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ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

**BELL BEND NUCLEAR POWER PLANT
RESPONSE TO RAI SET NO. 5
BNP-2009-089 Docket No. 52-039**

References: 1) M. Canova (NRC) to R. Sgarro (PPL Bell Bend, LLC), Bell Bend COLA – Request for Information No. 5 (RAI No. 5) – CIB1-2682, email dated May 21, 2009

The purpose of this letter is to respond to the request for additional information (RAI) identified in the referenced NRC correspondence to PPL Bell Bend, LLC. This RAI addresses Pressure-Temperature Limits, Upper-Shelf Energy, and Pressurized Thermal Shock, as discussed in Section 5.3 of the Final Safety Analysis Report (FSAR), as submitted in Part 2 of the Bell Bend Nuclear Power Plant Combined License Application (COLA).

The enclosure provides our response to RAI No. 5, Question 05.03.02-1, which includes revised COLA content. A Licensing Basis Document Change Request has been initiated to incorporate this change in a future revision of the COLA. This future revision of the COLA is the only new regulatory commitment.


The response to Question 05.03.02-1 is consistent with the R-COLA (CCNPP3) response to RAI 77, Question 05.03.03-1.

If you have any questions, please contact the undersigned at 570-802-8102.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on June 09, 2009

Respectfully,


Rocco R. Sgarro

Enclosure: As stated

D079
NRO

cc: Mr. Samuel J. Collins
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Enclosure 1

Response to NRC Request for Additional Information Set No. 5
Bell Bend Nuclear Power Plant

RAI Set No. 5

Question 05.03.02-1

The as-procured reactor vessel material properties will be available to the COL holder after the acceptance of the reactor vessel. In order to provide sufficient time for projected values of reference temperature pressurized thermal shock (RT_{PTS}) values, using the as-procured reactor vessel material properties to be verified by the NRC, the staff requests that a more specific and timely milestone for submitting the pressurized thermal shock (PTS) evaluation to the NRC staff. Therefore, the NRC staff proposes that a license condition will be added, related to Section 5.3.3 that states, within a reasonable period of time following acceptance of the reactor vessel (e.g., 1 year after acceptance of the reactor vessel), the COL holder will submit to the NRC staff its plant-specific, projected values of RT_{PTS} for each reactor vessel beltline material for the end-of-life fluence of the material.

Response

The Bell Bend COLA will be updated as part of the next revision to include a license condition requiring submittal of the plant-specific PTS evaluation to the NRC staff within one year of acceptance of the reactor vessel. This RAI is similar to Calvert Cliffs Unit 3 RAI No. 77. In this RAI, the NRC proposed that a similar license condition be added to FSAR Section 5.3.3, Reactor Vessel Integrity. The Calvert Cliffs Unit 3 COLA is the Reference COLA (R-COLA) for Bell Bend and the proposed license condition was added as generic text in FSAR Section 5.3.3. This proposed modification for generic text has been endorsed by Bell Bend and the proposed license condition will be added to FSAR Section 5.3.3 to maintain continuity with the R-COLA.

COLA Impact

COLA Part 02 (FSAR) Table 1.1-1, Acronyms Used in this Document, and Section 5.3.3, Reactor Vessel Integrity will be revised as follows:

PST	Pre-Service Testing
PTS	Pressurized Thermal Shock
PW	Pittsburgh - Washington Lineament

Section 5.3.3 REACTOR VESSEL INTEGRITY

~~No departures or supplements.~~ The plant-specific pressurized thermal shock (PTS) evaluation will be submitted to the NRC within one year of acceptance of the reactor vessel.

COLA Part 10 (ITAAC) will include an additional item in Appendix A, PROPOSED COMBINED LICENSE CONDITIONS as follows:

11. REACTOR VESSEL INTEGRITY

COL application FSAR Section 5.3.3 requires submittal of the plant-specific pressurized thermal shock (PTS) evaluation to the NRC within one year of acceptance of the reactor vessel.

PROPOSED LICENSE CONDITION

The plant-specific PTS evaluation will be submitted to the NRC within one year of acceptance of the reactor vessel.