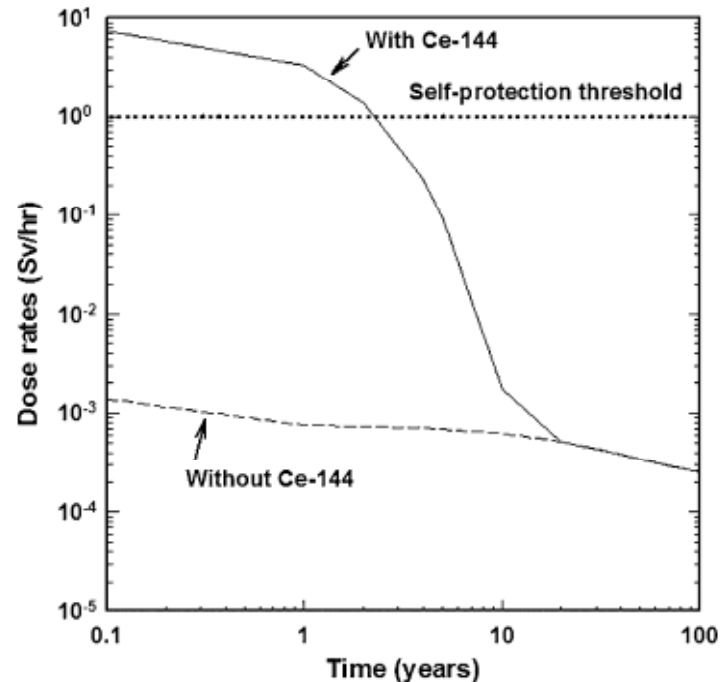
	Weapon Grade Pu	Reactor Grade Pu	Electro-refining
Production	Low burnup PUREX	High burnup PUREX	✓ Fast reactor Pyroprocess
Composition	Pure Pu 94% Pu-239	Pure Pu 65% Pu-fissile	✓ Pu + MA + U 50% Pu-fissile
Thermal power (w/kg)	2 – 3	5 – 10	✓ 80 – 100
Spontaneous neutrons (n/s/g)	60	200	✓ 300,000
Gamma radiation (r/hr at ½ m)	0.2	0.2	✓ 200

DOSE RATES AS A FUNCTION OF DISTANCE



DOSE RATES

Kang and von Hippel



Source: J. Kang and F. von Hippel, "Limited Proliferation-Resistance Benefits from Recycling Unseparated Transuranics and Lanthanides from Light-Water Reactor Spent Fuel," *Science and Global Security* **13** (2005)

ASSESSING SELF-PROTECTION

- Pyroprocessing would have a dose rate greater than 100 rem/hr at 1 meter only for a short period of time, after which it would disappear rapidly
- Mixing Pu with U only (COEX) has no appreciable impact on dose rate
- Conclusion: these are all minor modifications of conventional PUREX that would have no significant impact on the ability of skilled adversaries to steal weapon-usable materials and process them to produce weapons

NUCLEAR WEAPON USABILITY

- Plutonium-uranium mixtures are directly weapon-usable for a wide range of plutonium contents
 - COEX mixture is equivalent to 70 to 80%-enriched HEU oxide
- In addition, “the separation of ... uranium and plutonium involves an additional chemical process that is known to most fuel chemistry experts.” --
- T.A. Taiwo et al., “Co-Extraction Impacts on LWR and Fast Reactor Fuel Cycles,” Argonne National Laboratory, ANL-AFCI-187, May 31, 2007.
- Unirradiated Pu-U fuel:
 - plutonium content > 12% correspond to >20% HEU in bare crit mass
 - Categorized by IAEA as direct-use material with respect to safeguards and Category I with respect to physical protection
 - NRC has downgraded of Pu-U fuel with less than 20% plutonium content, based on difficulty of stealing fuel assemblies --- but assemblies can be quickly disassembled using shaped charges
- TRU
 - DOE Manual 470.4-6 (Chg. 1): separated neptunium and separated americium must be “protected, controlled and accounted for as if they were SNM.” [specifically, U-235]

NATIONAL LAB STUDY

	Attractiveness Level	FOM ^a
WEAPONS Assembled weapons and test devices	A	
PURE PRODUCTS Pits, major components, button ingots, recastable metal, directly convertible materials	B	> 2
HIGH-GRADE MATERIALS Carbides, oxides, nitrates, solutions (≥ 25 g/L) etc.; fuel elements and assemblies; alloys and mixtures; UF ₄ or UF ₆ ($\geq 50\%$ enriched)	C	1-2
LOW-GRADE MATERIALS Solutions (1 to 25 g/L), process residues requiring extensive reprocessing; moderately irradiated material; Pu-238 (except waste); UF ₄ or UF ₆ ($\geq 20\%$ < 50% enriched)	D	0-1
ALL OTHER MATERIALS Highly irradiated forms, solutions (<1 g/L), uranium containing <20% U-235 or <10% U-233 (any form, any quantity)	E	< 0 ^b

NATIONAL LAB STUDY: RESULTS

- Attractiveness levels of Pu and Pu+Np mixtures (UREX+2,+3,+4) are virtually identical
- For Pu+U metal mixtures (COEX)
 - $U \leq 20\%$, attractiveness level B (same as plutonium metal)
 - $20\% < U \leq 82\%$, attractiveness level C (same as plutonium oxide)
- For Pu+TRU metal mixtures (UREX+1a, PYROX),
 - Attractiveness level C unless diluted by $> 75\%$ U
- Overall, the study confirms that there is little reduction in attractiveness associated with UREX+, COEX or PYROX compared to PUREX

PHYSICAL PROTECTION ISSUES

- NRC Category I theft design basis threat and related security requirements become applicable for facilities possessing Category I quantities of plutonium or HEU
 - Separated Np, Am and Cm not addressed
- NRC and DOE design basis threats are different
 - Makes life difficult for NRC-licensed facilities on DOE sites (e.g. MFFF)
- NRC regulations generally do not give credit for dilution or other material properties that may reduce “attractiveness” of strategic special nuclear material
- But in 2005 NRC granted an exemption from Category I requirements to Duke Energy for possession of plutonium at a power reactor in the form of LWR mixed-oxide (Pu and U) fuel assemblies with Pu < 10 wt%
- Revision of NRC’s power reactor security regulations (§ 73.55) raised the Cat I exemption threshold to 20% wt% Pu
 - NRC’s gift to fast reactors? (driver fuel would likely be exempt)

PHYSICAL PROTECTION ISSUES (cont.)

- But no compelling technical basis exists for a relaxation of security standards for any of these materials compared to separated plutonium
- NRC should retain its attractiveness-neutral regulations
- There should be a stricter standard for exemptions from Category I security requirements than is currently on the books
 - Applicants seeks an exemption from Category I security requirements on the basis of material unattractiveness need to demonstrate that the same level of protection against the Cat I DBT would be maintained

CAT I MPC&A STANDARDS

- Gap analysis: reprocessing plants should be classified as Cat I facilities ...
- ... But the Cat I MC&A standards themselves are inadequate
 - Semi-annual vs. bi-monthly inventories
 - Is process monitoring technology sufficiently advanced to the point needed to detect abrupt losses?
 - Consistency with DOE standard
 - Throughput-dependent limit on the standard error of the inventory difference (SEID) (0.1% of active inventory)
 - Is this appropriate for very large reprocessing plants? (SEID could be on the order of 20-30 kg Pu, or 10-15 formula quantities, per year)
 - Can the limit be met even if it is not tightened?
 - Collusion threat restricted to two individuals
 - Impacts on MC&A of other weapon-usable special nuclear materials (Np, etc)

INRA PROPOSALS: THE WRONG DIRECTION

- The International Recycling Alliance (INRA) presented a paper in July 2008 outlining its vision for safeguards and security requirements at U.S. “recycling” plants: these proposals would weaken standards to allow licensing of inadequately secure facilities
 - INRA proposes *decreasing* the frequency of physical inventories for reprocessing plants from every 9 months to every year, instead of increasing it for consistency with other NRC Category I facilities
 - Because the NRC SEID limit of 0.1% of active inventory is “not attainable in a typical reprocessing plant,” INRA proposes raising the bar by replacing regulatory caps with license conditions based on actual plant measurement errors (which are at least ten times higher)

CONCLUSIONS

- NRC needs to revise its regulations for facilities handling weapon-usable materials to ensure that all SNM will be appropriately protected against the threats that our country faces
- The burden should be on applicants to demonstrate that they can account for and protect these materials to the highest standard before construction begins