PMSTPCOL PEmails

From: Govan, Tekia
Sent: Wednesday, May 13, 2009 12:39 PM
To: 'jeprice@stpegs.com'
Cc: STPCOL
Subject: DRAFT RAIs
Attachments: RAI 2370.doc; RAI 2364.doc; RAI 2578.doc; RAI 2580.doc; RAI 2579.doc

John,

Please review the attached RAIs. If you feel we need a conference call to clarify the requested information, please contact me. If a conference call is not needed (please send an email) I will continue the formal process of issuing the RAIs to STPNOC.

These RAIs relates to sections 4.2, 4.3, and 4.6.

Thanks
Tekia

Tekia V. Govan, Project Manager
U.S. Nuclear Regulatory Commission
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Hearing Identifier: SouthTexas34Public_EX
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Subject: DRAFT RAIs
Sent Date: 5/13/2009 12:39:26 PM
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From: Govan, Tekia

Created By: Tekia.Govan@nrc.gov

Recipients:
"STPCOL" <STP.COL@nrc.gov>
Tracking Status: None
"'jeprice@stpegs.com'" <jeprice@stpegs.com>
Tracking Status: None

Post Office: HQCLSTR01.nrc.gov

Files	Size	Date & Time
MESSAGE	586	5/13/2009 12:39:27 PM
RAI 2370.doc	30202	
RAI 2364.doc	30202	
RAI 2578.doc	30202	
RAI 2580.doc	30202	
RAI 2579.doc	30202	

Options
Priority: Standard
Return Notification: No
Reply Requested: No
Sensitivity: Normal
Expiration Date:
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Request for Additional Information No. 2370 Revision 2

South Texas Project Units 3 and 4
South Texas Project Nuclear Operating Co
Docket No. 52-012 and 52-013
SRP Section: 04.06 - Functional Design of Control Rod Drive System
Application Section: 4.6.1.2.4.1

QUESTIONS for Reactor System, Nuclear Performance and Code Review (SRSB)

04.06-***

In Section 4.6.1.2.4.1, it is stated that "Approximately 4 L/min purge flow is provided to the NBS reference leg instrument lines." Provide the basis (assumptions, boundary conditions, references) for this value.

Request for Additional Information No. 2364 Revision 2

South Texas Project Units 3 and 4
South Texas Project Nuclear Operating Co
Docket No. 52-012 and 52-013
SRP Section: 04.06 - Functional Design of Control Rod Drive System
Application Section: 4.6.1.2.3

QUESTIONS for Reactor System, Nuclear Performance and Code Review (SRSB)

04.06-***

In Section 4.6.1.2.3, "Hydraulic Control Units", Standard Departure 4.6-1 deletes the phrase "a small pump and associated", so that the text would read "The test fixture contains hydraulic controls to pressurize the underside of the hollow piston." Since hydraulic controls alone cannot perform the pressurization function, please clarify the intended description of the special test fixture.

Request for Additional Information No. 2578 Revision 0

South Texas Project Units 3 and 4
South Texas Project Nuclear Operating Co
Docket No. 52-012 and 52-013
SRP Section: 04.03 - Nuclear Design
Application Section: 4A. Typical Control Rod Patterns and Associated Power Distribution for ABWR

QUESTIONS for Reactor System, Nuclear Performance and Code Review (SRSB)

04.03-***

The STP-3/4 COL application states that no exceptions were taken to Appendix 4A of the generic ABWR DCD. However, Appendix 4A of the STP-3/4 COL application shows inconsistencies in the MLHGR values in Figure 4A-1a and in the Integrated Power per Bundle in Figure 4A-1d. Provide an explanation for these inconsistencies between the STP-3/4 COL application and the generic ABWR DCD.

Request for Additional Information No. 2580 Revision 0

South Texas Project Units 3 and 4
South Texas Project Nuclear Operating Co
Docket No. 52-012 and 52-013
SRP Section: 04.02 - Fuel System Design
Application Section: 4D. Reference Fuel Design Compliance with Acceptance Criteria

QUESTIONS for Reactor System, Nuclear Performance and Code Review (SRSB)

04.02-***

The standard administrative departure in STP-3/4 COL was identified to be in Table 4D-1 on pg. 4D-7. Page 4D-7 refers to the page in the ABWR DCD. This corresponds to page 4D-5 in the STP-3/4 COL. Provide clarification of this inconsistency in the STP-3/4 COL text.

Request for Additional Information No. 2579 Revision 0

South Texas Project Units 3 and 4
South Texas Project Nuclear Operating Co
Docket No. 52-012 and 52-013
SRP Section: 04.02 - Fuel System Design
Application Section: 4B. Fuel Licensing Acceptance Criteria

QUESTIONS for Reactor System, Nuclear Performance and Code Review (SRSB)

04.02-***

The first full paragraph under Section 4B.2 references “Section 4.2.II.D.3 of the Standard Review Plan (SRP)”. This should be Section 4.2.II.4.C. Please explain the discrepancy and correct as necessary.