



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

August 4, 2009

Mr. John F. McCann  
Director, Nuclear Safety and Licensing  
Entergy Operations, Inc.  
Entergy Nuclear Operations, Inc.  
440 Hamilton Avenue  
White Plains, NY 10601

SUBJECT: ARKANSAS NUCLEAR ONE, UNITS 1 AND 2; JAMES A. FITZPATRICK NUCLEAR POWER PLANT; GRAND GULF NUCLEAR STATION, UNIT 1; INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3; PALISADES NUCLEAR PLANT; PILGRIM NUCLEAR POWER STATION; RIVER BEND STATION, UNIT 1; AND WATERFORD STEAM ELECTRIC STATION, UNIT 3 - ISSUANCE OF AMENDMENTS RE: ADOPTION OF TSTF-511, REVISION 0, "ELIMINATE WORKING HOUR RESTRICTIONS FROM TS 5.2.2 TO SUPPORT COMPLIANCE WITH 10 CFR PART 26" (TAC NOS. ME1198, ME1199, ME1200, ME1201, ME1202, ME1203, ME1204, ME1205, ME1206, AND ME1207)

Dear Mr. McCann:

The Nuclear Regulatory Commission has issued amendments consisting of changes to the Technical Specifications (TSs) in response to your application dated April 27, 2009, as supplemented by letter dated July 10, 2009. The following amendments are enclosed:

- Amendment No. 237 to Renewed Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1;
- Amendment No. 285 to Renewed Facility Operating License No. NPF-6 for Arkansas Nuclear One, Unit No. 2;
- Amendment No. 295 to Renewed Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant;
- Amendment No. 183 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1;
- Amendment No. 261 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2;
- Amendment No. 240 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3;

- Amendment No. 238 to Renewed Facility Operating License No. DPR-20 for the Palisades Nuclear Plant;
- Amendment No. 233 to Facility Operating License No. DPR-35 for the Pilgrim Nuclear Power Station;
- Amendment No. 164 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1; and
- Amendment No. 221 to Facility Operating License No. NPF-38 for the Waterford Steam Electric Station, Unit 3.

The amendments delete those portions of TSs superseded by Title 10 of the *Code of Federal Regulations* (10 CFR) Part 26, Subpart I, consistent with NRC-approved TS Task Force (TSTF) traveler TSTF-511, Revision 0, "Eliminate Working Hour Restrictions from TS 5.2.2 to Support Compliance with 10 CFR Part 26."

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,



Carl F. Lyon, Project Manager  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-313, 50-368, 50-333,  
50-416, 50-247, 50-286, 50-255, 50-293,  
50-458, and 50-382

Enclosures:

1. Amendment No. 237 to DPR-51
2. Amendment No. 285 to NPF-6
3. Amendment No. 295 to DPR-59
4. Amendment No. 183 to NPF-29
5. Amendment No. 261 to DPR-26
6. Amendment No. 240 to DPR-64
7. Amendment No. 238 to DPR-20
8. Amendment No. 233 to DPR-35
9. Amendment No. 164 to NPF-47
10. Amendment No. 221 to NPF-38
11. Safety Evaluation

cc w/encls: Distribution via Listserv



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 237  
Renewed License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated April 27, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

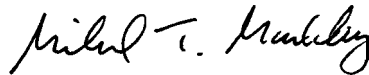
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-51 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 237, are hereby incorporated in the renewed license. EOI shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented by October 1, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Renewed Facility  
Operating License No. DPR-51  
and Technical Specifications

Date of Issuance: August 4, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 237  
RENEWED FACILITY OPERATING LICENSE NO. DPR-51  
DOCKET NO. 50-313

Replace the following pages of the Renewed Facility Operating License No. DPR-51 and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Operating License

REMOVE

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INSERT

3

Technical Specifications

REMOVE

5.0-3

INSERT

5.0-3

- (5) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
  - (6) EOI, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- c. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level  
EOI is authorized to operate the facility at steady state reactor core power levels not in excess of 2568 megawatts thermal.
  - (2) Technical Specifications  
The Technical Specifications contained in Appendix A, as revised through Amendment No. 237, are hereby incorporated in the renewed license. EOI shall operate the facility in accordance with the Technical Specifications.
  - (3) Safety Analysis Report  
The licensee's SAR supplement submitted pursuant to 10 CFR 54.21(d), as revised on March 14, 2001, describes certain future inspection activities to be completed before the period of extended operation. The licensee shall complete these activities no later than May 20, 2014.
  - (4) Physical Protection  
EOI shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans, including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains Safeguards Information protected under 10 CFR 73.21, is entitled: "Arkansas Nuclear One Physical Security Plan, Training and Qualifications Plan, and Safeguards Contingency Plan," as submitted on May 4, 2006.

## 5.0 ADMINSTRATIVE CONTROLS

### 5.2 Organization

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- c. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) for one unit, one control room, and 5.2.2.a and 5.2.2.f for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
- e. The operations manager or assistant operations manager shall hold an SRO license.
- f. In MODES 1, 2, 3, or 4, an individual shall provide advisory technical support for the operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift.



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ENTERGY OPERATIONS, INC.

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 285  
Renewed License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated April 27, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 2



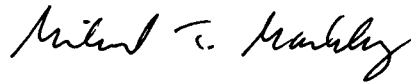
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-6 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 285, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications

3. The license amendment is effective as of its date of issuance and shall be implemented by October 1, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Renewed Facility  
Operating License No. NPF-6  
Technical Specifications

Date of Issuance: August 4, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 285  
RENEWED FACILITY OPERATING LICENSE NO. NPF-6  
DOCKET NO. 50-368

Replace the following pages of the Renewed Facility Operating License No. NPF-6 and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Operating License

REMOVE

-3-

INSERT

-3-

Technical Specifications

REMOVE

6-2

INSERT

6-2

- (4) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) EOI, pursuant to the Act and 10 CFR Parts 30 and 70 to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to conditions specified in the following Commission regulations in 10 CFR Chapter I; Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

EOI is authorized to operate the facility at steady state reactor core power levels not in excess of 3026 megawatts thermal. Prior to attaining this power level EOI shall comply with the conditions in Paragraph 2.C.(3).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 285, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

Exemptive 2nd paragraph of 2.C.2 deleted per Amendment 20, 3/3/81.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following issuance of the renewed license or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission.

- (a) Deleted per Amendment 24, 6/19/81.

## ADMINISTRATIVE CONTROLS

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### 6.2.2 UNIT STAFF

- a. A non-licensed operator shall be on site when fuel is in the reactor and two additional non-licensed operators shall be on site when the reactor is in MODES 1, 2, 3, or 4.
- b. The minimum shift crew composition for licensed operators shall meet the minimum staffing requirements of 10 CFR 50.54(m)(2)(i) for one unit, one control room.
- c. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) for one unit, one control room, and 6.2.2.a and 6.2.2.f for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
- e. The operations manager or the assistant operations manager shall hold a SRO license.
- f. In MODES 1, 2, 3, or 4, an individual shall provide advisory technical support for the operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift.



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ENERGY NUCLEAR FITZPATRICK, LLC  
AND ENERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT  
AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 295  
Renewed Facility Operating License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Nuclear Operations, Inc. (the licensee), dated April 27, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-59 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 295, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented by October 1, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION



Nancy L. Salgado, Chief  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Renewed Facility Operating  
License and Technical Specifications

Date of Issuance: August 4, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 295  
RENEWED FACILITY OPERATING LICENSE NO. DPR-59  
DOCKET NO. 50-333

Replace the following page of the License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page  
-3-

Insert Page  
-3-

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages  
5.2-2

Insert Pages  
5.2-2

- (4) ENO pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess, and use, at any time, any byproduct, source and special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration; or associated with radioactive apparatus, components or tools.
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

ENO is authorized to operate the facility at steady state reactor core power levels not in excess of 2536 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 295, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Fire Protection

ENO shall implement and maintain in effect all provisions of the approved fire protections program as described in the Final Safety Analysis Report for the facility and as approved in the SER dated November 20, 1972; the SER Supplement No. 1 dated February 1, 1973; the SER Supplement No. 2 dated October 4, 1974; the SER dated August 1, 1979; the SER Supplement dated October 3, 1980; the SER Supplement dated February 13, 1981; the NRC Letter dated February 24, 1981; Technical Specification Amendments 34 (dated January 31, 1978), 80 (dated May 22, 1984), 134 (dated July 19, 1989), 135 (dated September 5, 1989), 142 (dated October 23, 1989), 164 (dated August 10, 1990), 176 (dated January 16, 1992), 177 (dated February 10, 1992), 186 (dated February 19, 1993), 190 (dated June 29, 1993), 191 (dated July 7, 1993), 206 (dated February 28, 1994) and 214 (dated June 27, 1994); and NRC Exemptions and associated safety evaluations dated April 26, 1983, July 1, 1983, January 11, 1985, April 30, 1986, September 15, 1986 and September 10, 1992 subject to the following provision:



5.2 Organization

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5.2.2 Plant Staff (continued)

- b. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) and 5.2.2.a and 5.2.2.f for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
  - c. A radiation protection technician shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
  - d. Deleted
  - e. The operations manager or assistant operations manager shall hold an SRO license.
  - f. When the plant is in MODE 1, 2, or 3, an individual shall provide advisory technical support to the shift supervisor (SS) in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the plant. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift, published in the October 28, 1985 Federal Register (50 FR 43621).
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
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ENTERGY OPERATIONS, INC.

SYSTEM ENERGY RESOURCES, INC.

SOUTH MISSISSIPPI ELECTRIC POWER ASSOCIATION

ENTERGY MISSISSIPPI, INC.

DOCKET NO. 50-416

GRAND GULF NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 183  
License No. NPF-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated April 27, 2009, as supplemented by letter dated July 10, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 4

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-29 is hereby amended to read as follows:

- (2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 183 are hereby incorporated in the license. Entergy Operations, Inc. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented by October 1, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Facility Operating  
License No. NPF-29 and the  
Technical Specifications

Date of Issuance: August 4, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 183

FACILITY OPERATING LICENSE NO. NPF-29

DOCKET NO. 50-416

Replace the following pages of the Facility Operating License No. NPF-29 and the Appendix A, Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Operating License

<u>Remove</u>	<u>Insert</u>
4	4

Technical Specifications

<u>Remove</u>	<u>Insert</u>
5.0-3	5.0-3
5.0-4	5.0-4

(b) SERI is required to notify the NRC in writing prior to any change in (i) the terms or conditions of any new or existing sale or lease agreements executed as part of the above authorized financial transactions, (ii) the GGNS Unit 1 operating agreement, (iii) the existing property insurance coverage for GGNS Unit 1 that would materially alter the representations and conditions set forth in the Staff's Safety Evaluation Report dated December 19, 1988 attached to Amendment No. 54. In addition, SERI is required to notify the NRC of any action by a lessor or other successor in interest to SERI that may have an effect on the operation of the facility.

C. The license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Entergy Operations, Inc. is authorized to operate the facility at reactor core power levels not in excess of 3898 megawatts thermal (100 percent power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 183 are hereby incorporated into this license. Entergy Operations, Inc. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

The Surveillance Requirements (SRs) for Diesel Generator 12 contained in the Technical Specifications and listed below, are not required to be performed immediately upon implementation of Amendment No. 169. The SRs listed below shall be successfully demonstrated at the next regularly scheduled performance.

SR 3.8.1.9,  
SR 3.8.1.10, and  
SR 3.8.1.14

Amendment No. 183

5.2 Organization (continued)

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5.2.2 Unit Staff

The unit staff organization shall include the following:

- a. A non-licensed operator shall be on site when fuel is in the reactor and an additional non-licensed operator shall be on site while the unit is in MODE 1, 2, or 3.
- b. At least one licensed RO shall be present in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, or 3, at least one licensed SRO shall be present in the control room.
- c. Shift crew composition may be one less than the minimum requirement of 10CFR50.54(m)(2)(i) and Specifications 5.2.2.a and 5.2.2.f for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- d. A health physicist shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.

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(continued)

5.2 Organization

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5.2.2 Unit Staff (continued)

- e. The operations manager or at least one operations middle manager shall hold an SRO license.
  - f. The Shift Technical Advisor (STA) shall provide advisory technical support to the shift superintendent in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. In addition, the STA shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift.
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UNITED STATES  
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ENTERGY NUCLEAR INDIAN POINT 2, LLC

ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 261  
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Nuclear Operations, Inc. (the licensee), dated April 27, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 261, are hereby incorporated in the license. ENO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented by October 1, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION



Nancy L. Salgado, Chief  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the License and  
Technical Specifications

Date of Issuance: August 4, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 261

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Replace the following page of the License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page

-3-

Insert Page

-3-

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

5.2-2

Insert Pages

5.2-2

instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (4) ENO pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; Amdt. 42  
10-17-78
- (5) ENO pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility. Amdt. 220  
09-06-01

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

ENO is authorized to operate the facility at steady state reactor core power levels not in excess of 3216 megawatts thermal. Amdt. 241  
10-27-2004

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 261, are hereby incorporated in the license. ENO shall operate the facility in accordance with the Technical Specifications.

(3) The following conditions relate to the amendment approving the conversion to Improved Standard Technical Specifications:

- 1. This amendment authorizes the relocation of certain Technical Specification requirements and detailed information to licensee-controlled documents as described in Table R, "Relocated Technical Specifications from the CTS," and Table LA, "Removed Details and Less Restrictive Administrative Changes to the CTS" attached to the NRC staff's Safety Evaluation enclosed with this amendment. The relocation of requirements and detailed information shall be completed on or before the implementation of this amendment.

5.2 Organization

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5.2.2 Unit Staff (continued)

- b. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) and 5.2.2.a and 5.2.2.f for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
  - c. A radiation protection technician shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
  - d. Not Used
  - e. The operations manager or assistant operations manager shall hold an SRO license.
  - f. An individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift. This position must be manned only when in MODES 1, 2, 3, or 4.
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENTERGY NUCLEAR INDIAN POINT 3, LLC

ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 240  
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Nuclear Operations, Inc. (the licensee), dated April 27, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

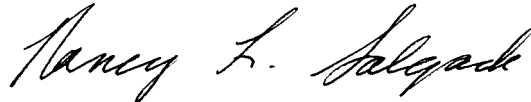
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 240, are hereby incorporated in the license. ENO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented by October 1, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION



Nancy L. Salgado, Chief  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the License and  
Technical Specifications

Date of Issuance: August 4, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 240

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Replace the following page of the License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page

-3-

Insert Page

-3-

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

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5.0-4

Insert Pages

5.0-3

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- (4) ENO pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; Amdt. 203  
11/27/00
- (5) ENO pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility. Amdt. 203  
11/27/00
- C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
  - (1) Maximum Power Level  
ENO is authorized to operate the facility at steady state reactor core power levels not in excess of 3216 megawatts thermal (100% of rated power).
  - (2) Technical Specifications  
The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 240 are hereby incorporated in the License. ENO shall operate the facility in accordance with the Technical Specifications.
  - (3) (DELETED) Amdt. 205  
2-27-01
  - (4) (DELETED) Amdt. 205  
2-27-01
- D. (DELETED) Amdt.46  
2-16-83
- E. (DELETED) Amdt.37  
5-14-81
- F. This amended license is also subject to appropriate conditions by the New York State Department of Environmental Conservation in its letter of May 2, 1975, to Consolidated Edison Company of New York, Inc., granting a Section 401 certification under the Federal Water Pollution Control Act Amendments of 1972.



5.2 Organization

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5.2.2 Unit Staff

The unit staff organization shall include the following:

- a. A non-licensed operator shall be assigned to each reactor containing fuel and an additional non-licensed operator shall be assigned for each control room from which a reactor is operating in MODES 1, 2, 3, or 4.
- b. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) and 5.2.2.a and 5.2.2.f for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- c. A radiation protection technician shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
- d. Not Used

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(continued)

5.2 Organization

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5.2.2 Unit Staff (continued)

- e. The operations manager or assistant operations manager shall hold an SRO license.
  - f. An individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift. This position must be manned in Mode 1, 2, 3 or 4 only.
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-255

PALISADES NUCLEAR PLANT

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 238  
License No. DPR-20

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Nuclear Operations, Inc. (the licensee), dated April 27, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 7

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to the license amendment and Paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-20 is hereby amended to read as follows:

The Technical Specifications contained in Appendix A, as revised through Amendment No. 238, and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. Entergy Nuclear Operations shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance and shall be implemented by October 1, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION



Lois M. James, Chief  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Facility Operating License  
and Technical Specifications

Date of Issuance: August 4, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 238  
RENEWED FACILITY OPERATING LICENSE NO. DPR-20  
DOCKET NO. 50-255

Replace the following page of the Renewed Facility Operating License No. DPR-20 with the attached revised page. The changed area is identified by a marginal line.

REMOVE

INSERT

-3-

-3-

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

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5.0-3

- (1) Pursuant to Section 104b of the Act, as amended, and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," (a) ENP to possess and use, and (b) ENO to possess, use and operate, the facility as a utilization facility at the designated location in Van Buren County, Michigan, in accordance with the procedures and limitation set forth in this license;
  - (2) ENO, pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess, and use source and special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report, as supplemented and amended;
  - (3) ENO, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use byproduct, source, and special nuclear material as sealed sources for reactor startup, reactor instrumentation, radiation monitoring equipment calibration, and fission detectors in amounts as required;
  - (4) ENO, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material for sample analysis or instrument calibration, or associated with radioactive apparatus or components; and
  - (5) ENO, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operations of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations in 10 CFR Chapter I and is subject to all applicable provisions of the Act; to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) ENO is authorized to operate the facility at steady-state reactor core power levels not in excess of 2565.4 Megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.
  - (2) The Technical Specifications contained in Appendix A, as revised through Amendment No. 238, and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. ENO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
  - (3) ENO shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SERs dated 09/01/78, 03/19/80, 02/10/81, 05/26/83, 07/12/85, 01/29/86, 12/03/87, and 05/19/89 and subject to the following provisions:

## 5.2 Organization

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### 5.2.2 Plant Staff (continued)

- c. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i), and 5.2.2.a and 5.2.2.g for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the requirements.
  - d. A radiation safety technician shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
  - e. Not Used
  - f. The operations manager or an assistant operations manager shall hold an SRO license. The individual holding the SRO license shall be responsible for directing the activities of the licensed operators.
  - g. An individual shall provide advisory technical support to the plant operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the plant. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift (Published in Federal Register 50 FR 43621, October 28, 1985).
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENTERGY NUCLEAR GENERATION COMPANY  
ENTERGY NUCLEAR OPERATIONS, INC.  
DOCKET NO. 50-293  
PILGRIM NUCLEAR POWER STATION  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 233  
License No. DPR-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by Entergy Nuclear Operations, Inc. (the licensee), dated April 27, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



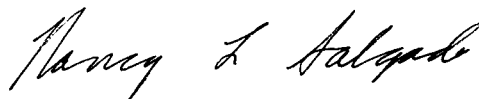
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 233, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented by October 1, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION



Nancy L. Salgado, Chief  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the License and  
Technical Specifications

Date of Issuance: August 4, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 233

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Replace the following page of the Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

Insert

-3-

-3-

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Insert

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5.0-3

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 233, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. Records

ENO shall keep facility operating records in accordance with the requirements of the Technical Specifications.

D. Equalizer Valve Restriction - DELETED

E. Recirculation Loop Inoperable - DELETED

F. Fire Protection

ENO shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SER dated December 21, 1978 as supplemented subject to the following provision:

ENO may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

G. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Pilgrim Nuclear Power Station Physical Security, Training and Qualification, and Safeguards Contingency Plan, Revision 0" submitted by letter dated October 13, 2004, as supplemented by letter dated May 15, 2006.

5.2 Organization

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5.2.2 Unit Staff (continued)

- b. At least one licensed Reactor Operator (RO) shall be present in the control room when fuel is in the reactor. In addition, while the unit is in an operational mode other than Cold Shutdown or Refueling, at least one licensed Senior Reactor Operator (SRO) shall be present in the control room.
  - c. At least two licensed ROs shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.
  - d. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) and 5.2.2.a and 5.2.2.i for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
  - e. Higher grade licensed operators may take the place of lower grade licensed or unlicensed personnel.
  - f. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
  - g. Deleted
  - h. The operations manager or assistant operations manager shall hold a Senior Reactor Operator License.
  - i. An individual shall provide advisory technical support to the unit operations shift crew in the areas of engineering and accident assessment. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift. This individual with a Senior Reactor Operator license may simultaneously serve a required SRO position.
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENERGY GULF STATES LOUISIANA, LLC

AND

ENERGY OPERATIONS, INC.

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 164  
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated April 27, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

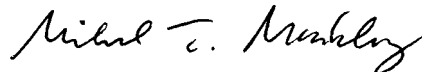
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 164 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. EOI shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented by October 1, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Facility Operating  
License No. NPF-47 and  
Technical Specifications

Date of Issuance: August 4, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 164

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following pages of the Facility Operating License No. NPF-47 and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change.

Facility Operating License

<u>Remove</u>	<u>Insert</u>
-3-	-3-

Technical Specifications

<u>Remove</u>	<u>Insert</u>
5.0-3	5.0-3
5.0-4	5.0-4

- (3) EOI, pursuant to the Act and 10 CFR Part 70, to receive, possess and to use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
  - (4) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - (5) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
  - (6) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level  
  
EOI is authorized to operate the facility at reactor core power levels not in excess of 3091 megawatts thermal (100% rated power) in accordance with the conditions specified herein. The items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license.
  - (2) Technical Specifications and Environmental Protection Plan  
  
The Technical Specifications contained in Appendix A, as revised through Amendment No. 164 and the Environmental Protection Plan contained in Appendix 8, are hereby incorporated in the license. EOI shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.



5.2 Organization

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5.2.2 Unit Staff (continued)

- b. At least one licensed Reactor Operator (RO) shall be present in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, or 3, at least one licensed Senior Reactor Operator (SRO) shall be present in the control room.
- c. Shift crew composition may be one less than the minimum requirements of 10CFR50.54(m)(2)(i) and specifications 5.2.2.a and 5.2.2.g for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- d. A health physics technician shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
- e. DELETED

(continued)

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5.2 Organization

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5.2.2 Unit Staff (continued)

- f. The operations manager or at least one operations middle manager shall hold an SRO license.
  - g. The Shift Technical Advisor (STA) shall provide advisory technical support to the shift superintendent in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. In addition, the STA shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift.
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENERGY OPERATIONS, INC.

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 221  
License No. NPF-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (EOI), dated April 27, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

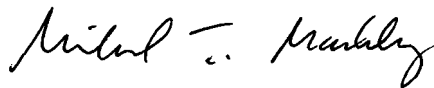
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.2 of Facility Operating License No. NPF-38 is hereby amended to read as follows:

2. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 221, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. EOI shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented by October 1, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Facility Operating  
License No. NPF-38 and  
Technical Specifications

Date of Issuance: August 4, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 221

TO FACILITY OPERATING LICENSE NO. NPF-38

DOCKET NO. 50-382

Replace the following pages of the Facility Operating License and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Operating License

REMOVE

INSERT

-4-

-4-

Technical Specifications

REMOVE

INSERT

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or indirectly any control over (i) the facility, (ii) power or energy produced by the facility, or (iii) the licensees of the facility. Further, any rights acquired under this authorization may be exercised only in compliance with and subject to the requirements and restrictions of this operating license, the Atomic Energy Act of 1954, as amended, and the NRC's regulations. For purposes of this condition, the limitations of 10 CFR 50.81, as now in effect and as they may be subsequently amended, are fully applicable to the equity investors and any successors in interest to the equity investors, as long as the license for the facility remains in effect.

- (b) Entergy Louisiana, LLC (or its designee) to notify the NRC in writing prior to any change in (i) the terms or conditions of any lease agreements executed as part of the above authorized financial transactions, (ii) any facility operating agreement involving a licensee that is in effect now or will be in effect in the future, or (iii) the existing property insurance coverages for the facility, that would materially alter the representations and conditions, set forth in the staff's Safety Evaluation enclosed to the NRC letter dated September 18, 1989. In addition, Entergy Louisiana, LLC or its designee is required to notify the NRC of any action by equity investors or successors in interest to Entergy Louisiana, LLC that may have an effect on the operation of the facility.

- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

- 1. Maximum Power Level

EOI is authorized to operate the facility at reactor core power levels not in excess of 3716 megawatts thermal (100% power) in accordance with the conditions specified herein.

- 2. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 221, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. EOI shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

## ADMINISTRATIVE CONTROLS

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### UNIT STAFF (Continued)

- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the reactor is in MODE 1, 2, 3, or 4, at least one licensed Senior Operator shall be in the control room.
- c. A Health Physics Technician\* shall be on site when fuel is in the reactor.
- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- e. The Operations Manager or the Assistant Operations Manager (Shift) shall hold a senior reactor operator license.

\*This requirement tolerates Health Physics Technician unexpected absences for not more than 2 hours, provided management takes immediate action to fill the required Health Physics Technician position.

ADMINISTRATIVE CONTROLS

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Page 6-2a  
Not Used.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NOS. 237 AND 285 TO RENEWED FACILITY  
OPERATING LICENSE NOS. DPR-51 AND NPF-6,  
AMENDMENT NO. 295 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-59,  
AMENDMENT NO. 183 TO FACILITY OPERATING LICENSE NO. NPF-29,  
AMENDMENT NOS. 261 AND 240 TO FACILITY OPERATING LICENSE NOS. DPR-26 AND  
DPR-64,  
AMENDMENT NO. 238 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-20,  
AMENDMENT NO. 233 TO FACILITY OPERATING LICENSE NO. DPR-35,  
AMENDMENT NO. 164 TO FACILITY OPERATING LICENSE NO. NPF-47, AND  
AMENDMENT NO. 221 TO FACILITY OPERATING LICENSE NO. NPF-38  
ENTERGY OPERATIONS, INC., AND ENTERGY NUCLEAR OPERATIONS, INC.  
ARKANSAS NUCLEAR ONE, UNITS 1 AND 2  
JAMES A. FITZPATRICK NUCLEAR POWER PLANT  
GRAND GULF NUCLEAR STATION, UNIT 1  
INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3  
PALISADES NUCLEAR PLANT  
PILGRIM NUCLEAR POWER STATION  
RIVER BEND STATION, UNIT 1  
WATERFORD STEAM ELECTRIC STATION, UNIT 3  
DOCKET NOS. 50-313, 50-368, 50-333, 50-416, 50-247, 50-286,  
50-255, 50-293, 50-458, AND 50-382

1.0 INTRODUCTION

By application dated April 27, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML091390436), as supplemented by letter dated July 10, 2009 (ADAMS Accession No. ML091910486), Entergy Operations, Inc., and Entergy Nuclear Operations, Inc. (EOI and ENO, the licensees), requested changes to the Technical Specifications (TSs) for Arkansas Nuclear One, Units 1 and 2, James A. FitzPatrick Nuclear Power Plant, Grand Gulf Nuclear Station, Unit 1, Indian Point Nuclear Generating Unit Nos. 2 and 3, and Palisades Nuclear Plant, Pilgrim Nuclear Power Station, River Bend Station, Unit 1, and Waterford Steam Electric Station, Unit 3.

The proposed changes would delete those portions of TSs superseded by Title 10 of the *Code of Federal Regulations* (10 CFR) Part 26, Subpart I, consistent with U.S. Nuclear Regulatory Commission (NRC)-approved TS Task Force (TSTF) change traveler TSTF-511, Revision 0, "Eliminate Working Hour Restrictions from TS 5.2.2 to Support Compliance with 10 CFR Part 26." The NRC announced the availability of this TS improvement in the *Federal Register* on December 30, 2008 (73 FR 79923), as part of the consolidated line item improvement process (CLIIP). The licensees stated in the April 27, 2009, submittal that they are proposing no variations or deviations from TSTF-511, Revision 0, or the NRC staff's model safety evaluation as published in the *Federal Register* on December 30, 2008, as part of the CLIIP Notice of Availability.

The supplement dated July 10, 2009, which applied only to Grand Gulf Nuclear Station, Unit 1, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on June 2, 2009 (74 FR 26432).

Specifically, the proposed changes would delete the portion of TSs associated with unit staff, as listed in Table 1 below.

**Table 1**

<b>Nuclear Power Plant</b>	<b>TS Section</b>	<b>Paragraph</b>
Arkansas Nuclear One Unit 1 (ANO1)	TS 5.2.2	e
Arkansas Nuclear One Unit 2 (ANO2)	TS 6.2.2	e
Grand Gulf Nuclear Station Unit 1 (GGNS)	TS 5.2.2	e
Indian Point Nuclear Generating Unit 2 (IP2)	TS 5.2.2	d
Indian Point Nuclear Generating Unit 3 (IP3)	TS 5.2.2	d
James A. FitzPatrick Nuclear Power Plant (JAF)	TS 5.2.2	d
Palisades Nuclear Plant (PAL)	TS 5.2.2	e
Pilgrim Nuclear Power Station (PIL)	TS 5.2.2	g
River Bend Power Station (RBS)	TS 5.2.2	e
Waterford Steam Electric Station Unit 3 (WF3)	TS 6.2.2	e

## 2.0 REGULATORY EVALUATION

The history of NRC regulations pertaining to prevention of worker impairment is summarized in the *Federal Register* notice containing the final rule that amended 10 CFR Part 26 (73 FR 16966, March 31, 2008). Subpart I, "Managing Fatigue," of 10 CFR Part 26 provides the regulatory requirements for managing worker fatigue at nuclear power plants.

In Section 50.36, "Technical specifications," of 10 CFR, the Commission established its regulatory requirements related to the content of TSs. Pursuant to 10 CFR 50.36, TSs are required to include items in the following five specific categories related to station operation:

(1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. The administrative controls are the provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facilities in a safe manner.

The NRC's guidance for the format and content of licensee TSs can be found in NUREG-1430, Revision 3.0, "Standard Technical Specifications Babcock and Wilcox Plants," NUREG-1431, Revision 3.0, "Standard Technical Specifications Westinghouse Plants," NUREG-1432, Revision 3.0, "Standard Technical Specifications Combustion Engineering Plants," NUREG-1433, Revision 3.0, "Standard Technical Specifications General Electric Plants, BWR/4," and NUREG-1434, Revision 3.0, "Standard Technical Specifications General Electric Plants, BWR/6," referred to as the Standard Technical Specifications (STS). Section 5 of the STS contains administrative controls. Paragraph d of Section 5.2.2 of the STS contains requirements for administrative procedures to limit the working hours of personnel who perform safety related functions. This paragraph represents the NRC's guidance on how licensee TS requirements should address work hour controls. Licensees adhere to the guidance in varying degrees due to minor administrative differences and differences in each licensee's current licensing basis. The inconsistent level of adherence to NRC guidance has led to inconsistent TS interpretation and implementation. This has also made it difficult for NRC to enforce the requirements.

The new requirements of 10 CFR Part 26, Subpart I supersede the guidance for requirements found in paragraph d of Section 5.2.2 of all STS. Subpart I distinguishes between work hour controls and fatigue management and strengthens the requirements for both. Subpart I requires nuclear power plant licensees to ensure against worker fatigue adversely affecting public health and safety and the common defense and security by establishing clear and enforceable requirements for the management of worker fatigue. Licensees are required to implement Subpart I by October 1, 2009, as announced in the Final Rule that revised 10 CFR Part 26 (73 FR 16966, March 31, 2008). TSTF-511, Revision 0, proposed a change to STS that would delete paragraph d of STS 5.2.2, since it is superseded by the changes to 10 CFR Part 26. This change was approved by the NRC staff and made available in the *Federal Register* on December 30, 2008 (73 FR 79923).

## 2.1 Adoption of TSTF-511, Revision 0, by the Plants Listed in Table 1

Adoption of TSTF-511, Revision 0, and implementation of 10 CFR Part 26, Subpart I by the licensees will provide reasonable assurance that the licensees will maintain limits on the working hours of personnel who perform safety related functions. In its submittal dated April 27, 2009, the licensees made the following regulatory commitment:

Removal of the plant-specific Technical Specification requirements will be performed concurrently with the implementation of the 10 CFR 26, Subpart I requirements. This commitment will be completed no later than October 1, 2009.

### 3.0 TECHNICAL EVALUATION

Controls on work hours for personnel at nuclear power plants are necessary to prevent worker fatigue from adversely affecting public health and safety and the common defense and security. Work hour controls for EOI and ENO nuclear plants are currently located in the TS section and paragraph referenced in Table 1. When implemented, the regulatory requirements of 10 CFR Part 26, Subpart I replace the plant-specific TS requirements found in Table 1 for the EOI and ENO nuclear plants.

The licensees propose to delete the TS sections and paragraphs referenced in Table 1. The licensees committed to implement the new requirements of 10 CFR Part 26, Subpart I concurrently with the deletion of the TS requirements on work hour controls. The NRC staff finds that reasonable controls for the implementation and subsequent evaluation of the proposed changes pertaining to the above regulatory commitment are provided by the licensees' administrative processes, specifically its commitment management program (see NRC Regulatory Issue Summary 2000-017, "Managing Regulatory Commitments Made by Power Reactor Licensees to the NRC Staff," dated September 21, 2000).

#### 3.1 EOI and ENO Nuclear Plant-Specific TS Changes

##### 3.1.1 Arkansas Nuclear One Unit 1 (ANO1)

The text of the current ANO1 TS 5.2.2, paragraph e, is not an exact duplicate of the text in TSTF-511, Revision 0, STS 5.2.2, paragraph d, but is similar.

- TSTF-511, Revision 0, STS 5.2.2, paragraph d states the following:

Administrative procedures shall be developed and implemented to limit the working hours of personnel who perform safety related functions (e.g., [licensed Senior Reactor Operators (SROs), licensed Reactor Operators (ROs), health physicists, auxiliary operators, and key maintenance personnel]).

The controls shall include guidelines on working hours that ensure adequate shift coverage shall be maintained without routine heavy use of overtime.

Any deviation from the above guidelines shall be authorized in advance by the plant manager or the plant manager's designee, in accordance with approved administrative procedures, and with documentation of the basis for granting the deviation. Routine deviation from the working hour guidelines shall not be authorized.

Controls shall be included in the procedures to require a periodic independent review be conducted to ensure that excessive hours have not been assigned.

- The current ANO1 TS 5.2.2, paragraph e states,

The amount of overtime worked by unit staff members performing safety related functions must be limited and controlled in accordance with the NRC Policy Statement on working hours (Generic Letter 82-12).

Generic Letter 82-12 has the same guidelines as the requirements of STS 5.2.2, paragraph d. Since the intent of TSTF-511, Revision 0, is to delete TSs superseded by the revised regulations in 10 CFR Part 26, Subpart I, the proposed change is acceptable.

### 3.1.2 Arkansas Nuclear One Unit 2 (ANO2)

The current ANO2 TS 6.2.2, paragraph e has text similar to the text of TSTF-511, Revision 0, STS 5.2.2, paragraph d, but is not an exact duplicate.

- TSTF-511, Revision 0, STS 5.2.2, paragraph d states the following:

Administrative procedures shall be developed and implemented to limit the working hours of personnel who perform safety related functions (e.g., [licensed Senior Reactor Operators (SROs), licensed Reactor Operators (ROs), health physicists, auxiliary operators, and key maintenance personnel]).

The controls shall include guidelines on working hours that ensure adequate shift coverage shall be maintained without routine heavy use of overtime.

Any deviation from the above guidelines shall be authorized in advance by the plant manager or the plant manager's designee, in accordance with approved administrative procedures, and with documentation of the basis for granting the deviation. Routine deviation from the working hour guidelines shall not be authorized.

Controls shall be included in the procedures to require a periodic independent review be conducted to ensure that excessive hours have not been assigned.

- The current ANO2 TS 6.2.2, paragraph e states,

The amount of overtime worked by unit staff members performing safety related functions must be limited and controlled in accordance with the NRC Policy Statement on working hours (Generic Letter 82-12).

Generic Letter 82-12 has the same guidelines as the requirements of STS 5.2.2, paragraph d. Since the intent of TSTF-511, Revision 0, is to delete TSs superseded by the revised regulations in 10 CFR Part 26, Subpart I, the proposed change is acceptable.

### 3.1.3. James A. FitzPatrick Nuclear Power Plant (JAF)

The text of the current JAF TS 5.2.2, paragraph d, is a duplicate of the text of TSTF-511, Revision 0, STS 5.2.2, paragraph d.

### 3.1.4 Grand Gulf Nuclear Station Unit 1 (GGNS)

There are minor differences between GGNS TSs and TSTF-511, Revision 0. The equivalent GGNS TSs where work hour restrictions are located is TS 5.2.2, paragraph e. This differs from TSTF-511, Revision 0, where work hour restrictions are located in STS 5.2.2, paragraph d.

Also, the text of the current GGNS TS 5.2.2, paragraph e, is similar to the text of TSTF-511, Revision 0, STS 5.2.2, paragraph d, but it is not an exact duplicate. The most significant differences are as follows:

- TSTF-511, Revision 0, STS 5.2.2, paragraph d states, "The controls shall include guidelines on working hours that ensure adequate shift coverage shall be maintained without routine heavy use of overtime." GGNS TS 5.2.2, paragraph e states, "Adequate shift coverage shall be maintained without routine heavy use of overtime," and incorporates the guidelines to be followed in TS 5.2.2, paragraph e.
- TSTF-511, Revision 0, STS 5.2.2, paragraph d states that the plant manager or plant manager's designee must authorize deviations from the guideline on working hours. GGNS TS 5.2.2, paragraph e allows such deviations to be approved by the plant manager, or their designee, or by higher levels of management.
- TSTF-511, Revision 0, STS 5.2.2, paragraph d requires the conduct of a periodic, independent review to ensure that excessive hours have not been assigned. GGNS TS 5.2.2, paragraph e does not require that this review be independent but rather requires that the review be conducted monthly by the plant manager or a designee.

These differences are insignificant as the intent of TSTF-511, Revision 0, is to delete TSs superseded by the revised regulations in 10 CFR Part 26, Subpart I.

### 3.1.5 Indian Point Nuclear Generating Unit 2 (IP2)

The text of the current IP2 TS 5.2.2, paragraph d, is a duplicate of the text of TSTF-511, Revision 0, STS 5.2.2, paragraph d.

### 3.1.6 Indian Point Nuclear Generating Unit 3 (IP3)

The text of the current IP3 TS 5.2.2, paragraph d, is a duplicate of the text of TSTF-511, Revision 0, STS 5.2.2, paragraph d.

### 3.1.7 Palisades Nuclear Plant (PAL)

The current PAL TSs have minor differences when compared with TSTF-511, Revision 0. The equivalent PAL TS where work hour restrictions are located is TS 5.2.2, paragraph e. This differs from TSTF-511, Revision 0, where work hour restrictions are located in STS 5.2.2, paragraph d.

Also, the text of PAL TS 5.2.2, paragraph e, is similar to the text of TSTF-511, Revision 0, STS 5.2.2, paragraph d, but it is not an exact duplicate. The most significant difference is as follows:

- TSTF-511, Revision 0, STS 5.2.2, paragraph d states that the plant manager or plant manager's designee must authorize deviations from the guideline on working hours. PAL TS 5.2.2, paragraph e allows such deviations to be approved by the plant superintendent, or their designee.

These differences are insignificant since the intent of TSTF-511, Revision 0, is to delete TSs superseded by the revised regulations in 10 CFR Part 26, Subpart I.

### 3.1.8 Pilgrim Nuclear Power Station (PIL)

The current PIL TS 5.2.2, paragraph g, is similar to the text of TSTF-511, Revision 0, STS 5.2.2, paragraph d.

- TSTF-511, Revision 0, STS 5.2.2, paragraph d states the following:

Administrative procedures shall be developed and implemented to limit the working hours of personnel who perform safety related functions (e.g., [licensed Senior Reactor Operators (SROs), licensed Reactor Operators (ROs), health physicists, auxiliary operators, and key maintenance personnel]).

The controls shall include guidelines on working hours that ensure adequate shift coverage shall be maintained without routine heavy use of overtime.

Any deviation from the above guidelines shall be authorized in advance by the plant manager or the plant manager's designee, in accordance with approved administrative procedures, and with documentation of the basis for granting the deviation. Routine deviation from the working hour guidelines shall not be authorized.

Controls shall be included in the procedures to require a periodic independent review be conducted to ensure that excessive hours have not been assigned.

- The current PIL TS 5.2.2, paragraph g states,

The amount of overtime worked by unit staff members performing safety related functions shall be limited and controlled in accordance with the NRC Policy Statement on working hours (Generic Letter 82-12)."

Generic Letter 82-12 has the same guidelines as the requirements of STS 5.2.2, paragraph d. Since the intent of TSTF-511, Revision 0, is to delete TSs superseded by the revised regulations in 10 CFR Part 26, Subpart I, the proposed change is acceptable.

### 3.1.9 River Bend Station Unit 1 (RBS)

The current RBS TSs have minor differences when compared with TSTF-511, Revision 0. The equivalent RBS TSs where work hour restrictions are located is TS 5.2.2, paragraph e. This differs from TSTF-511, Revision 0, where work hour restrictions are located in STS 5.2.2, paragraph d.

The text of the current RBS TS 5.2.2, paragraph e, is similar to the text of TSTF-511, Revision 0, STS 5.2.2, paragraph d, but it is not an exact duplicate. The most significant differences are as follows:

- TSTF-511, Revision 0, STS 5.2.2, paragraph d states, "The controls shall include guidelines on working hours that ensure adequate shift coverage shall be maintained without routine heavy use of overtime." RBS TS 5.2.2, paragraph e states, "Adequate shift coverage shall be maintained without routine heavy use of overtime," and incorporates the guidelines to be followed in TS 5.2.2, paragraph e.
- TSTF-511, Revision 0, STS 5.2.2, paragraph d states that the plant manager or plant manager's designee must authorize deviations from the guideline on working hours. RBS TS 5.2.2, paragraph e allows such deviations to be approved by the plant manager, or their designee, or by higher levels of management.
- TSTF-511, Revision 0, STS 5.2.2, paragraph d requires the conduct of a periodic, independent review to ensure that excessive hours have not been assigned. RBS TS 5.2.2, paragraph e does not require that this review be independent but rather requires that the review be conducted monthly by the plant manager or their designee.

These differences are insignificant as the intent of TSTF-511, Revision 0, is to delete TSs superseded by the revised regulations in 10 CFR Part 26, Subpart I.



### 3.1.10 Waterford Steam Electric Station Unit 3 (WF3)

The current WF3 TSs have minor differences when compared with TSTF-511, Revision 0. The equivalent WF3 TSs where work hour restrictions are located is TS 6.2.2, paragraph e. This differs from TSTF-511, Revision 0, where work hour restrictions are located in STS 5.2.2, paragraph d.

The text of the current WF3 TS 6.2.2, paragraph e, is similar to the text of TSTF-511, Revision 0, STS 5.2.2, paragraph d, but it is not an exact duplicate. The most significant differences are as follows:

- TSTF-511, Revision 0, STS 5.2.2, paragraph d states, "The controls shall include guidelines on working hours that ensure adequate shift coverage shall be maintained without routine heavy use of overtime." WF3 TS 5.2.2, paragraph e states, "Adequate shift coverage shall be maintained without routine heavy use of overtime," and incorporates the guidelines to be followed in TS 6.2.2 paragraph e.
- TSTF-511, Revision 0, STS 5.2.2, paragraph d states that the plant manager or plant manager's designee must authorize deviations from the guideline on working hours. WF3 TS 6.2.2, paragraph e allows such deviations to be approved by the general manager plant operations, or their designee, or by higher levels of management.
- TSTF-511, Revision 0, STS 5.2.2, paragraph d requires the conduct of a periodic, independent review to ensure that excessive hours have not been assigned. WF3 TS 6.2.2, paragraph e does not require that this review be independent but rather requires that the review be conducted monthly by the general manager plant operations or their designee.
- TSTF-511, Revision 0, STS 5.2.2, paragraph d states that administrative procedures shall be developed and implemented to limit the working hours of personnel who perform safety-related functions. WF3 TS 6.2.2, paragraph e requires that administrative procedures shall be developed and implemented to limit the working hours of individuals of the nuclear power plant operating staff who are responsible for manipulating plant controls or for adjusting on-line systems and equipment affecting plant safety which have an immediate impact on public health and safety.

These differences are insignificant as the intent of TSTF-511, Revision 0, is to delete TSs superseded by the revised regulations in 10 CFR Part 26, Subpart I.

The NRC staff concluded that the licensees' proposed changes comply with the applicable regulatory requirements of 10 CFR 50.36 and 10 CFR Part 26 and are consistent with TSTF-511, Revision 0, which changed the STS. Based on the licensees' regulatory commitment noted in Section 2.1 above, there is reasonable assurance that the licensees will comply with the regulations for work hour controls, either through TS requirements or through

the requirements of 10 CFR Part 26, Subpart I, at all times. Accordingly, the staff has reasonable assurance that the licensees will continue to prevent worker fatigue from adversely affecting public health and safety and the common defense and security. Therefore, the NRC staff finds the proposed changes acceptable.

#### 4.0 REGULATORY COMMITMENTS

In the April 27, 2009, submittal, the licensees made the following regulatory commitment for each of the operating power reactors that are the subject of this License Amendment Request:

Removal of the plant-specific Technical Specification requirements will be performed concurrently with the implementation of the 10 CFR 26, Subpart I requirements. This commitment will be completed no later than October 1, 2009.

The NRC staff finds these commitments acceptable.

#### 5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State officials for Arkansas, Mississippi, Louisiana, New York, Massachusetts, and Michigan were notified of the proposed issuance of the amendments. The State officials had no comments.

#### 6.0 ENVIRONMENTAL CONSIDERATION

The amendments change recordkeeping, reporting, or administrative procedures or requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding published in the *Federal Register* on June 2, 2009 (74 FR 26432). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Bucholtz

Date: August 4, 2009

- Amendment No. 238 to Renewed Facility Operating License No. DPR-20 for the Palisades Nuclear Plant;
- Amendment No. 233 to Facility Operating License No. DPR-35 for the Pilgrim Nuclear Power Station;
- Amendment No. 164 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1; and
- Amendment No. 221 to Facility Operating License No. NPF-38 for the Waterford Steam Electric Station, Unit 3.

The amendments delete those portions of TSs superseded by Title 10 of the *Code of Federal Regulations* (10 CFR) Part 26, Subpart I, consistent with NRC-approved TS Task Force (TSTF) traveler TSTF-511, Revision 0, "Eliminate Working Hour Restrictions from TS 5.2.2 to Support Compliance with 10 CFR Part 26."

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,  
 /RA/  
 Carl F. Lyon, Project Manager  
 Plant Licensing Branch IV  
 Division of Operating Reactor Licensing  
 Office of Nuclear Reactor Regulation

Docket Nos. 50-313, 50-368, 50-333,  
 50-416, 50-247, 50-286, 50-255, 50-293,  
 50-458, and 50-382

Enclosures:

1. Amendment No. 237 to DPR-51
2. Amendment No. 285 to NPF-6
3. Amendment No. 295 to DPR-59
4. Amendment No. 183 to NPF-29
5. Amendment No. 261 to DPR-26
6. Amendment No. 240 to DPR-64
7. Amendment No. 238 to DPR-20
8. Amendment No. 233 to DPR-35
9. Amendment No. 164 to NPF-47
10. Amendment No. 221 to NPF-38
11. Safety Evaluation

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RidsOgcRp Resource	RidsNrrLAJBurkhardt Resource	RidsNrrPMJKim Resource	

ADAMS Accession No. ML091680321

\*SE dated

\*\*Previously concurred

OFFICE	NRR/LPL4/PM	NRR/LPL4/LA	DIRS/ITSB/BC	OGC
NAME	FLyon	JBurkhardt	RElliott*	AJones** - NLO
DATE	7/15/09	7/6/09	8/15/09	7/14/09
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DATE	7/23/09	7/24/09	8/4/09	8/4/09