

Sarah Rich

From: Richard Conte, *RI*
Sent: Sunday, January 11, 2009 6:59 PM
To: Richard Conte
Subject: cover guts
Attachments: OC 2008-07 LRI_rev-2A Upfront Guts.doc

Information in this record was deleted in
accordance with the Freedom of Information Act.
Exemptions 5
FOIA/PA 2009-0070

J/31

Received: from R1CLSTR01.nrc.gov ([148.184.99.7]) by R1MS01.nrc.gov
([148.184.99.10]) with mapi; Sun, 11 Jan 2009 18:58:53 -0500
Content-Type: application/ms-tnef; name="winmail.dat"
Content-Transfer-Encoding: binary
From: Richard Conte <Richard.Conte@nrc.gov>
To: Richard Conte <Richard.Conte@nrc.gov>
Date: Sun, 11 Jan 2009 18:58:52 -0500
Subject: cover guts
Thread-Topic: cover guts
Thread-Index: Acl0SI4kiOvOe+YORzWK0Zodj29ZKA==
Message-ID:
<2856BC46F6A308418F033D973BB0EE72AA60FC7615@R1CLSTR01.nrc.gov>
Accept-Language: en-US
Content-Language: en-US
X-MS-Has-Attach: yes
X-MS-Exchange-Organization-SCL: -1
X-MS-TNEF-Correlator:
<2856BC46F6A308418F033D973BB0EE72AA60FC7615@R1CLSTR01.nrc.gov>
MIME-Version: 1.0

Conte Input for Upfront Guts of Report

Dear Mr. Pardee

On December 23, 2008, the U. S. Nuclear Regulatory Commission (NRC) completed an inspection at your Oyster Creek Generating Station. The enclosed report documents the inspection results, which were discussed on December 23, 2008, with Mr. T. Rausch, Site Vice President, Mr. M. Gallagher, Vice President License Renewal, and other members of your staff in a telephone conference observed by representatives from the State of New Jersey.

Part 50 results

With respect to activities authorized by 10 CFR 50, the inspectors found no findings of safety significance.

(b)(5)

Rx5

Further, as you well know, an appeal of a licensing board decision regarding the Oyster Creek application for a renewed license is pending before the Commission related to the adequacy of the aging management program for the Oyster Creek drywell.

The midcycle letter of Indicated that the inspections of

(b)(5)

Rx5

The NRC is conducting these inspections using the guidance of Inspection Procedure (IP) 71003 "Post-Approval Site Inspection for License Renewal" as a prudent measure in order to take the opportunity to make observations of Oyster Creek license renewal activities during the last refuel outage prior to entering the period of extended operation. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

(b)(5)

Rx5

A-3

(b)(5)

5/25

DRAFT

(b)(5)

Res

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web-site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

(b)(5)

Res

Sincerely,

Darrell Roberts, Director
Division of Reactor Safety

DI

REPORT DETAILS

4. OTHER ACTIVITIES (OA)

4OA2 License Renewal Follow-up (IP 71003)

1. Background

Because the application for a renewed license remains under Commission review for final decision, and a renewed license has not been approved for Oyster Creek, the standards used to judge the adequacy of selected IP 71003 inspection samples do not apply.

This inspection was conducted in order to observe AmerGen's continuing license renewal activities during the last refueling outage prior to Oyster Creek (OC) entering the extended period of operation.

Accordingly, the inspectors recorded observations, without any assessment of implementation adequacy or safety significance.

IP 71003 verifies license conditions added as part of a renewed license, license renewal commitments, selected aging management programs, and license renewal commitments revised after the renewed license was granted, are implemented in accordance with Title 10 of the Code of Federal Regulations (CFR) Part 54, "Requirements for the Renewal of Operating Licenses for Nuclear Power Plants."

2. Inspection Sample Selection Process

The inspection team selected a number of inspection samples for review, using the NRC accepted guidance based on their importance in the license renewal application process, as an opportunity to make observations on license renewal activities.

Inspection observations were considered, in light of pending 10 CFR 54 license renewal commitments and license conditions, as documented in NUREG-1875, "Safety Evaluation Report (SER) Related to the License Renewal of Oyster Creek Generating Station," as well as programmatic performance under on-going implementation of 10 CFR 50 current licensing basis (CLB) requirements.

The reviewed SER proposed commitments and license conditions were selected based on several attributes including: the risk significance using insights gained from sources such as the NRC's "Significance Determination Process Risk Informed Inspection Notebooks," revision 2; the extent and results of previous license renewal audits and inspections of aging management programs; the extent or complexity of a commitment; and the extent that baseline inspection programs will inspect a system, structure, or component (SSC), or commodity group.

For each commitment and on a sampling basis, the inspectors reviewed supporting

documents including completed surveillances, conducted interviews, performed visual inspection of structures and components including those not accessible during power operation, and observed selected activities described below. The inspectors also reviewed selected corrective actions taken as a consequence of previous license renewal inspections.

DRAFT

DRAFT

3. Review Details

3.1 NRC Unresolved Item

10 CFR 50 existing requirements (e.g., current licensing basis (CLB))

xxx USE words from PN

- The conclusions of PNO-1-08-012 remain unchanged

- An Unresolved Item (URI) will be opened to evaluate whether existing current licensing basis commitments were adequately performed and, if necessary, assess the safety significance for any related performance deficiency.

- The issues for follow-up include the strippable coating de-lamination, reactor cavity trough drain monitoring, and sand bed drain monitoring.

- The commitment tracking, implementation, and work control processes will be reviewed, based on corrective actions resulting from AmerGen's review of deficiencies and operating experience, as a Part 50 activity.

3.2 Drywell Floor Trench Inspections

DRAFT